VOLUME III

OMAHA PUBLIC POWER DISTRICT - FORT CALHOUN STATION

EMERGENCY PLAN IMPLEMENTING PROCEDURES

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Fort Calhoun Station - Unit No. 1
Emergency Plan Implementing Procedure
EPIP-OSC-8

MEDICAL ASSISTANCE

Method for obtaining quick medical assistance, primarily transportation and treatment, for major personal injuries at the Fort Calhoun Station.

I. PURPOSE

This procedure provides the instruction to identify personal injuries, procure emergency first aid and receive specialized medical treatment.

II. PREREQUISITE

A. The injured person has been moved, if possible, from a radiation area or shielding installed to reduce radiation exposure.

B. The injured person has been decontaminated and protective clothing removed, if possible.

IV. PRECAUTION

A. In case of severe injury, first aid and medical assistance takes precedence over contamination control.

B. The injured person should not be left unattended. A fellow employee should remain with the victim for first aid and encouragement until full treatment is provided.

V PROCEDURE

1. Injuries sustained outside the radiation control areas shall receive medical assistance in accordance with Standing Order G-14, Personal Injuries.

2. Locate the injured person and ensure this information is available at the Control Room and Emergency Operation Facility.

3. Minor injuries should be treated at the Station First Aid facilities. Follow up service and major injuries should be handled in accordance with the remainder of this procedure.

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V. PROCEDURE (Continued)

4. If a major radiological injury was received or it has not been determined that the person is contamination free, contact the Blair Rescue Squad (Telephone 911) for advanced first aid and emergency transportation. Emphasize that the injury occurred at the Fort Calhoun Power Station, O.P.P.D.

5. Notify the University of Nebraska Medical Center, Radiation Treatment Facility (Telephone 559-5299) that a radiation accident has occurred at Fort Calhoun Station and the hospital will receive the injured person in approximately 45 minutes. Describe known radiological and medical information available.

6. Activate Emergency Team Members 2 and 3, if necessary, to perform re-entry and rescue activities. Operators and C/RP Technicians should be utilized if the Emergency Team has not reported.

7. Activate Emergency Team Member 18 to perform Rescue Squad Monitor activites. An Operator or C/RP Technician should be utilized if the Emergency Team has not reported.

8. Ensure the location of the injured person is coordinated. Monitor 18 and the Security Force should share this information.

9. Provide the UMMC with additional information after the Rescue Squad vehicle has departed the site.

10. Provide notification and report to Production Operations Management through the Training and Safety Coordinator that an injury requiring hospital assistance has occurred.

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FORT CALHOUN STATION UNIT NO. 1
Emergency Plan Implementing Procedure
EPIP-EOF-14

EDO to Recovery Manager Transition

I. PURPOSE

This procedure provides the instruction to be followed by the EDO and Recovery Manager regarding the EDO to Recovery Manager transition.

II. PREREQUISITE

A. The emergency has been placed into one of the three higher emergency classifications requiring activation of the Recovery Organization.

NOTE: Alert classification does not necessitate full Recovery Organization activation.

B. Recovery Organization activation has been accomplished in accordance with Implementing Procedure EPIP-RR-1.

III. PRECAUTIONS

A. All significant events and actions shall be logged in the operations log book.

B. The Emergency Operations Facility has been determined to be habitable.

C. The transition may occur by telephone from two different emergency facilities. Ensure all communications are clear and understood.

IV. PROCEDURE

A. The Recovery Manager shall review the following information during the EDO to Recovery Manager transition.

1. Current Plant Status:

_____Review primary system pressure and temperature status.

_____Review primary system chemistry parameters to determine the extent of core damage if any.

_____Review operational status of all available heat sinks that could be employed for the removal of decay heat.

_____Review operational status of nuclear instrumentation systems.

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IV. PROCEDURE (Continued)

_____Review any abnormal situations involving plant systems and alignments.

_____Review the emergency operations log book

2. Radiological Assessments Status:

_____Review current radiological release data, including meteorological data, activity release estimates and dose assessments.

_____Review current on-site and off-site radiation, contamination and airborne surveys.

_____Review the graphics of present emergency conditions and projections on the status board.

3. Emergency Actions:

_____Review status of any in-plant evacuation measure that may have been taken including an accountability of plant personnel and visitors.

