OPERATING DATA REPORT

50-289 DOCKET NO. December 15, 1982 DATE C. W. Smyth COMPLETED BY TELEPHONE ______ 948-8551

OPERATING STATUS

Three Mile Island Nuclear Station, Unit I 1. Unit Name:

November, 1982 2. Reporting Period: .

3. Licensed Thermal Power (MWt): _____2535

871 4. Nameplate Rating (Gross MWe): ____

5. Design Electrical Rating (Net MWe): 819

840 6. Maximum Dependable Capacity (Gross MWe):

776 7. Maximum Dependable Capacity (Net MWe):

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yrto-Date	Cumulative
11 Hours In Reporting Period	720.	8016.	72289.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	1.0	840.5
14 Hours Generator On-Line	0.0	0.0	31180.9
15 Linit Reserve Shutdown Hours	0.0	0.0	0.0
16 Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17 Cross Floatnian Energy Generated (MWH)	0.	0.	25484330.
1. Gross Electrical Energy Generated (MWH)	0.	0.	23840053.
18. Net Electrical Energy Generated (MWH)	0.0	0.0	43.1
19. Unit Service Factor	0.0	0.0	43.1
20. Unit Availability Factor	0.0	0.0	42.1
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	40.3
22. Unit Capacity Factor (Using DER Net)	100.0	100.0	52.0
23. Unit Forced Outage Rate	100.0	100.0	52.0
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration	of Each):	

25.	If Shut Down At End Of Report Period, Estimated Date of Startup:	
26.	Units In Test Status (Prior to Commercial Operation):	Forecast

Forecast

Achieved

INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-289	
UNIT	TMI-I	
DATE	December 15,	1982
COMPLETED BY	C. W. Smyth	
TELEPHONE	(717) 948-855	51

MONTH	November, 1982
DAY	AVERAGE DAILY POWER LEVEL (Mwe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

					UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT MONTH <u>November</u> , 1982				DOCKET NO. UNIT NAME DATE COMPLETED BY TELEPHONE 50-289 TMI-I December 15, 19 C. W. Smyth (717) 948-8551
No.	Date	Type ¹	Duration (Hours)	Reason -	Method of Shutting Down Reactor3	Licensee Event Report #	System Code ⁴	Component Cude ⁵	Cause & Corrective Action to Prevent Recurrence
1	11/1/82	F	720	D	1				Regulatory Restraint Order
F Forced S Scheduled		2 Reason A Equipment Failure (Explain) B-Maintenance or Test C Retueling D-Regulatory Restriction E Operator Training & License Examination F A-Iministrative G Operational Error (Explain) H Other (Explain)				ination	3 Method 1-Manu: 2-Manu: 3-Auton 4-Other	al al Scram. natic Scram. (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit 1 - Same Source

OPERATING SUMMARY

Unit I remained in a cold shutdown the entire month by order of the NRC. The Reactor Coolant System remained partially drained to permit repair work on both OTSGs. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

In addition to continuing work on mestart modifications, the following maintenance activities were performed.

The Once Through Steam Generator (OTSG) program continued with the following items accomplished.

- 1. RC-H-1A ("A" OTSG)
 - A. Crevis drying and dehumidification.
 - B. Completed first explosive expansion of approximately 15,270 tubes.
 - C. Drained immunol from generator.
 - D. Prepared for tube cleaning and testing.
- 2. RC-H-1B ("A" OTSG)
 - A. Crevis drying and dehumidification.
 - B. Completed first explosive expansion of approximately 15,443 tubes.
 - C. Drained immunol from generator.
 - D. Preparations for tube cleaning and testing.

The Local Leak Rate Testing program continued with the following valves testing satisfactorily.

- 1. IC-V-2
- 2. IC-V-4
- 3. IC-V-6
- 4. IC-V-16
- 5. WDG-V-3
- 6. WDG-V-4

The River Water Fire Pump (FS-P-2) overhaul was completed with the pump reassembled, electric motor reinstalled, and all tests being performed satisfactorily.

The WDG-V-4 piping replacement work commensed with the prefabrication of piping in progress. PT inspections of welds were performed.

A planned building spray outage commensed with the following major work items and inspections being completed.

1. "A" Loop

1.1

- A. Drained loop.
- B. Opened valves BS-V-21A/23A for chemistry samples and closed valves.
- C. BS-P-1A work items:
 - Removed suction spool piece for chemistry sample and reinstalled.
 - (2) Repaired various threaded pipe leaks.
 - (3) Repaired discharge flange leak.
 - (4) Installed pump shaft coupling guard.
- D. Repaired flange leak on valve BS-V-34A.
- E. Installed blank flanges at valves BS-V-50A/56A.
- F. Repacked valve BS-V-60A/49A.
- G. Relief valve maintenance BS-V-45A/63A.
- 2. "B" Loop
 - A. Drained loop.
 - B. Repacked valve BS-V-3B.
 - C. Completed radiography inspections as follows:
 - (1) B-99
 - (2) B-11
 - (3) BS-FW-301
 - (4) BS-SW-302
 - (5) BS-SW-301
 - (6) BS-SW-303
 - (7) BS-SW-304
 - (8) BS-SW-305

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).