EAR REQUES UNITED STATES NUCLEAR REGULATORY COMMISSION REGION II 101 MARIETTA STREET, N.W., SUITE 2900 ATLANTA, GEORGIA 30323-0199 Report Nos.: 50-424/94-05 and 50-425/94-05 Licensee: Georgia Power Company P. O. Box 1295 Birmingham, AL 35201 Decket Nos.: 50-424 and 50-425 License Nos.: NPF-68 and NPF-81 Facility Name: Vogtle 1 and 2 Inspection Conducted: February 20, 1994 - March 19, 1994 4.13.94 Inspector: dein Date Signed Aren Bonser, Senior Resident Inspector Β. 4.13.94 Starkey, Resident Inspector Date Signed 4.13.94 Balmain, Resident Inspector Date Signed 4.13.94 Approved by: Skinner, Chief Date Signed Reactor Projects Section 3B Division of Reactor Projects SUMMARY

Scope: This routine, inspection entailed inspection in the following areas: plant operations, surveillance, maintenance, evaluation of licensee self-assessment, review of corporate engineering and design change support, and follow-up of open items.

Results:

One non-cited violation and a weakness were identified.

The non-cited violation was identified for failure to assign and perform the correct functional tests according to Maintenance Work Order Functional Test and Inservice Test Program procedures. This error occurred on three occasions for maintenance on Unit 1 component cooling water pumps and is considered a weakness in the assignment of functional tests (paragraph 4.d).

The inspectors observed an operator error when control rods were pulled instead of inserted during a monthly rod operability surveillance. The error did not affect core reactivity or safety, since the rods were already fully withdrawn. However this is an example of an error made during a routine activity which should have been prevented by the licensee's self checking program (paragraph 3.b).

A review of corporate engineering support and Safety Review Board activities was performed in Birmingham, Alabama. The review found that the corporate staff supported plant activities well. The review also concluded that the Safety Review Board functioned effectively (paragraphs 5 and 6).

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REPORT DETAILS

1. Persons Contacted Licensee Employees *J. Beasley, General Manager Nuclear Plant S. Bradley, Reactor Engineering Supervisor *R. Brown, Operations Training Supervisor *W. Burmeister, Manager Engineering Support *S. Chesnut, Manager Engineering Technical Support *C. Christiansen, SAER Supervisor C. Coursey, Maintenance Superintendent R. Dorman, Manager Training and Emergency Preparedness *G. Frederick, Manager Maintenance *W. Gabbard, Nuclear Specialist, Technical Support *M. Griffis, Manager Plant Modifications M. Hobbs, I&C Superintendent *K. Holmes, Manager Operations D. Huyck, Nuclear Security Manager *W. Jukes, Nuclear Security Supervisor W. Kitchens, Assistant General Manager Plant Support R. LeGrand, Manager Health Physics and Chemistry *G. McCarley, ISEG Supervisor R. Moye, Plant Engineering Supervisor *M. Sheibani, Nuclear Safety and Compliance Supervisor *C. Stinespring, Manager Administration *J. Swartzweider, Manager Outage and Planning *C. Tynan, Nuclear Procedures Supervisor J. Williams, Supervisor Work Planning and Controls

Other licensee employees contacted included technicians, supervisors, engineers, operators, maintenance personnel, quality control inspectors, and office personnel.

Oglethorpe Power Company Representative

T. Mozingo

NRC Inspectors

B. Bonser, Senior Resident Inspector

D. Starkey, Resident Inspector

*P. Balmain, Resident Inspector

D. Seymour, Project Engineer

*Attended Exit Interview

An alphabetical list of abbreviations is located in the last paragraph of the inspection report.

2. Plant Operations (71707)

a. General

The inspection staff reviewed plant operations throughout the reporting period to verify conformance with regulatory requirements, TSs, and administrative controls. Control logs, shift supervisors' logs, shift relief records, LCO status logs, night orders, standing orders, and clearance logs were routinely reviewed. Discussions were conducted with plant operations, maintenance, chemistry, health physics, engineering support and technical support personnel. Daily plant status meetings were routinely attended.

Activities within the control room were monitored during shifts and shift changes. Actions observed were conducted as required by the licensee's procedures. The complement of licensed personnel on each shift met or exceeded the minimum required by TS. Direct observations were conducted of control room panels, instrumentation and recorder traces important to safety. Operating parameters were verified to be within TS limits. The inspectors also reviewed DCs to determine whether the licensee was appropriately documenting problems and implementing corrective actions.