_____Review status of any contaminated and/or injured personnel that may have been transported to UNMC or Blair Memorial Hospital for treatment.

4. Notification and Recommendations:

_____Determine emergency notification status of outside emergency response organizations.

_____Review recommended protective actions that may have been transmitted to local and state governments.

5. Status of Emergency Response Organizations:

_____Review activation status of the Emergency Team and Technical Augmentation Staff.

_____Determine activation status of Operation Support Center.

_____Determine activiation status of Technical Support Center.

_____Determine that the activation status of the remainder of the Recovery Organization is progressing smoothly and that the Emergency Team and Technical Augmentation Staff are functioning within this organization.

IV. PROCEDURE (Continued)

B. 1. Following the review of this pertinent information the Recovery Manager will relieve the EDO of emergency duties, responsibilities and authority. _____

2. The EDO replaced by the Recovery Manager will assist the Recovery Manager as an ex-officio member of the Recovery Manager staff until re-assigned by the Recovery Manager. _____

Fort Calhoun Station Unit No. 1
Energency Plan Implementing Procedure
EPIP-EOF-15

Determination of Contamination Release Through Plant Stack

I. PURPOSE

To provide a relationship between the effluent radiation monitor readings and the amount of contamination on the ground and the resulting whole body gamma dose rates.

II. PREREQUISITES

A. The Emergency Operations Facility has been activated per EPIP-EOF-1.

III. PRECAUTIONS

NONE

IV. PROCEDURE

1. Project the amount of contamination deposited on the ground at selected downwind location(s).

   a. Obtain the wind direction at 10 meters from the control room and enter in Table EOF-15.1.

   b. Determine the affected wind sector with, letter designation A to R, using the wind direction data from Step 1.a, above. (affected wind sector is the sector opposite to the wind direction sector). Enter the affected wind sector in Table EOF-15.1.

   c. Obtain the stack flow rate from the control room and enter in Table EOF-15.1; convert this value to m^3/sec and enter in Table EOF-15.1.

   d. Determine the relative deposition factor (D/Q) for a selected downwind distance from Table EOF-15.2 and enter in Table EOF-15.1.

      NOTE: If the selected downwind distance is between two of the listed distances, use the next SMALLER distance.

   e. Determine the duration of a release from the stack and enter in Table EOF-15.1.

   f. Draw air samples on particulate filters from the stack via RM-060 and RM-061 air pumps.

   g. Remove particulate filters and return to the laboratory for performance of isotopic analysis of radionuclides listed in Table EOF-15.1 and enter the data in Column II of Table EOF-15.1.

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h. Calculate the radionuclides release rates by multiplying values in Column II by the stack flow rate and enter the data in Column III of Table EOF-15.1.

i. Calculate the contamination deposition by multiplying the values in Column III by D/Q and the release duration; enter the data in Column IV of Table EOF-15.1.

2. Project Whole Body Gamma Dose Rates from radionuclide deposited on the ground.

   a. Multiply the contamination deposition in Column IV by the dose conversion factor for each radionuclide from Column V and enter the data in Table EOF-15.1.

   b. Sum the whole body dose rates values and Column VI and enter in Table EOF-15.1.

3. Refer to Table EOF-15.3 for protective action guidelines for accumulated Whole Body doses received from the contamination deposited on ground.

4. Notify the Emergency Coordinator of the above evaluated results.

5. Reassess the situation and/or repeat Steps 1 through 3 as necessary.

TABLE EOF-15.1
Amount of Contamination on the Ground and
Whole Body Gamma Dose Rates

1. Wind Direction at 10 meters from Control Room: _____ Degrees at _____ hrs.
2. Affected Wind Sector: ____ (Letter designation A to R)
3. Stack Flow Rate from Control Room: _____ cfm x 4.72 E-04 = _____ m^3/Sec
4. Relative Deposition Factor (D/Q) from Table EOF-15.2: _____ m^{-2} at _____ miles
5. Release Duration (Projected or actual): _____ Sec

