

NRC FORM 368 (6-89)										U.S. NUCLEAR REGULATORY COMMISSION										APPROVED OMB NO. 3160-0104 EXPIRES: 4/30/87																																		
LICENSEE EVENT REPORT (LER)																																																						
FACILITY NAME (1)																				DOCKET NUMBER (2)										PAGE (3)																								
Peach Bottom Atomic Power Station Unit 3																				05000278										1 OF 04																								
TITLE (4) Technical Specification Violation when a Suppression Chamber - Reactor Building Vacuum Breaker Surveillance Requirement was not performed																																																						
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																								
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										DOCKET NUMBER(S)																	
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08			08			93			93			007			00			09			16			93													050000																	
OPERATING MODE (9)										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 43. (Check one or more of the following) (11)																																												
N																																																						
POWER LEVEL (10)										20.402(e)										20.406(e)										60.73(a)(2)(iv)										72.71(b)														
01918										20.406(a)(1)(ii)										60.36(e)(1)										60.73(a)(2)(v)										72.71(c)														
										20.406(a)(1)(iii)										60.36(e)(2)										60.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 368A)														
										20.406(a)(1)(iii)										X 60.73(a)(2)(ii)										60.73(a)(2)(viii)(A)																								
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LICENSEE CONTACT FOR THIS LER (12)																																																						
NAME																				TELEPHONE NUMBER																																		
Anthony J. Wasong Manager - Experience Assessment																				717 456-7014																																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																						
CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRC					CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRC									
SUPPLEMENTAL REPORT EXPECTED (14)																																																						
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)																								

On 08/20/93, the Surveillance Test Coordinator (STC) identified that a Surveillance Test (ST) (SI3D-7-3503-B1CR) had not been performed prior to exceeding the Technical Specification (Tech Spec) out of surveillance date. This resulted in a violation of Tech Spec 4.7.A.3.a on 08/08/93. The cause of the event was that the initial review of the proposed Tech Spec change to determine its impact on individual Surveillance Test frequencies was inadequate. After discovery of the event, SI3D-7-3503-B1CR was completed satisfactorily on 08/20/93 to ensure that the instrumentation and setpoints associated with these Vacuum Breakers were operable. In addition, completion of the review of this Tech Spec Amendment verified that this was the only case where an ST was not completed within its allowable surveillance frequency. The event has been discussed with the involved individuals. The pertinent information from the event will be provided to the appropriate Site Engineering personnel to emphasize the importance of attention to detail when reviewing Tech Spec Proposals and approved Amendments. No actual safety consequences occurred as a result of this event. There were no previous similar events identified.

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PDR ADDCK 05000278
S PDR

U.S. NUCLEAR REGULATORY COMMISSION

NRC FORM 365A
(6-89)

APPROVED OMB NO. 3150-0104

EXPIRES: 4/30/92

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS
INFORMATION COLLECTION REQUEST: 400 HRS. ESTIMATED
COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS
MANAGEMENT BRANCH (3150-0104) U.S. NUCLEAR
THE PAPERWORK REDUCTION PROJECT (3150-0104) OFFICE
OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Peach Bottom Atomic Power Station
Unit 3

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

Requirements of the Report

This report is submitted pursuant to 10 CFR 50.73 (a)(2)(i)(B) as a result of a Technical Specification (Tech Spec) violation when a Suppression Chamber-Reactor Building Vacuum Breaker Pressure Switch Surveillance Test (ST) was not performed within its allowable eighteen month test frequency.

Unit Conditions at Time of Event

Unit 3 was in the "RUN" mode at approximately 98 % of thermal reactor (EHS:EA) power. There were no systems, structures, or components that were inoperable that contributed to the event.

Description of the Event

On 08/20/93, during a review of an approved Tech Spec Amendment (179/182) "EXTENSION OF TECHNICAL SPECIFICATION SURVEILLANCE INTERVALS TO 24 MONTHS", the Surveillance Test Coordinator (STC) identified that a Surveillance Test (SICD-7-3503-B1CR) "CALIBRATION CHECK OF SUPPRESSION CHAMBER - REACTOR BUILDING PRESSURE INSTRUMENT DPIS 3503B" had not been performed prior to exceeding the Tech Spec out of surveillance date. The test was performed twelve days late. This resulted in a violation of Tech Spec 4.7.A.3.a on 08/08/93. Tech Spec 4.7.A.3.a specifies that "The pressure suppression chamber - reactor building vacuum breakers shall be checked for operation every refueling outage. Associated instrumentation including setpoint shall be checked for operation every eighteen months".