Plant tours were taken during the reporting period on a routine basis. They included, but were not limited to the turbine building, the auxiliary building, electrical equipment rooms, cable spreading rooms, NSCW towers, DG buildings, AFW buildings, and the low voltage switchyard.

During plant tours, housekeeping, security, equipment status and radiation control practices were observed.

b. Unit 1 Summary

The unit began the inspection period at 100% power. On March 13, power was decreased to 70% due to scheduled maintenance activities on the MFPT B overspeed trip test circuitry and replacement of HDP A. On March 14, power was increased to 90% after completion of work on MFPT B. On March 18, HDP A was returned to service and power was increased from 90% to 100%, where it remained through the remainder of the reporting period.

c. Unit 2 Summary

The unit operated at 100% power throughout the inspection period except for a brief unanticipated automatic load reduction which occurred on February 26, 1994 (paragraph 2.d).

Unit 2 Automatic Load Decrease

On February 26, Unit 2 experienced an automatic load reduction of approximately 350 MWE, then automatically recovered to full turbine load approximately 20 seconds later. During the event operators received alarms on steam flow/feed flow mismatch on all 4 SGs, T_{AVG}/T_{REF} deviation, and OPDT/OTDT alert. The standby condensate pump and EHC pump started and several steam dumps opened. When the unit was stabilized at 1200 MWE, operators placed the main turbine in standby load control (manual control) until the problem with automatic load control could be investigated.

Subsequent troubleshooting revealed that the load rejection was caused by an EHC electrical malfunction on the turbine backup speed sensor, due to a loose wire in the turbine EHC speed control cabinet. There are two speed sensor circuits, primary and backup, which provide turbine speed signals to the load control circuitry of the turbine EHC system. Troubleshooting efforts also identified a loose wire on the primary speed sensor circuitry. Adjustments were made to tighten the loose wires and the system was returned to automatic load control. The licensee initiated a tracking item to inspect, during a future outage, the EHC cabinets of both units for loose wires. The inspector had no concerns regarding the event or the licensee's corrective actions.

Review of Steam Generator Tube Leak/Rupture Operating Procedure Guidance.

During this inspection period the inspector reviewed the licensee's procedural guidance for steam generator tube leaks or ruptures. This review was done in response to NRC Information Notice 93-56, Weakness in Emergency Operating Procedures Found As Result Of Steam Generator Tube Rupture. The IN discussed problems in EOP requirements for monitoring secondary radiation monitor parameters, which contributed to a delay in isolating a ruptured generator during a SGTR event at Palo Verde.

The inspector reviewed the licensee's actions taken in response to the IN and industry operating experience. The inspector noted that operators were trained to respond to historical trend data which is available in the main control room to determine if abnormal radiation conditions exist, and do not rely solely on instantaneous alarms to diagnose a primary to secondary leak. The inspector reviewed radiation monitor setpoints for SG blowdown, SJAE, and main steamline radiation monitors; and noted that alarms were set slightly above background levels and will actuate to provide indication of a tube leak.

The licensee does not currently have administrative guidance for monitoring and responding to minor steam generator tube leaks

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which are less than TS limits. The licensee has established a task team to develop administrative guidelines and limits for "acceptable" tube leakage, and guidance for continuing operation if leakage is less than TS limits. The inspector reviewed RER 94-008, which was initiated to perform an engineering evaluation of radiation monitor capabilities and responses to various leakrates.

Based on this review the inspector concluded that the guidance available in EOPs and operator training for monitoring secondary radiation parameters would allow for timely diagnosis and isolation of a primary to secondary leak.

No violations or deviations were identified.

- Surveillance Observation (61726)
 - a. General

Surveillance tests were reviewed by the inspectors to verify procedural and performance adequacy. The completed tests reviewed were examined for necessary test prerequisites, instructions, acceptance criteria, technical content, data collection, independent verification where required, handling of deficiencies noted, and review of completed work. The tests witnessed, in whole or in part, were inspected to determine that approved procedures were available, equipment was calibrated, prerequisites were met, tests were conducted according to procedure, test results were acceptable and systems restoration was completed.

