



Entergy
Operations

Entergy Operations, Inc.

PO Box 8

North LA 70006

TEL 504-735-8650

Ref: 10CFR50.73(a)(2)(i)

W3B5-91-0062

A4.05

QA

March 4, 1991

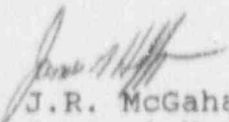
U.S. Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, D.C. 20555

Subject: Waterford 3 SFS
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Gentlemen:

Attached is Licensee Event Report Number LER-91-001-00 Waterford
Steam Electric Station Unit 3. This Licensee Event Report is
submitted pursuant to 10CFR50.73(a)(2)(i).

Very truly yours,

 for J.R. McGaha

J.R. McGaha
General Manager - Plant Operations

JRM/DDW/rk
Attachment

cc: Messrs. R.D. Martin
G.L. Florreich
J.T. Wheelock - INPO Records Center
E.L. Blake
D.L. Wigginton
NRC Resident Inspectors Office

JE22
11

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH 11-
INFORMATION COLLECTION REQUEST 400 HRS. FORWARD
COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORD
AND REPORTS MANAGEMENT BRANCH (P-530) U.S. NUCLEA
REGULATORY COMMISSION WASHINGTON, DC 20545 AND T
THE PAPERWORK REDUCTION PROJECT (3150-0104) OFFICE
OF MANAGEMENT AND BUDGET WASHINGTON, DC 20503

FACILITY NAME (1):

DOCKET NUMBER (2):

PAGE (3)

Waterford Steam Electric Station Unit 3

0 5 0 0 0 3 8 1 2 1 OF 0 2

TITLE (4):

Failure to Place Plant Protection Channel in Bypass during Surveillance Testing

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)			
02	03	91	91	001	00	03	04	91	N/A	0 5 0 0 0 0 0 0 0 0 0 0			
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11):										
1			20.402(b)			20.405(c)			60.73(a)(2)(vi)			73.71(b)	
POWER LEVEL (10)			20.405(a)(1)(i)			60.36(c)(1)			60.73(a)(2)(iv)			73.71(c)	
1 0 0			20.405(a)(1)(ii)			60.36(c)(2)			60.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)	
			20.405(a)(1)(iii)			X 60.73(a)(2)(ii)			60.73(a)(2)(viii)(A)				
			20.405(a)(1)(iv)			60.73(a)(2)(iii)			60.73(a)(2)(viii)(B)				
			20.405(a)(1)(v)			60.73(a)(2)(iii)			60.73(a)(2)(ix)				

LICENSEE CONTACT FOR THIS LER (12):

NAME

TELEPHONE NUMBER

R.S. Starkey, Operations Superintendent

AREA CODE

5 1 0 4 4 6 4 - 1 3 1 1 3 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On February 3, 1991, during the performance of Operating Procedure OP-903-107, "Plant Protection System Channel B Functional Test," Steam Generator 2 low pressure setpoint was determined to be out of tolerance. Contrary to Technical Specification 3.3.1 Action Statement 2, channel B was not placed in the bypassed or tripped condition within 1 hour. This event is reportable as a condition prohibited by Technical Specifications.

This event was caused by an inappropriate action resulting from a failure to recognize that the channel was required to be placed in bypass during the surveillance test. The inoperable channel bistables were placed in bypass when the error was realized. Since the other three Steam Generator 2 low pressure channels were within specification, this event did not result in an increased risk to the health and safety of the public or plant personnel.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Waterford Steam
Electric Station Unit 3

0 5 0 0 0 3 8 2 9 1 - 0 0 1 - 0 0 0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On February 3, 1991, during the performance of Operating Procedure OP-903-107, Plant Protection System (EIIIS Identifier JC) Channel B Functional Test, Steam Generator (EIIIS Identifier SG) 2 low pressure (EIIIS Identifier PA) setpoint was determined to be out of tolerance. Contrary to Technical Specification (TS) 3.3.1 Action Statement 2, the inoperable channel was not placed in the bypassed or tripped condition within 1 hour. This event is reportable as a condition prohibited by TSs.

At 0254 on February 3, 1991, Waterford Steam Electrical Station Unit 3 was operating at 100% power when Steam Generator 2 low pressure setpoint was declared inoperable. Personnel performing OP-903-107 did not place the inoperable channel in the bypassed or tripped position, and continued performing the remainder of the surveillance procedure contrary to limitation 3.2.2 of OP-903-107. After the entire surveillance procedure had been performed, personnel reviewed the results and realized that the affected channel should have been placed in bypass within 1 hour. At 0524, the inoperable channel was placed in bypass.

The personnel involved in performing the procedure have been counseled. The operations department will include this report in departmental required reading prior to March 15, 1991.

Although the time requirement of placing the inoperable channel B in bypassed or tripped position was not met, the other three channels of SG low pressure were operable; therefore, this event did not result in an increased risk to the health and safety to the public or plant personnel.

SIMILAR EVENTS

NONE

PLANT CONTACT

R.S. Starkey, Operations Superintendent, (504)464-3134