



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DESIGNATED ORIGINAL

Docket Nos. 50-445
and 50-446

April 4, 1991

certified By

LICENSEE: Texas Utilities Electric Company (TU Electric)
FACILITY: Comanche Peak Steam Electric Station, Units 1 and 2 (CPSES)
SUBJECT: SUMMARY OF MEETING ON NRC REVIEW SCHEDULE FOR TUE RELOAD
ANALYSIS METHODOLOGY REPORTS

A meeting was held on March 14, 1991 to discuss NRC's review schedule for TU Electric's reload analysis methodology reports. The meeting was a followup to the July 24, 1990, meeting held on the same topic to review the current status, including the recent licensee submittals of the Transient Analysis Methods and Power Distribution Control Analysis reports.

The NRC staff noted that TU Electric has submitted all reports to date on or before the schedule discussed at the July 24, 1990, meeting. The NRC staff indicated that completion of NRC review of Cycle 2 reports was currently expected in July 1991. The NRC staff also indicated that current schedules would result in completion of Cycle 3 reports no earlier than the end of June 1992.

The licensee stated that their current refueling schedule calls for the first refueling outage to start in September 1991, and the second to start in September 1992. The licensee noted that the current review schedule of Cycle 3 reports would not provide approved methods in time to conduct the required analyses to support submittal of the Cycle 3 reload report in May 1992 prior to the second refueling outage. The licensee stated their desire to obtain NRC approval of the analysis methodologies by February 1992, which is the latest the utility felt it could start its analyses to support a May 1992 reload report. The licensee also asked if there was anything they could do to expedite the review process, including meeting with the reviewers during the review process to reduce the time to provide clarifications or address open items. The licensee also asked if an alternative method of having TU Electric conduct the analyses, and the original vendor (ANF) conduct an independent review/certification of TU Electric's application of the methods and the results, as a way of allowing TU Electric to use its own analysis methods while providing a method for NRC to accept the results in time to support use of the analyses for Cycle 3 reload. The NRC staff indicated it would evaluate these alternatives and provide feedback to the licensee.

The licensee also provided a background discussion on the similarities of its analysis methods to NRC approved methods. TU Electric indicated they are using RELAP 5 Mod 2 for small break LOCA and RELAP 4 for the large break LOCA, using a 5-node model instead of the more standard 3-node model. They are using an

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unmodified VIPRE-01 code for thermal-hydraulic analysis, but were unsure if this code had been applied elsewhere. They are using a self-generated DNB correlation (TUE-1) code, but TU Electric feels it is consistent with industry codes. The licensee has submitted a supplement to this code, and is planning another to accommodate the larger fuel they have been using to date, and plan to use in the future to take advantage of the fuel's better mixing characteristics. TU Electric is using the same version of RETRAN that was used to analyze steam generator tube rupture and other events, including main steam line breaks, for licensing.

The NRC staff and the licensee discussed general alternatives to the current situation, including: (1) whether the NRC is able to expedite its reviews, (2) the licensee conducting its analyses concurrent with NRC review of the analysis methods, and (3) TU Electric contracting with an approved vendor to conduct the analyses with approved generic codes, which TU Electric estimates would cost about \$1.5 million for Cycle 3. The NRC staff emphasized that the review schedule for these plant-specific codes was dependent on their relative safety significance, and that although the NRC staff recognizes that conduct of analyses by approved generic vendors duplicated the plant-specific capabilities and was very costly, approval of the plant-specific analysis methods by the NRC staff in time to support the Cycle 3 analyses could not be assured.

Original signed by
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Enclosure:
Attendance List

cc w/enclosure:
See next page

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ENCLOSURE

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