SURVEILLANCE NO.	TITLE
14410-1	Control Rod Operability Test
14980-1	Diesel Generator

The inspectors did not identify any problems or concerns during the observation of these surveillance activities.

b. Operator Error During Control Rod Operability Test

On February 28, during performance of Procedure 14410-1, Control Rod Operability Test, the Unit 1 Reactor Operator attempted to withdraw SDB B, which was at 231 steps, rather than insert it ten steps as directed by the procedure. Since SDB B was already fully withdrawn at 231 steps, no actual rod movement occurred, however, the demand step counter indication increased from 231 to 233 steps. When the RO recognized his mistake he attempted to insert SDB B back to 231 steps as indicated on the step counter. At that point SDB B moved from its actual position at 231 steps to 229 steps, while the demand step counter went from 233 steps to 231 steps. The RO informed the reactor engineer, who was observing the surveillance, and the USS of his error. The USS directed that Procedure 13502-1, Control Rod Drive and Position Indication System, Section 4.4.4 be entered. The procedure directed the operator how to correct the discrepancy, and both SDB B and the demand step counter were returned to 231 steps. It should be noted that at no time did this rod position indication discrepancy violate the requirements of TS 3.1.3.1, Moveable Control Assemblies.

The inspector, who observed this surveillance from the control room, concluded that, although there was no safety significance to this operator error, it represented an example of an event which could have been prevented if the operator had implemented the guidance of the licensee's self checking program. The inspector had no further concerns.

No violations or deviations were identified.

- Maintenance Observation (62703)
 - a. General

Maintenance activities were observed and/or reviewed during the reporting period to verify that work was conducted in accordance with approved procedures, TSs, and applicable industry codes and standards. Activities, procedures, and work orders were examined to verify proper authorization to begin work, provisions for fire, cleanliness, radiation exposure control, proper return of equipment to service, and that limiting conditions for operation were met.

The inspectors witnessed or reviewed the following maintenance activities:

MWO NOS.	WORK DESCRIPTION
29400659	Install Vent Port On DG 2B Overspeed Trip Device
29300426	Rework ARV 2PV-3020
19400810/ 19400811	Verify Reseating of HHSI Alternate Mini-Flow Relief Valves
19400680	Replace Pressurizer Pressure Transmitter 1PT-0457
19401391	Replace Compressor On DG 1A Air Start Dryer #2

The inspectors did not identify any problems or concerns during the observation of these maintenance activities.

b. Review of MSIV Full Stroke Testing Requirements

Due to a recent event at another facility where an MSIV failed to stroke fully closed when required, the inspectors reviewed the full stroke testing requirements and the implementation of the requirements at Vogtle. The Vogtle MSIVs are manufactured by Rockwell International and are 30-inch hydraulically opened gate valves which are closed by a stored energy system in the form of a volume of nitrogen pressurized to approximately 2500 psig.

The Vogtle FSAR, Section 3.9.6.2, Inservice Testing of Valves. states that the test program will conform to the requirements of ASME Code, Section XI, Subsection IWV, to the extent practical, and comply with all applicable portions of 10 CFR 50.55a (g). The surveillance requirements of TS 4.7.1.5, Main Steam Line Isolation Valves, requires that each MSIV shall be demonstrated operable by verifying full closure within 5 seconds when tested pursuant to TS 4.0.5, Surveillance Requirements. TS 4.0.5 states that this testing shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. ASME Section XI. Subsection IWV-3000, Inservice Testing of Valves in Nuclear Power Plants, states that valves shall be exercised to the position required to fulfill their function unless such operation is not practical during plant operation. If only limited operation is practical during plant operation, the valve shall be part-stroke exercised during plant operation and full-stroke exercised during cold shutdown. The inspector also reviewed the Rockwell Maintenance Manual for MSIVs and determined that the vendor does not require that the MSIVs be full-stroke tested during plant operations at normal operating temperature.

The inspector reviewed the licensee's implementation of the above requirements by discussing the IST program with plant engineers and reviewing procedures related to MSIV stroke testing. The MSIVs are full-stroke tested prior to Mode 3 entry per Procedure 14850-1/2, Cold Shutdown Valve Inspection Test. The frequency of this surveillance is once every cold shutdown but not more often than every three months. Once every three months a 10% stroke test is performed on all MSIVs per Procedure 14842-1/2, Main Steam Isolation Valves Partial Stroke Inservice Test. This surveillance is applicable in Modes 1,2 and 3.

