

OPERATING DATA REPORT

DOCKET NO. 50-266

DATE December 6, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1 NOTES .
 2. REPORTING PERIOD: NOVEMBER 1982
 3. LICENSED THERMAL POWER (MWT): 1518.
 4. NAMEPLATE RATING (GROSS MWE): 523.8
 5. DESIGN ELECTRICAL RATING (NET MWE): 497.
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): 390.0
 10. REASONS FOR RESTRICTIONS, (IF ANY): Power level restricted because of self-imposed hot leg
temperature limitation in an attempt to limit steam generator tube corrosion.
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|------------|------------|-------------|
| 11. HOURS IN REPORTING PERIOD | 720 | 8,016 | 105,792 |
| 12. NUMBER OF HOURS REACTOR WAS CRITICAL | 0.0 | 6,642.4 | 86,995.6 |
| 13. REACTOR RESERVE SHUTDOWN HOURS | 0.0 | 11.4 | 618.7 |
| 14. HOURS GENERATOR ON LINE | 0.0 | 6,602.1 | 84,574.3 |
| 15. UNIT RESERVE SHUTDOWN HOURS | 0.0 | 27.4 | 791.7 |
| 16. GROSS THERMAL ENERGY GENERATED (MWH) | 0 | 7,996,075 | 115,348,915 |
| 17. GROSS ELECTRICAL ENERGY GENERATED (MWH) | 0 | 2,656,150 | 38,677,430 |
| 18. NET ELECTRICAL ENERGY GENERATED (MWH) | 0 | 2,514,460 | 36,795,671 |
| 19. UNIT SERVICE FACTOR | 0.0 | 82.4 | 79.9 |
| 20. UNIT AVAILABILITY FACTOR | 0.0 | 82.7 | 80.7 |
| 21. UNIT CAPACITY FACTOR (USING MDC NET) | 0.0 | 63.4 | 71.2 |
| 22. UNIT CAPACITY FACTOR (USING DER NET) | 0.0 | 63.1 | 70.0 |
| 23. UNIT FORCED OUTAGE RATE | 0.0 | 0.2 | 2.8 |
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
NONE
 25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 12/10/82

DOCKET NO. 50-266

UNIT NAME Point Beach Unit 1

DATE December 6, 1982

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH November, 1982

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-2</u>	11	<u>-2</u>	21	<u>-2</u>
2	<u>-2</u>	12	<u>-2</u>	22	<u>-2</u>
3	<u>-2</u>	13	<u>-2</u>	23	<u>-2</u>
4	<u>-2</u>	14	<u>-2</u>	24	<u>-2</u>
5	<u>-2</u>	15	<u>-2</u>	25	<u>-2</u>
6	<u>-2</u>	16	<u>-2</u>	26	<u>-2</u>
7	<u>-2</u>	17	<u>-2</u>	27	<u>-2</u>
8	<u>-2</u>	18	<u>-2</u>	28	<u>-2</u>
9	<u>-2</u>	19	<u>-1</u>	29	<u>-2</u>
10	<u>-2</u>	20	<u>-2</u>	30	<u>-2</u>
				31	<u></u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1982DOCKET NO. 50-266UNIT NAME Point Beach Unit 1DATE December 6, 1982COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code	Cause and Corrective Action To Prevent Recurrence
5	821022	S	720.0	C	1	N/A	ZZ	ZZZZZZ	Continuing seven week refueling outage.

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266
Unit Name Point Beach Unit 1
Date December 6, 1982
Completed By C. W. Fay
Telephone 414/277-2811

Unit 1 was midway into the sixth week of its scheduled seven-week refueling outage at the end of the reporting period.

The fuel movements were completed at 2113 hours on November 6, 1982 with no major problems or delays.

On Friday, November 5, 1982, while completing an annular search, a number of items were discovered on the secondary side of the steam generators. In the "A" steam generator investigations with fiberoptics revealed a 6" "C" clamp, a swivel pad which separated from the clamp, along with a 3" stainless steel hose clamp and a number of pieces of lockwire along with the residual sludge and scale. Among the items identified in "B" steam generator were pieces of weld rod, lockwire, a 1/4" x 3/8" x 58" piece of metal bar stock, and 2 blocks of steel approximately 1-1/4" x 2-1/4" x 6-1/2".

