

NOTICE OF VIOLATION

Commonwealth Edison Company
Byron Nuclear Power Station

Docket Nos. 50-454; 50-455
Licenses No. NPF-37; NPF-66

During an NRC inspection conducted on May 9 - 27, 1994, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, the violations are listed below:

1. 10 CFR 50, Appendix B, Criterion XVI, states, in part, that measures shall be established to assure that conditions adverse to quality, such as failures, malfunctions, deficiencies are promptly identified and corrected. In the case of significant conditions adverse to quality, the measures shall assure that the cause of the condition is determined and corrective action taken to preclude repetition.

Contrary to the above, as of May 23, 1994, corrective actions to address design deficiencies identified in September of 1990 (minimal design margin) with 16 auxiliary feedwater to steam generator isolation valves (AF-013s) were not prompt and the cause of the September 1993 failures (unknown degradation) was not determined or corrected.

This is a Severity Level IV violation (Supplement 1)

2. 10 CFR 50, Appendix B, Criterion V, states, in part, that activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, of a type appropriate to the circumstances and shall be accomplished in accordance with these instructions, procedures, or drawings.

Contrary to the above, as of May 23, 1994, steps to clean and lubricate the entire stems and stem nuts for the AF-013 valves (activities affecting quality) were not prescribed by documented instructions, procedures, or drawings of a type appropriate to the circumstances and were not accomplished.

This is a Severity Level IV violation (Supplement 1)

3. 10 CFR Part 50, Appendix B, Criterion X, Inspection, requires that inspection of activities affecting quality shall be established and executed by or for the organization performing the activity to verify conformance with the documented instructions, procedures, and drawings for accomplishing the activity.

Contrary to the above, in three cases, including installation of relief valve 1SA148A on the Emergency Diesel Generator air start system, inadequate inspection allowed conditions outside of design tolerance to continue without identification or correction.

This is a Severity Level IV violation (Supplement 1).

Pursuant to the provisions of 10 CFR 2.201, Commonwealth Edison Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington D.C. 20555 with a copy to the U. S. Nuclear Regulatory Commission, Region III, 801 Warrenville Road, Lisle, Illinois, 60532-4351, and a copy to the NRC Resident Inspector at the Byron Nuclear Power Plant within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or a demand for information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Lisle, Illinois
this 1st day of July, 1994.