

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20565

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77 License No. NPF-11

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated July 26, 1989, as supplemented on July 9, 1990. December 5, 1990. January 2, 1991, and January 18, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without engangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license ar andment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 77, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

and

Richard J. Barrett, Director Project Directorate III-2 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 21, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 77

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 3-11	3/4 3-11
3/4 3-14(a)	3/4 3-14(a)
B 3/4 3-2	B 3/4 3-2

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

TRIP	FUNCTION		VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (b)	APPLICABLE OPERATIONAL CONDITION	ACTION		
Α.	AUT	MATIC INITIATION						
1.	PRI	MARY CONTAINMENT ISOLATION						
	a.	Reactor Vessel Water Level (1) Low, Level 3 (2) Low Low, Level 2 (3) Low Low Low, Level 1	7 2, 3 1, 10	2 2 2	1, 2, 3 1, 2, 3 1, 2, 3	20 20 20		
	b.	Drywell Pressure - High	2, 7, 10	2	1, 2, 3	20		
	c.	Main Steam Line 1) Radiation - High 2) Pressure - Low	1 3 1	2 2 2 2(1):(d)	1, 2, 3 1, 2, 3 1	21 22 23		
		3) Flow - High	1	2/11ne	1, 2, 3	21		
	d.	Main Steam Line Tunnel Temperature - High	1	2	$1^{(i)(j)}, 2^{(i)(j)}, 3^{(i)(j)}, 2^{(i)(j)}, 3^{(i)(j)}$	21		
	e.	Main Steom Line Tunnel ∆Temperature - High	1	2	$\binom{1^{(i)(j)}, 2^{(i)(j)}}{3^{(i)(j)}}$	21		
	f.	Condenser Vacuum - Low	1	2	1, 2*, 3*	21		
2.	SECONDARY CONTAINMENT ISOLATION							
	ā.	Reactor Building Vent Exhaust Plenum Radiation - High	4(c)(e)	2	1, 2, 3 and **	24		
	b.	Drywell Pressure - High	4(c)(e)	2	1, 2, 3	24		
	c.	Reactor Vessel Water Level - Low Low, Level 2	4(c)(e)	2	1, 2, 3, and #	24		
	d.	Fuel Pool Vent Exhaust Radiation - High	4(c)(e)	2	1, 2, 3, and **	24		

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Amendment No. 77

TABLE 3.3.2-1 (Continued)

NOTES (Continued)

- (g) Requires RCIC steam supply pressure-low coincident with drywell pressure-high.
- (h) Manual initiation isolates 1E51-F008 only and only with a coincident reactor vessel water level-low, level 2, signal.
- (i) Both channels of each trip system may be placed in an inoperable status for up to 4 hours for required reactor building ventilation system corrective maintenance, filter changes, damper cycling and surveillance tests, other than Surveillance Requirement 4.6.5.1.c, without placing the trip system in the tripped condition.
- (j) Both channels of each trip system may be placed in an inoperable status for up to 12 hours for performance of Surveillance Requirement 4.6.5.1.c without placing the trip system in the tripped condition.

INSTRUMENTATION

BASES

3/4.3.2 ISOLATION ACTUATION INSTRUMENTATION

This specification ensures the effectiveness of the instrumentation used to mitigate the consequences of accidents by prescribing the OPERABILITY trip setpoints and response times for isulation of the reactor systems. When necessary, one channel may be inoperable for brief intervals to conduct required surveillance. Both channels of each trip system for the main steam tunnel ambient temperature and ventilation system differential temperature may be placed in an inoperable status for up to 4 hours for required reactor building ventilation system maintenance and testing and 12 hours for the required secondary containment Leak Rate test without placing the trip system in the tripped condition. This will allow for maintaining the reliability of the ventilation system and secondary containment. Some of the trip settings may have tolerances explicitly stated where both the high and low values are critical and may have a substantial effect on safety. The setpoints of other instrumentation, where only the high or low end of the setting have a direct bearing on safety, are established at a level away from the normal operating range to prevent inadvertent actuation of the systems involved.

Except for the MSIVs, the safety analysis does not address individual sensor response times or the response times of the lugic systems to which the sensors are connected. For D.C. operated valves, a 3 second delay is assumed before the valve starts to move. For A.C. operated valves, it is assumed that the A.C. power supply is lost and is restored by startup of the emergency diesel generators. In this event, a time of 13 seconds is assumed before the valve starts to move. In addition to the pipe break, the failure of the D.C. operated valve is assumed; thus the signal delay is concurrent with the 13 second diesel startup. The safety analysis considers an allowable inventory loss in each case which in turn determines the valve speed in conjunction with the 13 second delay. It follows that checking the valve speeds and the 13 second time for emergency rower establishment will establish the response time for the isolation functions. However, to enhance overall system reliability and to monitor instrument channel response time trends, the isolation actuation instrumentation response time shall be measured and recorded as a part of the ISOLATION SYSTEM RESPONSE TIME.

3/4.3.3 EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

The emergency core cooling system actuation instrumentation is provided to initiate actions to mitigate the consequences of accidents that are beyond the ability of the operator to control. This specification provides the OPERABILITY requirements, trip setpoints and response times that will ensure effectiveness of the systems to provide the design protection. Although the instruments are listed by system, in some cases the same instrument may be used to send the actuation signal to more than one system at the same time.