All of the objects were removed from the "B" steam generator on November 16, 1982 except for several pieces of light-weight lockwire which extended down tube columns and appeared to be cemented in the hard sludge. The 1/4" x 3/8" x 58" rod was carbon steel and showed no signs of wearing against the tubes. The rod is estimated to have been in the steam generator for at least five years and possibly could have been in the steam generator since before initial startup in 1970. The two metal blocks (carbon steel) were identified as items used in the wrapper support structure. The blocks did not show any signs of having been installed. A remote visual inspection verified that there were no installed support blocks missing. Thus, the blocks were extraneous and have been in the steam generator annulus since fabrication.

All of the items were removed from the "A" steam generator on November 24, 1982. The "C" clamp had to be cut before it could be maneuvered out of the steam generator. It is believed that the "C" clamp fell into the steam generator in October 1977 when the downcomer flow resistor plate removal was performed. The origin of the hose clamp is unknown. During the retrieval efforts, a pin 1/4" in diameter by 1-1/2" long was lost from one of the retrieval tools. Attempts to locate the pin were unsuccessful. Because of the small size of the pin, calculations of impact velocity of the pin against a steam generator tube demonstrate that the potential for tube damage is insignificant.

After retrieval of the foreign objects was complete, a visual inspection of the tubes which could have been damaged was performed using fiberoptics. The fiberscope, which had a magnification capability of 14X, showed only slight scratches and scraping of a limited number of tubes.

To aid in the analysis of possible tube damage, the Westinghouse eddy current testing crew and equipment returned for inspection of the peripheral tubes on the cold leg side of the steam generators. The hot leg side peripheral tubes had been previously inspected.

The expanded eddy current inspection results of the cold leg peripheral tubes revealed five tubes in the "B" steam generator with defect indications and three tubes in the "A" steam generator with defect indications. These tubes were plugged. In addition, ten tubes in the "A" steam generator, with defects less than the plugging limit, were plugged as a conservative measure.

Upon review of field eddy current tapes it was determined that one tube which had been sleeved in the "A" steam generator in November 1981 had an anomolous eddy current indication 11" above the tube end. On November 19, 1982, the sleeve was removed from the tube in which it was inserted and the tube was plugged. The sleeve was sent to Westinghouse for evaluation. Preliminary results of the removed sleeve show that the sleeve was not defective.

The NRC Resident Inspector has been notified of these findings. This event is reportable in accordance with Technical Specification 15.6.9.A.3 and Licensee Event Report No. 82-022/01T-0 was filed with the NRC covering the foreign object retrieval efforts, eddy current findings, and the mechanical plugging.

While performing refueling leakage tests of containment isolation valves on November 2, 1982, the "B" reactor coolant pump component cooling water supply containment isolation valve (1-755B) was found to have leakage such that the limit in Technical Specification 15.4.4.III.B was exceeded. The as-measured leakage was 128,000 sccm, which when extrapolated to the required test pressure of P_a (60 psig), would be 317,000 sccm. This event is reportable in accordance with Technical Specification 15.6.9.2.A.3, "Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment." The Resident Inspector was notified of this event. A follow-up report will be issued within 14 days of the completion of all Type "B" and "C" testing containing as-found and as-left leakage information.

On November 3, 1982, it was discovered that the existing Technical Specification fire detection system was disconnected by contract personnel to facilitate the installation of new C-900 mimic bus alarm panel of the new fire detection system. Upon discovery, proper fire watches were established and an increased effort was made to upgrade the new system to an operational condition. Upon completion of tests, parts of the new fire detection system were declared operational and the fire watch was suspended in the applicable areas. This event is considered reportable and a 30-day Licensee Event Report No. 82-021/03L-0 was filed.

On November 6, 1982, during routine I&C calibration, it was discovered that steam flow channel, 1FT-465, was out of specification. This channel provides input to the high steam flow reactor trip for Unit 1. While this channel was only .17% nonconservative, this event will be the subject of a 30-day Licensee Event Report.

On November 22, 1982, one undervoltage relay on each of the 480 V safeguards system load centers 1B03 and 1B04 was found to be out of specification per existing Technical Specification limits of Table 15.3.5-1. A 30-day Licensee Event Report will be submitted to the NRC. These relays have setpoint tolerances which by design make the Technical Specification limits difficult to achieve. Technical Specification Change Request No. 77 which increases the tolerance allowances has been submitted to the NRC to correct this situation.

On November 26, 1982 at 1230 hours, while securing the gas turbine after testing, it was noted that when the fire alarm detector was activated, there was no alarm received in the control room. Investigation by contractor personnel revealed a fuse for the alarm master output breaker was blown. The fuse was replaced and the system tested satisfactorily. This event is reportable in accordance with Technical Specifications and a 30-day Licensee Event Report will be submitted to the NRC.