The inspector concluded that the licensee has implemented a program to meet the TS requirements and vendor recommendations for inservice testing of the MSIVs. No concerns were identified during this review.

c. Verification of HHSI Alternate Mini-flow Relief Valve Reseating

On February 2, 1994, Unit 1 experienced a reactor trip/safety injection as discussed in NRC IR 50-424,425/94-02. During the SI.

the HHSI alternate mini-flow relief valves, 1PSV-8510 A/B, lifted and diverted a portion of SI ECCS injection flow to the RWST. These relief valves performed as designed since their purpose is to prevent deadheading of the HHSI pumps when the primary system pressure is higher than the shutoff head of the pumps. The highest RCS pressure noted during this event was 2315 psig, while the shutoff head of the HHSI pumps is approximately 2685 psig. Since RCS pressure was relatively high, only 150 gpm were injected into the RCS and some lesser flow was diverted to the RWST. IR 50-424,425/92-18 discusses the function of the HHSI alternate mini-flow relief valves and the NRC's concerns regarding their operation. These relief valves were removed from Unit 2 during the last refueling outage and will be removed from Unit 1 during the next refueling outage in September 1994.

On February 14, 1994, the NRC issued a letter to the licensee which provided guidance for the minimum test requirements considered necessary to ensure the operational readiness of the HHSI alternate mini-flow relief valves. One of the guidelines was that an assessment of valve operability should be made following any system actuation requiring valve discharge, and that this assessment should include a visual inspection and verification of valve reseating. On February 2, following the SI, operations personnel performed a visual inspection of the area where the valves are located and did not observe any deficiencies. On February 4, a walkdown was done of the area by Engineering Support personnel and again no deficiencies were observed. On March 3, 1994, after evaluating several potential methods of verifying valve reseating, the licensee chose to attempt to gag the valve in the fully closed position. During this test no valve stem movement was observed which indicated that the valves were already fully closed. The inspector was present during the reseat verification and observed that both valves were fully closed.

The inspector concluded that the licensee followed the intent of the NRC HHSI alternate mini-low relief valve inspection guidance and that the system was not adversely affected by the SI on February 2.

d. Improper Assignment of Post Maintenance Functional Tests

On February 14 the licensee identified, during an IST review of completed MWOs, that the incorrect post maintenance tests were assigned and performed on three separate occasions following maintenance on Unit 1 CCW pumps. The inspector reviewed the licensee's IST program requirements, functional test assignment requirements, and MWO documentation to determine the significance of this issue.

Maintenance performed on the three CCW pumps included a PM task on the pump-to-motor couplings. The task involved disassembly, regreasing and reassembly of the couplings. TS 4.0.5, Surveillance Requirements, requires performance of Inservice Testing of safety related pumps in accordance with ASME Section XI. Section XI, Part IWP-3111, requires an inservice test run when a reference value or values may have been affected by repair or routine servicing of pump. The licensee performed an engineering evaluation of the maintenance performed on the CCW pump couplings with the pump vendor, and concluded that the maintenance would not have affected pump performance. The inspector verified, following discussion with an NRC Mechanical Systems Reviewer, that the licensee's evaluation was acceptable and the performance of an IST surveillance was not required following the maintenance performed on the CCW pumps. The inspector also reviewed subsequent IST data for the CCW pumps and verified that the reference valves had not degraded.

IST program requirements and post maintenance functional test assignment requirements are specified in the licensee's procedures. The inspector reviewed Procedures 00412-C, Inservice Testing Program, and 29401-C, Maintenance Work Order Functional Tests. Procedure 00412-C, Step 4.6.4.1.1, requires IST surveillance retesting following the disassembly and reassembly of rotating parts (which includes couplings). Procedure 29401-C, Step 4.1, requires the work planning group involved in the review of MWO packages for functional test requirements to use the MWO Functional Test Guidelines when establishing the required post maintenance testing. Attachment A, MWO Functional Test Guidelines - Pumps, requires the performance of an IST surveillance test following replacement of a coupling device.

For the three MWOs involving the CCW pump maintenance (19300939, 19301063, and 19301204) separate work planners did not recognize that an IST surveillance was required to be assigned per procedure as the functional test. The licensee has initiated corrective actions to train the operations planners and operations personnel on functional test assignment and performance. The licensee is evaluating procedure revisions to the MWO Functional Test Procedure to better define IST testing requirements.

