

Department of Energy Washington, D.C. 20545 Docket No. 50-537 HQ:S:82:160

DEC 2 1 1982

Mr. Paul S. Check, Director CRBR Program Office Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Dear Mr. Check:

STRUCTURAL MARGIN BEYOND DESIGN BASE LOADS ON THE REACTOR VESSEL SUPPORT LEDGE

Enclosed is the subject analysis that was requested at the December 9, 1982, meeting between the Applicant and the Nuclear Regulatory Commission staff.

If you have any questions, please call W. Pasko (FTS 626-6096) of the Project Office Public Safety Division.

Sincerely, John R. Longeneder

John R. Longenecker Acting Director, Office of Breeder Demonstration Projects Office of Nuclear Energy

Enclosure

10

cc: Service List Standard Distribution Licensing Distribution

Dool

SMBDB ANALYSIS

. . .

14

.

.

.

/

REACTOR VESSEL SUPPORT LEDGE

1. Requirements, Loads and Allowable Stresses

The SMBDB is a third level margin design loading. The SMBDB conditions are defined by a dynamic load on the ledge from the Reactor System. The layout of the reactor vessel ledge support is shown in Fig. 1. The dynamic load was provided as time-histories of vertical and toroidal loads. Figures 2 and 3 show the time-histories of the loads. In addition to the dynamic load, dead load, live load and thermal loads due to normal operating conditions were included. The aplicable load combination is:

1.6 S = D + L + To + A

Where:

- D = Dead Load
- L = Live Load
- To = Thermal Loads Due to Normal Operating Conditions
- A = SMBDB Load
- S = Required section strength based on the elastic design methods and the allowable stresses defined in Part 1 of the AISC Manual of Steel Construction

For plate elements the allowable stresses are the same as for load combinations 6, 8 or 10 of the Design Basis Conditions (Table I). For buckling the allowable stresses are those given in Table II for load combinations 5 to 10. Load combinations are given in Table III.

2. Method of Analysis

The analysis was performed using the computer program STARDYNE. It consisted of two parts. The first part was simplified dynamic analysis to obtain an equivalent dynamic load factor. The second part consisted of an equivalent static analysis on a detailed finite element model, with a load equal to the peak of the time-history multiplied by the equivalent dynamic load factor.

The first part of the analysis used four different models.

The first model consisted of a two degree of freedom system representing respectively the ledge and the

- 2 -

Reactor Cavity below. Parametric studies conducted with this model showed that the ledge responses were relatively insensitive to the cavity properties. An equivalent dynamic load factor of 1.6 was calculated from this model.

The other three models for dynamic analysis consisted of finite element representations of 100 sectors of the ledge. 100 is the typical angle between brackets. The models represented respectively, sections with a full bracket, with a cut-out bracket, and with a cut-out bracket with a box column. The cavity was represented by an equivalent mass and spring. It was found that if an equivalent dynamic factor of 2 was used, the highest stresses in all the models were enveloped. Therefore, a dynamic load factor of 2 was adopted.

The detailed stress analysis was conducted with the same 90° model used for the design basis conditions. The peaks of the SMBDB force time-histories were multiplied by a factor of two and applied as equivalent static forces.

In the static analysis four cases were considered. They represented the four possible combinations of signs of the two loads. The results of the four cases were enveloped.

3. Results

Tables IV and V summarize the calculated stresses and compare them with the allowable values.

In all cases the calculated stresses did not exceed the stress limits.

For the bolt sleeves, which are not listed in the Tables, buckling was verified using the interaction equations of Section 1.6.1 of the AISC Manual of Steel Construction and the requirements were met.

The maximum compressive stress in the box column is 30.5 ksi which is less than the limit, 34.6 ksi. In the box column verification the compressive stresses due to bending were added to the axial stresses and compared with the axial allowable. This is a conservative approach.



s. 18 ...

.

FIGURE 1

- 4 -

TYPICAL SECTION THROUGH REACTOR VESSEL LEDGE SUPPORT



..

SMBDB

VERTICAL FORCE HISTORY FOR REACTOR VESSEL SUPPORT LEDGE Figure 2



SMBDB TOROIDAL MOMENT HISTORY FOR REACTOR VESSEL SUPPORT LEDGE Figure 3

- 6 -

TABLE I

STRESS INTENSITY LIMITS FOR PLATE ELEMENTS

Units: ksi

Load		Primary	Stresses	Primary Plus Secondary Stresses		
Combinations		Membrane	Membrane Plus Bending			
1 to 4	Normal Upset	26.7 (23.3)	40.0 (35.0)	80.0 (70.0)		
5 to 10	Faulted	56.0 (49.0)	56.0 (49.0)	80.0 (70.0)		