I	II		III	IV	V	VI
Radionuclide	Isotopic Analysis (µCi/cc)		Release Rate (Ci/Sec) (Column II x Stack Flow Rate)	Contamination Deposition (Ci/m^2) (Column III x D/Q x Release Duration)	Dose Conversion Factor (mrem-m^2/Hr-Ci)	Whole Body Dose (mrem/hr) (Column IV x Column V)
	Using RM-060	Using RM-061				
Mn-54					5.8E+03	
CO-60					1.7E+04	
Sr-89					5.6E-01	
I-131					2.8E+03	
I-133					3.7E+03	
I-135					1.2E+04	
Cs-134					1.2E+04	
Cs-137					4.2E+03	
Ba-140					2.1E+03	
La-140					1.5E+04	
				Total Whole Dose Rate =		

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TABLE EOF-15.2
Relative Deposition Factors (m^{-2})

Wind Direction (Degrees)	Affected Sector	0.50 Mile	1.50 Mile	2.50 Mile	3.50 Mile	4.50 Mile	7.50 Mile	15.0 Mile	25.0 Mile	35.0 Mile	45.0 Mile
169 - 191	A	6.7E-08	5.9E-09	1.7E-09	7.6E-10	4.5E-10	1.6E-10	5.3E-11	1.9E-11	1.1E-11	6.6E-12
191 - 214	B	5.2E-08	4.6E-09	1.3E-09	5.9E-10	3.5E-10	1.2E-10	4.1E-11	1.5E-11	8.2E-12	5.1E-12
214 - 236	C	3.3E-08	2.9E-09	8.4E-10	3.8E-10	2.2E-10	7.9E-11	2.6E-11	9.5E-12	5.2E-12	3.3E-12
236 - 259	D	2.5E-08	2.2E-08	6.2E-10	2.8E-10	1.6E-10	5.8E-11	1.9E-11	7.0E-12	3.9E-12	2.4E-12
259 - 281	E	2.9E-08	2.6E-09	7.4E-10	3.3E-10	2.0E-10	6.9E-11	2.3E-11	8.3E-12	4.6E-12	2.9E-12
281 - 304	F	5.9E-08	5.2E-09	1.5E-09	6.7E-10	3.9E-10	1.4E-10	4.6E-11	1.7E-11	9.2E-12	5.8E-12
304 - 326	G	8.8E-08	7.7E-09	2.2E-09	1.0E-9	5.9E-10	2.1E-10	6.9E-11	2.5E-11	1.4E-11	8.6E-12
326 - 349	H	7.5E-08	6.6E-09	1.9E-09	8.5E-10	5.0E-10	1.8E-10	5.9E-11	2.2E-11	1.2E-11	7.4E-12
349 - 11	J	7.1E-08	6.3E-09	1.8E-09	8.1E-10	4.8E-10	1.7E-10	5.6E-11	2.0E-11	1.1E-11	7.0E-12
11 - 34	K	3.7E-08	3.3E-09	9.4E-10	4.2E-10	2.5E-10	8.8E-11	2.9E-11	1.1E-11	5.8E-12	3.6E-12
34 - 56	L	1.9E-08	1.7E-09	4.8E-10	2.2E-10	1.3E-10	4.5E-11	1.5E-11	5.5E-12	3.0E-12	1.9E-12
56 - 79	M	1.8E-08	1.6E-09	4.5E-10	2.0E-10	1.2E-10	4.2-11	1.4E-11	5.1E-12	2.8E-12	1.7E-12
79 - 101	N	2.1E-08	1.9E-09	5.4E-10	2.4E-10	1.4E-10	5.1E-11	1.7E-11	6.2E-12	3.4E-12	2.1E-12
101 - 124	P	4.0E-08	3.5E-09	1.0E-09	4.5E-10	2.7E-10	9.5E-11	3.1E-11	1.1E-11	6.3E-12	3.9E-12
124 - 146	Q	7.0E-08	6.1E-09	1.8E-09	7.9E-10	4.7E-10	1.7E-10	5.5E-11	2.0E-11	1.1E-11	6.9E-12
146 - 169	R	6.6E-08	5.8E-09	1.7E-09	7.5E-10	4.5E-10	1.6E-10	5.2E-11	1.9E-11	1.0E-11	.5E-12

Table EOF-15.3
Whole Body Dose Protective Action Guidelines

Projected Dose to Population (Rem)	Recommended Actions	Comments
<1	No planned protective action. State may issue advisory to seek shelter await further instructions. Monitor environmental radiation levels.	Previously recommended actions may be reconsidered or terminated.
1 to <5	Seek shelter as a minimum. Consider evacuation. Evacuate unless constraints make it impractical. Monitor environmental radiation levels. Control access.	If constraints exist, consideration should be given to evacuation of pregnant women & children.
<5	Conduct mandatory evacuation. Monitor environmental radiation levels and adjust area for mandatory evacuation based on these levels. Control access.	Seeking shelter would be an alternative if evacuation were not possible.