Based on this review the inspector concluded that the failure to properly assign post maintenance functional tests for CCW pump maintenance activities is a violation of Procedures 00412-C and 29401-C. This violation will not be subject to enforcement action because the licensee's efforts in identifying and correcting the violation meet the criteria specified in Section VII.B of the Enforcement Policy. This item is identified as NCV 50-424/94-05-01, Improper Assignment Of Functional Tests. The inspector also considered this as a weakness in the assignment of functional test requirements since an IST surveillance was not assigned on three MWOs by three separate work planners.

One non-cited violation was identified.

5. Review of Corporate Engineering and Design Change Support (40703, 37838)

During this inspection period, the inspectors visited the Southern Nuclear Company/Georgia Power Company Corporate offices in Birmingham, Alabama. The purpose of this visit was to review and evaluate the effectiveness and capabilities of the offsite support organization.

The inspectors held discussions with the SNC/GPC Nuclear Engineering, Licensing Manager and Nuclear Maintenance and Support Manager on several on-going design changes, including the Wilson Feed DCP and the 1E Battery replacement DCP. The inspectors reviewed details of the Wilson Feed DCP and the maintenance rule implementation with Vogtle Project support engineers. The inspectors also reviewed the status of several licensing activities with the Licensing Manager.

The inspectors also met with SCS Engineering personnel, including the Vogtle Project Engineering Manager, Project Design Manager and several supervisors and staff members. The inspector reviewed FCR trending, and ABN incorporation trend data and concluded that the DCP process worked effectively. The inspector also noted that SCS met its goal to deliver 90% of planned DCPs to the site six months prior to the beginning of the next Unit 1 refueling outage in September 1994.

The inspector observed a demonstration of the Reference File Manager, a computer based CAD system, which is being implemented to convert Vogtle drawings to electronic media.

The inspector reviewed recent SAER audits of the Nuclear Engineering and Licensing Support Group, and of the Nuclear Maintenance and Support Group, and a SCS QA audit of SCS - Vogtle Project activities. The audit conclusions stated that these organizations provided adequate support to the plant.

Based on this review the inspector concluded that the SNC/GPC and SCS organization functioned effectively to support the site.

No violations or deviations were identified.

 Evaluation of Licensee Self Assessment Capability - Review of SRB Review Activities (405.3)

On March 10 the inspector attended an SRB meeting and reviewed SRB activities in Birmingham Alabama. The SRB is the offsite review committee required by TS 6.4.2, Safety Review Board. The inspector reviewed SRB meeting minutes for the last year and verified that TS requirements for committee composition, quorum and areas of review were met. The inspector observed the March 10 meeting and noted that the SRB review was of substantial depth and that several action items were generated during discussions which required followup at the plant staff and SAER audit staff levels. Review areas required by TS were discussed at the meeting. The inspector reviewed the qualification and expertise of the committee members and alternates and verified that the membership collectively possessed the expertise required to review designated activities in the areas required by TS 6.4.2.1. The inspector also reviewed SRB open item tracking and observed that open items were closed prior to the next scheduled SRB meeting unless an extension was authorized by the SRB chairman.

Based on this review the inspector concluded that the SRB functions effectively and performs the independent reviews required by TS.

No violations or deviations were identified.

7. Follow-up (90712) (92700) (92702)

The Licensee Event Reports and violation listed below were reviewed to determine if the information provided met NRC requirements. The determination included: adequacy of description, verification of TS compliance and regulatory requirements, corrective action taken, existence of potential generic problems, reporting requirements satisfied, and relative safety significance of each event.

a. (Closed) LER 50-424/93-008, Reactor Trip Due to Loss of Turbine Electrohydraulic Control Fluid Pressure.

The event was caused by a failure of the A EHC pump, followed by the failure of the B EHC pump to auto start at the correct pressure. The initiating failure of the A EHC pump was most likely caused by the inadvertent addition of water into the EHC fluid reservoir. The standby pump auto start failure was due to the pressure switch calibration drifting low.

The inspector reviewed MWO documentation and verified that the licensee completed corrective actions to replace the A EHC pump (MWO 19302005) recalibrate the standby pump autostart pressure switches, and increase the low EHC pressure alarm, setpoint (MWO 19300599). The inspector also verified that the Unit 2 EHC low pressure alarm setpoint was increased (MWO 29301994).

The inspector reviewed a memorandum from the maintenance manager to all maintenance personnel which reinforced maintenance personnel responsibilities regarding the use and disposal of storage containers. The inspector also reviewed the EHC Policy which was added to the Operations Policy Book following the event. This policy clearly outline guidance for the use and storage of EHC fluid and drums.