LA SALLE - UNIT 1



UNITED STATES NUCLEAR REGULATORY COMMISSION

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61 License No. NPF-18

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated July 26, 1989, as supplemented on July 9, 1990, December 5, 1990, January 2, 1991, and January 18, 1991, Complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter 1;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 61, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director Project Prectorate III-2 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 21, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 61

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT		
3/4 3-11	3/4 3-11		
3/4 3-14	3/4 3-14		
3/4 3-14(a)	3/4 3-14(a)		
B 3/4 3-2	B 3/4 3-2		

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

TRIP	FUN	ICTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (b)	APPLICABLE OPERATIONAL CONDITION	ACTION		
Α.	AUT	OMATIC INITIATION						
1.	PRI	MARY CONTAINMENT ISOLATION						
	a.	Reactor Vessel Water Level (1) Low, Level 3 (2) Low Low, Level 2 (3) Low Low Low, Level 1	7 2, 3 1, 10	2 2 2	1, 2, 3 1, 2, 3 1, 2, 3	20 20 20		
	b.	Drywell Pressure - High	2, 7, 10	2	1, 2, 3	20		
	c.	Main Steam Line 1) Radiation - High	1 3	2	1, 2, 3 1, 2, 3	21 22		
		2) Pressure - Low 3) Flow - High	1	2 2/line(d)	1, 2, 3	23 21		
	d.	Main Steam Line Turnel Temperature - Clich	1	2	$\binom{1^{(i)(j)}}{3^{(i)(j)}}$	21		
	e.	Main Steam Line Tunnel ∆Temperature - High	1	2	$1^{(i)(j)}, 2^{(i)(j)},$ $3^{(i)(j)}, 2^{(i)(j)},$	21		
	f.	Condenser Vacuum - Low	1	2	1, 2*, 3*	21		
2.	SECONDARY CONTAINMENT ISOLATION							
	a.	Reactor Building Vent Exhaust Plenum Radiation - High	4 ^{(c)(e)}	2	1, 2, 3 and **	24		
	b.	Drywell Pressure - High	4(c)(e)	2	1, 2, 3	24		
	c.	Reactor Vessel Water Level - Low Low, Level 2	4(c)(e)	2	1, 2, 3, and #	24		
	d.	Fuel Pool Vent Exhaust Radiation - High	4(c)(e)	2	1, 2, 3, and **	24		

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LA SALLE - UNIT 2

Amendment No. 61

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

ACTION STATEMENTS

ACTION	20	*	Be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN
ACTION	21	•	Bo in at least STARTUP with the associated isolation valves closed within 6 hours or be in at least HOT SHUTDOWN within
ACTION	22	•	12 hours and in COLD SHUTDOWN within the next 24 hours. Close the affected system isolation valves within 3 hour and declare the affected system incomprable.
ACTION	23		Be in at least STARTUP within 6 hours.
ACTION	24		Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas
ACTION	25		Lock the affected system isolation valves closed within 1 hour
ACTION	26		and declare the affected system inoperable. Provided that the manual initiation function is OPERABLE for
			each other group valve, inboard or outboard, as applicable, in each line, restore the manual initiation function to OPERABLE status within 24 hours; ctherwise, restore the manual initiation function to OPERABLE status within 8 hours; otherwise: a. Be in at least HOT SHUTDOWN within the next 12 hours and in

b. Close the affected system isolation valves within the next hour and declare the affected system in operable.

TABLE NOTATIONS

- * May be bypassed with reactor steam pressure < 1043 psig and all turbine stop valves closed.
- ** When handling irradiated fuel in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- During CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- (a) See Specification 3.6.3, Table 3.6.3-1 for valves in each valve group.
- (b) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the channel in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter. In addition for those trip systems with a design providing only one channel per trip system, the channel may be placed in an inoperable status for up to 8 hours for required surveillance testing without placing the channel in the tripped condition provided that the redundant isolation valve, inboard or outboard, as applicable, in each line is operable and all required actuation instrumentation for that redundant valve is OPERABLE, or place the trip system in the tripped condition.
- (c) Also actuates the standby gas treatment system.
- (d) A channel is OPERABLE if 2 of 4 instruments in that channel are OPERABLE.
- (e) Also actuates secondary containment ventilation isolation dampers per Table 3.6.5.2-1.
- (f) Closes only RWCU system inlet outboard valve.

LA SALLE - UNIT 2

TABLE 3.3.2-1 (Continued)

NOTES (Continued)

- (g) Requires RCIC steam supply pressure-low coincident with drywell pressure-high.
- (h) Manual initiation isolates 2E51-F008 only and only with a coincident reactor vessel water level-low, level 2, signal.
- (i) Both channels of each trip system may be placed in an inoperable status for up to 4 hours for required reactor building ventilation system corrective maintenance, filter changes, damper cycling and surveillance tests, other than Surveillance Requirement 4.6.5.1.c, without placing the trip system in the tripped condition.
- (j) Both channels of each trip system may be placed in an inoperable status for up to 12 hours for performance of Surveillance Requirement 4.6.5.1.c without placing the trip system in the tripped condition.

INSTRUMENTATION

BASES

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Except for the MSIVs, the safety analysis does not address individual sensor response times or the response times of the logic systems to which the sensors are connected. For D.C. operated valves, a 3 second delay is assumed before the valve starts to move. For A.C. operated valves, it is assumed that the A.C. power supply is lost and is restored by startup of the emergency diesel generators. In this event, a time of 13 seconds is assumed before the valve starts to move. In addition to the pipe break, the failure of the D.C. operated valve is assumed; thus the signal delay is concurrent with the 13 second diesel startup. The safety analysis considers an allowable inventory loss in each case which in turn determines the valve speed in conjunction with the 13 second delay. It follows that checking the valve speeds and the 13 second time for emergency power establishment will establish the response time for the isolation functions. However, to enhance overall system reliability and to monitor instrument channel response time trends, the isolation actuation instrumentation response time shall be measured and recorded as a part of the ISOLATION SYSTEM RESPONSE TIME.

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LA SALLE - UNIT 2