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Fort Calhoun Station Unit No. 1
Emergency Plan Implementing Procedure
EPIP-TSC-1

ACTIVATION OF TECHNICAL SUPPORT CENTER

Method for activation of the Technical Support Center (TSC) for emergency operations assistance during Alert, Site Area and General Emergency classes.

I. PURPOSE

This procedure provides the instruction for the TSC Manager and his staff to initiate support to the reactor operating personnel in administrative, communication, and technical evaluation functions.

II. PREREQUISITES

A. The emergency has been classified as Alert, Site or General (Procedure EPIP-OSC-1) by the Shift Supervisor or Emergency Duty Officer.

B. Personnel assigned operational, mechanical, electrical and radiological emergency support have been adequately trained.

III. PRECAUTIONS

A. Although activating the TSC is optional for 'Notification of Unusual Event' emergency classes, it is advisable that TSC members report to the plant when notified to share their expertise.

B. The TSC is sized for 20 technical persons. The TSC Manager must limit off-site support to ensure optimum working space.

IV. PROCEDURE

1. Check and initiate operations of radiological equipment to ensure monitoring capability and continued habitability of TSC.

2. Establish communications with control room and emergency response facilities (EPIP-TSC-2).

3. Provide plant and reactor operation support in accordance with implementing procedures EPIP-TSC-3, 4, 5 and 6.

4. Relieve reactor operator of auxiliary duties and communications not directly related to reactor system manipulations.

5. Maintain emergency offsite support group coordination until activation of EOF has been confirmed.

6. Review technical data displays and plant records available. (A Safety Parameter Display System will eventually standardize information for safety assessment.)

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IV. PROCEDURE (Continued)

7. Provide detailed analysis and diagnosis of abnormal plant conditions to control room operations.

8. Evaluate any significant release of radioactivity from the plant.

9. Maintain open communication with the control room and EOF.

10. Establish a long term schedule for staffing the TSC and control room with additional manpower if the emergency class is increasing or a duration of more than 24 hours is anticipated.

FORT CALHOUN STATION UNIT NO.1

Emergency Plan Implementing Procedure
EPIP-TSC-4
TECHNICAL SUPPORT CENTER

PLANT AND REACTOR OPERATION SUPPORT - SITE AREA EMERGENCY CLASSIFICATION

I. PURPOSE

This instruction provides general guidelines to Technical Support Center personnel as to the type of support/assistance that may be requested during an incident classified as a Site Area Emergency.

II. PREREQUISITES

A. Emergency classification has been defined per EPIP-OSC-1.

B. Technical Support Center has been activated per EPIP-TSC-1.

III. PRECAUTIONS

None

IV. PROCEDURE

1. Render support/assistance as requested by OSC.

   Example: Monitor primary coolant temperature/pressure instrumentation and maintain minimum of 50° subcooling for particular system pressure.

2. Establish contact with Region IV NRC, A-E, C-E.

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Fort Calhoun Station Unit No. 1
Emergency Plan Implementing Procedure
EPIP-TSC-6
TECHNICAL SUPPORT CENTER

PLANT ENGINEERING AND REPAIR

I. PURPOSE

This instruction provides general guidelines to the Technical Support Center personnel to perform engineering work, and repair supervision after any incident requiring activation of the TSC that caused equipment damage.

II. PREREQUISITES

A. Emergency classification has been defined per EPIP-OSC-1

B. Technical Support Center has been activated per EPIP-TSC-1

C. The recovery organization has been activated per EPIP-RR-1

D. Incident emergency status has been terminated and repair operations are beginning.

III. PRECAUTIONS

None

IV. PROCEDURE

1. Evaluate extent of damage resulting from incident.

2. Propose repair schedules, based upon evaluation of Step 1, to expedite recovery and return to power operation.

3. Implement repair schedules.

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