Based on a review of these corrective actions, this item is closed.

b.

(Closed) IFI 50-424,425/93-23-01, Review Licensee Safety Evaluations

The inspector discussed, with SCS corporate design engineering personnel, the circumstances which led to the improper deletion of leakage testing requirements for several Unit 1 and 2 ACCW valves. The FSAR revision that approved the deletion of the testing requirements was based on a safety evaluation which did not receive a design engineering review since the revision was not developed as a result of a DCP.

The inspector reviewed DCs which the licensee identified since the inadequate safety evaluation for the ACCW system was discovered. The inspector did not identify additional instances where safety evaluations generated for licensing document changes which were inadequate. The inspector will review the licensee's corrective actions during followup of the LER which was issued as a result of this error.

The inspector concluded that the inadequate safety evaluation generated to support the FSAR revision was isolated. Based on this review this item is closed.

No violations or deviations were identified.

8. Exit Meeting

The inspection scope and findings were summarized on March 21, 1994, with those persons indicated in paragraph 1. The inspector described the areas inspected and discussed in detail the inspection findings. No dissenting comments were received from the licensee. The licensee did not identify as proprietary any of the material provided to or reviewed by the inspectors during the inspection.

Item No.

Description and Reference

NCV 50-424/94-05-01

Improper Assignment Of Functional Tests

No violations or deviations were identified.

8. Abbreviations

ABN	- As Built Notice
ACCW	- Auxiliary Component Cooling Water System
AFW	- Auxiliary Feedwater System
ARV	- Atmospheric Relief Valves
ASME	- American Society of Mechanical Engineers
BTI	- Bypass Test Instrumentation
CAD	- Computer Assisted Design
CAS	- Central Alarm Station
CCP	- Centrifugal Charging Pump