() For plates of thickness greater than 4 inches

- 7 -

	-
	τ.
LE I	

	LOCATION OF PLATES	LC 1 & 2	LC 3 & 4	LC 5 - 10
1.	Horizontal Plates			
	At E1. 800.33	30.0	45.0	48.0
	At E1. 795.21	36.0	54.0	57.6
	At El. 793.67	36.0	54.0	57.6
	At El. 791.67	34.0	51.0	54.4
	At E1. 780.83	30.0	45.0	48.0
2.	Radial Plates			한 이 가는 것이
	3' Plate Embedded in Concrete	34.7	52.0	55.5
	Above E1. 795.21	34.7	52.0	55.5
	Below E1. 795.21*	30.8	46.2	49.3
3.	Vertical Cylindrical Pl	Lates		
	at $r = 12.125$	1		
	- 800.33 to /93.67	36.0	54.0	57.6
	- 791 67 +0 790 48	34.0	51.0	54.4
	at r = 14 04'	34.0	51.0	54.4
	at $r = 20'$	30.0	54.0	57.8
	- 800.33 to 795.21	34.7	52.0	55.5
	- 795.21 to 780.83	30.9	46.3	49.4
				12.1
4.	Web Stiffner	30.8	46.2	49.3
5.	Linear Elements		1	20.24
	Box Columns	25.8	38.7	41.3
	Bolt Sleeves	33.6	50.4	53.8

ALLOWABLE BUCKLING STRESSES (in ksi)

NOTE: Above stresses are for a metal temperature = 100°. Interpolate linearly from 1.0 to 0.8 for 100° to 350° for multiplication factor for above stresses

For radial bracket elements between R = 12' and R = 14' and Elevators 793.67 ft. and 795.25 ft. use the same allowable stresses as for above Elevation 795.21 ft.

TABLE .III

LOAD COMBINATIONS

- A. For Service Load Conditions
 - 1. S = D + L2a. S = D + L + E2b. S = D + L - E3a. 1.5S = D + L + To + E3b. 1.5S = D + L + To - E4a. 1.5S = D + L + Tu + E4b. 1.5S = D + L + Tu - E

B. For Factored Load Conditions

5a. 1.6S = D + I + To + E'5b. 1.6S = D + L + To - E'6a. $1.6S^* = D + L + Ta + Pa + E$ 6b. $1.6S^* = D + L + Ta + Pa - E$ 7a. $1.7S^* = D + L + Ta + Pa - E'$ 7b. $1.7S^* = D + L + Ta + Pa - E'$ 1.6S* = D + L + Ta + Pa + E 8b. $1.6S^* = D + L + Ta + Pa - E$ 9a. $1.7S^* = D + L + Ta + Pa - E'$ 9b. $1.7S^* = D + L + Ta + Pa - E'$ 10a. 1.6S = D + L + Tu + E'10b. 1.6S = D + L + Tu - E'

Notes for Load Combinations:

- * Plastic section modulus "Z" may be used in lieu of elastic section modulus.
- "S" is the required section strength based on the elastic design methods and the allowable stresses defined in Part 1 of the AISC Specification.

D	52	Dead Load	
L	=	Live Load	
E	=	OBE Loads	
E'	=	SSE Loads	
To	=	Thermal Loads Due to Normal Conditions	
Tu	=	Thermal Loads Due Upset Conditions	
Ta	=	Thermal Loads Due to Accident Conditions (DBA)	
Pa	=	Pressure Loads Due to DBA	

 Load combinations 6 and 7 considered DBA conditions at 12 hours. Load combinations 8 and 9 considered DBA conditions at 80 hours.

TABLE IV

SMBDB

Summary of Maximum Stress Intensities in Plates

	Рп	Pm		Pm + Pb		Pm + Pb + S	
LOCATION	Calc.	Allow.	Calc.	Allow.	Calc.	Allow.	
Bracket	55.8	56.0	55.9	56.0	55.9	80.0	
Bracket	54.3	56.0	54.6	56.0	54.6	80.0	
Stiffener	47.5	56.0	47.6	56.0	52.9	80.0	
Stiffener	46.3	56.0	46.5	56.0	52.8	80.0	
Stiffener	45.2	56.0	45.5	56.0	51.8	80.0	
P1. E1. 800.3'	20.0	49.0	37.9	49.0	46.0	70.0	

24

Units: ksi

. 10 .

> Primary membrane stress intensity Primary bending stress intensity Secondary stress intensity Pm =

Pb S =

==

TABLE V

SUMMARY OF MAXIMUM PRINCIPAL MEMBRANE COMPRESSIVE STRESSES IN PLATES

SMBDB

LOCATION	^f calc ksi	fall (+) ksi
Radial Stiffener	49.1* (41.3)	51.6
Bracket (Between R = 12' and R' = 14')	43.3* (29.5)	44.4
R = 12'	37.9*	48.4
Plate El. 800.3'	26.2*	40.3

* Peak Stress

.

(

14

() Average Stress

(+) Based on Steel temperature

- 11 -

:

FRONT-LINE AND SUPPORT SYSTEM RELIABILITY PROGRAMS

1. 1. 1. 1.

- Quantitative measures compatible with the overall measures
- Mechanisms for reliability tradeoff at system and component level
- Appropriate reliability information transfer
- Definition of performance requirements
- Assurance of continuing program continuity
- An auditable reliability program