On November 10, 1982 it was discovered that valve 1LCV-112A, which diverts reactor coolant from the chemical and volume control system to the waste holdup tanks, was pressurized causing the bonnets of valves 1FCV-110B and 1-256B to leak a very small amount of reactor coolant into the auxiliary building. Health physics personnel took airborne samples in the area which revealed no increase in airborne radioactivity. This event was not considered reportable.

Maintenance was completed on valves 1MOV-856A and 1MOV-856B because of a difficulty in stroking the "B" valve. The valve discs were discovered to be warped and were replaced. These valves control flow from the refueling water storage tank to the residual heat removal pumps. Investigations are underway to determine the cause of the warping of the discs. This event was not considered reportable.

Other work completed in conjunction with the outage included the overhaul of "A" reactor coolant pump motor, safety valve inspections, moisture separator reheater inspections, turbine generator overhaul, "B" reactor coolant pump motor and seal inspection, the changeout of air-operated containment isolation valves on the steam generator blowdown lines, TMI backfit modifications, environmental/seismic transmitter changeouts, BFD relay changeouts, the installation of steam generator wide range level transmitters, modifications to the steam generator wide range level transmitters, modifications to the steam line pressure sensing line heat tracing, and the addition of crossunder drain piping on the turbine.

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE December 6, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
2. REPORTING PERIOD: NOVEMBER 1982
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720	8,016	90,577
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	6,923.7	81,314.1
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	3.8	196.8
14. HOURS GENERATOR ON LINE	720.0	6,852.6	79,911.2
15. UNIT RESERVE SHUTDOWN HOURS	0.0	3.2	181.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,089,345	10,149,672	110,406,516
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	366,540	3,406,470	37,427,480
18. NET ELECTRICAL ENERGY GENERATED (MWH)	350,496	3,249,350	35,632,816
19. UNIT SERVICE FACTOR	100.0	85.5	88.2
20. UNIT AVAILABILITY FACTOR	100.0	85.5	88.4
21. UNIT CAPACITY FACTOR (USING MDC NET)	98.3	81.9	80.1
22. UNIT CAPACITY FACTOR (USING DER NET)	97.7	81.6	79.2
23. UNIT FORCED OUTAGE RATE	0.0	0.1	1.5
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	Eleven-week refueling outage scheduled to begin March 25, 1983.		

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN

DOCKET NO.	<u>50-301</u>
UNIT NAME	<u>Point Beach Unit 2</u>
DATE	<u>December 6, 1982</u>
COMPLETED BY	<u>C. W. Fay</u>
TELEPHONE	<u>414/277-2811</u>

AVERAGE DAILY UNIT POWER LEVEL

MONTH November, 1982

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>487</u>	11	<u>487</u>	21	<u>488</u>
2	<u>487</u>	12	<u>491</u>	22	<u>487</u>
3	<u>487</u>	13	<u>485</u>	23	<u>488</u>
4	<u>485</u>	14	<u>469</u>	24	<u>487</u>
5	<u>487</u>	15	<u>485</u>	25	<u>488</u>
6	<u>486</u>	16	<u>486</u>	26	<u>488</u>
7	<u>488</u>	17	<u>489</u>	27	<u>487</u>
8	<u>487</u>	18	<u>491</u>	28	<u>488</u>
9	<u>487</u>	19	<u>486</u>	29	<u>488</u>
10	<u>487</u>	20	<u>489</u>	30	<u>488</u>
				31	<u></u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1982DOCKET NO. 50-301UNIT NAME Point Beach Unit 2DATE December 6, 1982COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code	Cause and Corrective Action To Prevent Recurrence
None									

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-301
Unit Name Point Beach Unit 2
Date December 6, 1982
Completed By C. W. Fay
Telephone 414/277-2811

Unit 2 operated at approximately 488 MWe net during the entire reporting period with no major load reductions.

On November 22, 1982, during a review of computer data, it was discovered that the axial flux differential monitoring program and the associated alarm had been inadvertently halted during the period from 1508 hours, November 20, 1982 to 1008 hours on November 22, 1982, without manually logging the flux differential as required by Technical Specification 15.3.10 B.2.F. However, the flux differential had been continuously displayed and recorded in the control room and examination of these recordings disclosed no anomalies. A 14-day Licensee Event Report has been filed with the NRC.

No other safety-related maintenance was performed during the period.