CFR- Code of Federal ŘegulationsCR- Control RoomCS- Containment SprayCST- Condensate Storage TankDC- Deficiency CardDCP- Design Change PackageDG- Diesel GeneratorECCS- Emergency Core Cooling SystemEHC- Electo-Hydraulic Control SystemEOP- Emergency Operating ProcedureESF- Engineered Safety FeaturesESFAS- Engineered Safety Features Actuation SystemFCR- Field Change RequestFHB- Fuel Handling BuildingFSAR- Final Safety Analysis ReportGPC- Georgia Power Companygpm- Gallons Per MinuteHOP- Heater Drain PumpHHSI- High Head Safety InjectionHP- Health PhysicsHVAC- Heating, Ventilating and Air ConditioningIAC- Instrumentation and ControlsIFI- Inspector Following ItemIN- Information NoticeIR- Inservice TestLCO- Limiting Condition for OperationLDCR- Loss of Colant AccidentLOSP- Loss of Colant AccidentLOSP- Loss of Offsite PowerMFPT- Main Steam Isolation ValveMV- Motor Operated ValveMSIV- Main Steam Isolation ValveMWO- Maintenance Work OrderNCV- Non-Cited ViolationNPF- Nuclear Power FacilityNRC- Nuclear Steam Supply System	CCW	- Component Cooling Water System
CR - Control Room CS - Containment Spray CST - Condensate Storage Tank DC - Deficiency Card DCP - Design Change Package DG - Diesel Generator ECCS - Emergency Core Cooling System EHC - Electo-Hydraulic Control System EBF - Engineered Safety Feature ESF - Engineered Safety Features FCR - Field Change Request FHB - Fuel Handling Building FSAR - Final Safety Analysis Report GPC - Georgia Power Company gpm - Gallons Per Minute HOP - Heater Drain Pump HHSI - High Head Safety Injection HP - Heating, Ventilating and Air Conditioning I&C - Instrumentation and Controls IFI - Inspector Following Item IN - Information Notice IR - Inspection Report ISEG - Independent Safety Engineering Group IST - Inservice Test LCO - Limiting Condition for Operation LDCR - Licensing Document Change Request LER - License Event Report LOCA - Loss of Colant Accident LOSP - Loss of Offsite Power MFPT - Main Feed Pump Turbine MVV - Motor Operated Valve MSIV - Main Steam Isolation Valve MWE - Megawatts Electric MWO - Main Steam Isolation Valve MWE - Megawatts Electric MWO - Main Condition Maine NFFT - Nuclear Power Facility NRC - Nuclear Regulatory Commission NSCW - Nuclear Steam Supply System	CFR	- Code of Federal Regulations
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DG - Diesel Generator ECCS - Emergency Core Cooling System EHC - Electo-Hydraulic Control System EOP - Emergency Operating Procedure ESF - Engineered Safety Feature ESFAS - Engineered Safety Features Actuation System FCR - Field Change Request FHB - Fuel Handling Building FSAR - Final Safety Analysis Report GPC - Georgia Power Company gpm - Gallons Per Minute HOP - Heater Drain Pump HHSI - High Head Safety Injection HP - Health Physics HVAC - Heating, Ventilating and Air Conditioning I&C - Instrumentation and Controls IFI - Inspector Following Item IN - Information Notice IR - Inspection Report ISEG - Independent Safety Engineering Group IST - Inservice Test LCO - Limiting Condition for Operation LDCR - Licensing Document Change Request LER - Licensee Event Report LOCA - Loss of Colant Accident LOCA - Loss of Offsite Power MFPT - Main Feed Pump Turbine MOV - Motor Operated Valve MSIV - Main Steam Isolation Valve MWE - Megawatts Electric MWO - Maintenance Work Order NCV - Non-Cited Violation NPF - Nuclear Regulatory Commission NSCW - Nuclear Steam Supply System	DCP	- Design Change Package
ECCSEmergency Core Cooling SystemEHCElecto-Hydraulic Control SystemEOPEmergency Operating ProcedureESFEngineered Safety FeaturesESFASEngineered Safety Features Actuation SystemFCRField Change RequestFHBFuel Handling BuildingFSARFinal Safety Analysis ReportGPCGeorgia Power CompanygpmGallons Per MinuteHDPHeater Drain PumpHHSIHigh Head Safety InjectionHPHealth PhysicsHVACHeating, Ventilating and Air ConditioningI&CInspector Following ItemINInformation NoticeIRInspection ReportISEGIndependent Safety Engineering GroupISTInservice TestLCOLicensing Document Change RequestLERLicensing Document Change RequestLERLicensing Document Change RequestLCALoss of Coolant AccidentLOSPLoss of Offsite PowerMFPTMain Feed Pump TurbineMOVMotor Operated ValveMWEMegawatts ElectricMWOMaintenance Work OrderNCVNon-Cited ViolationNPFNuclear Regulatory CommissionNSCWNuclear Steam Supply System	DG	- Diesel Generator
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NSSS - Nuclear Steam Supply System	NSCW	- Nuclear Service Cooling Water System
The feat second oupping of second	NSSS	- Nuclear Steam Supply System
OPDT - Over Power Delta Temperature	OPDT	- Over Power Delta Temperature
OTDT - Over Temperature Delta Temperature	OTDT	- Over Temperature Delta Temperature
PA - Protected Area	PA	- Protected Area
PEO - Plant Equipment Operator	PEO	- Plant Equipment Operator
PM - Preventive Maintenance	PM	- Preventive Maintenance
psig - Pounds Per Square Inch Gauge	psig	Pounds Per Square Inch Gauge
0A - Quality Assurance	QA	- Quality Assurance
RCS - Reactor Coolant System	RCS	- Reactor Coolant System
RER - Request For Engineering Review	RER	- Request For Engineering Review

RHR	- Residual Heat Removal System
RO	- Reactor Operator
RPM	- Revolution Per Minute
RWST	- Refueling Water Storage Tank
SAER	- Safety Audit And Engineering Review
SCS	- Southern Company Services
SDB	- Shut Down Bank
SG	- Steam Generator
SGTR	- Steam Congrator Tube Pupture
ST	- Safaty Injection
S.JAF	Steam lat Air Fiector
SNC	Southorn Nuclear Company
CDR	Safety Poulou Peand
CDO	- Salety Review Board
SRU	- Senior Reactor Operator
SSPS	- Solid State Protection System
TAVG	- Average Temperature
TREF	- Reference Temperature
TDAFW	- Turbine Driven Auxiliary Feedwater
TS	- Technical Specifications
UOP	- Unit Operating Procedure
URI	- Upresolved Item
USS	- Unit Shift Supervisor
VIO	- Violation
184	- Unit 1 Fourth Refueling Outage
203	Unit 2 Thind Defueling Outage
6.17.50	- Unit c initia Neruering Outage