UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-388

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. NPF-22

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated October 2, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-22 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 72 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, and shall be implemented before startup from the Unit 2 Cycle 5 refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 28, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 72

FACILITY OPERATING LICENSE NO. NPF-22

DOCKET NO. 50-388

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The overleaf pages are provided to maintain document completeness.*

REMOVE	INSERT
3/4 6-21	3/4 6-21*
3/4 6-22	3/4 6-22
3/4 6-27	3/4 6-27*
3/4 6-28	3/4 6-28

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	MAXIMUM ISOLATION TIME (Seconds)	ISOLATION SIGNAL(S)[a]
Automatic Isolation Valves (Continued)		
Containment Atmosphere Sample		
SV-25734 A,B	N/A	B,Y
SV-25736 A	N/A	B,Y
SV-25736 B	N/A	B,Y
SV-25740 A,B	N/A	B,Y
SV-25742 A,B	N/A	B,Y
SV-25750 A,B	N/A	B,Y
SV-25752 A,B	N/A	B,Y
SV-25774 A,B	N/A	B,Y
SV-25776 A	N/A	B,Y
SV-25776 B	N/A	B,Y
SV-25780 A,B	N/A	B,Y
SV-25782 A,B	N/A	B,Y
Nitrogen Makeup		
SV-25737	N/A	B,Y,R
SV-25738	N/A	B,Y,R
SV-25767	N/A	B,Y,R
SV-25789	N/A	B,Y,R
Reactor Coolant Sample		
HV-243F019	2	B,C
HV-243F020	2	B,C
Liquid Radwaste		
HV-26108 A1,A2	15	B,Z
HV-26116 A1,A2	15	B,Z
RHR - Suppression Pool Cooling/Spray[c]		
HV-251F028 A,B	90	X,Z
CS Test[b][c]		
HV-252F015 A,B	60	X,Z
HPCI Suction[b][c]		
HV-255F042	90	L,LB

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	MAXIMUM ISOLATION TIME (Seconds)	ISOLATION SIGNAL(S)[a]
Automatic Isolation Valves (Continued)		
Suppression Pool Cleanup[b]		
HV-25766	35	A,Z
HV-25768	30	A,Z
HPCI Vacuum Breaker		
HV-255F075	15	LB,Z
HV-255F079	15	LB,Z
RCIC Vacuum Breaker		
HV-249F062	10	KB,Z
HV-249F084	10	KB,Z
TIP Ball Valves (d)		
C51-J004 A,B,C,D,E	5	A,Z
Containment Radiation Detection System		
SV-257100 A,B	N/A	(f)
SV-257101 A,B	N/A	(f)
SV-257102 A,B	N/A	(f)
SV-257103 A,B	N/A	(f)
SV-257104	N/A	(f)
SV-257105	N/A	(f)
SV-257106	N/A	(f)
SV-257107	N/A	(f)
b. Manual Isolation Valves		
MSIV-LCS Bleed Valve		
HV-239F001 B,F,K,P		
Feedwater[e]		
HV-241F032 A,B		
RWCU Return		
HV-24182 A,B		
RCIC Injection		
HV-249F013		
2-49-020		

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	
Excess Flow Check Valves (Continued)	
Reactor Recirculation	
XV-243F003	A,B
XV-243F004	A,B
XV-243F009	A,B,C,D
XV-243F010	A,B,C,D
XV-243F011	A,B,C,D
XV-243F012	A,B,C,D
XV-243F040	A,B,C,D
XV-243F057	A,B
Nuclear Boiler Vessel Instrument	
XV-242F041	
XV-242F043	A,B
XV-242F045	A,B
XV-242F047	A,B
XV-242F051	A,B,C,D
XV-242F053	A,B,C,D
XV-242F055	
XV-242F057	
XV-242F059	A,B,C,D,E,F,G,H,L,M,N,P,R,S,T,U
XV-242F061	
XV-24201	
XV-24202	
Nuclear Boiler	
XV-241F070	A,B,C,D
XV-241F071	A,B,C,D
XV-241F072	A,B,C,D
XV-241F073	A,B,C,D
XV-241F009	
MSIVLCS	
XV-23910	B,F,K,P

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES
NOTATION

(a) See Specification 3.3.2, Table 3.3.2-1, for isolation signal(s) that operates each automatic isolation valve. All power-operated isolation valves may be opened or closed remote-manually.

(b) Isolation barrier remains water filled or a water seal remains in the line post-LOCA. Isolation valve is tested with water. Isolation valve leakage is not included in 0.60 L_a total Type B and C tests.

(c) Redundant isolation boundary for this valve is provided by the closed system whose integrity is verified by Type A test.

(d) Automatic isolation signal causes TIP to retract; ball valve closes when probe is fully retracted.

(e) Power assisted check valve.

(f) Solenoid valves not capable of being opened due to the absence of permanently installed electrical power.