After discovery of the event on 08/20/93, SICD-7-3503-B1CR was completed satisfactorily to ensure that the instrumentation and setpoints associated with these Vacuum Breakers were operable. In addition, completion of the review of this Tech Spec Amendment verified that this was the only case where an ST was not completed within its allowable surveillance frequency.

Cause of the Event

The cause of the event was that the initial review of the proposed Tech Spec change to determine its impact on Individual Surveillance Test frequencies was inadequate. Tech Spec Amendment (179/182) allowed individual tests to be requested for extension of "Refuel" to have their frequency extended from 18 months to 24 months. PECC requested this amendment in late 1992 to support 24 month fuel cycles. This initial review in 1992 was intended to identify and list which of these tests needed to retain an 18

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 4/30/93

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-830), U.S. NUCLEAR REGULATORY COMMISSION, REGISTRATION DIV., DC 20585, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Beach Bottom Atomic Power Station
Unit 3

YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

month frequency. Review of the Tech Spec change for impact on surveillance testing was performed by Station Engineering (Utility : Non Licensed) in 1992. The review failed to identify that SI3D-7-3503-B1CR needed to retain an 18 month testing frequency. The review was performed using a list of Tech Spec Implementation Items generated by Corporate Engineering. However, the Corporate Engineering list was not a complete list and did not list SI3D-7-3503 B1CR as needing to be retained at a frequency of 18 months. The list was intended to be used as an aid only. Subsequently, this Tech Spec change was approved and issued by the NRC on 08/02/93. The Tech Spec Amendment was issued on the same day it was needed to prevent a Unit 3 shutdown due to various STs being out of surveillance. Using the initial 1992 review list for guidance, the STC changed all "Once per Cycle" and "Refuel" tests to a 24 month frequency in the ST Tracking Program except for those listed on the 1992 review list as maintaining an 18 month frequency. Since SI3D-7-3503-B1CR was not on the initial review list due to an inadequate review, its frequency was changed to 24 months when it should have remained at 18 months. Therefore, SI3D-7-3503-B1CR was not performed on time. This mistake was discovered during a subsequent review of the approved Tech Spec Amendment by STC.

Analysis of the Event

No actual safety consequences occurred as a result of this event.

The consequences are considered minimal due to the fact that SI3D-7-3503-B1CR was completed satisfactorily on 08/20/93 which ensured that the instrumentation associated with the Suppression Chamber - Reactor Building Vacuum Breakers were operable during the time that the ST was out of surveillance.

Corrective Actions

After discovery of the event, SI3D-7-3503-B1CR was completed satisfactorily on 08/20/93 to ensure that the instrumentation and signals associated with these Vacuum Breakers were operable.

In addition, completion of the review of this Tech Spec Amendment verified that this was the only case where an ST was not completed within its allowable surveillance frequency.

NRC FORM 386A
10-89

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 4/30/92

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (8)

PAGE (3)

Peach Bottom Atomic Power Station
Unit 3

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

The event has been discussed with the involved individuals. The pertinent information from the event will be provided to the appropriate Site Engineering personnel to emphasize the importance of attention to detail when reviewing Tech Spec Proposals and approved Amendments.

Previous Similar Events

There were no previous similar events identified involving a less than adequate review of Tech Spec Proposals or approved Amendments.



GARRETT D. EDWARDS
PLANT MANAGER

PHILADELPHIA ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION

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REF. TO

September 16, 1993

Docket No. 50-278

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Peach Bottom Atomic Power Station - Unit 3

This LER concerns a Technical Specification violation when a Suppression Chamber-Reactor Building Vacuum Breaker Surveillance Requirement was not performed.

Reference:	Docket No. 50-278
Report Number:	3-93-007
Revision Number:	00
Event Date:	08/08/93
Discovery Date:	08/20/93
Report Date:	09/16/93
Facility:	Peach Bottom Atomic Power Station RD1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B).

Sincerely,

cc: R. A. Burricelli, Public Service Electric & Gas
W. P. Dornsife, Commonwealth of Pennsylvania
INPO Records Center
T. T. Martin, US NRC, Administrator, Region I
R. I. McLean, State of Maryland
B. S. Norris, US NRC, Resident Inspector
C. D. Schaefer, DelMarVa Power
H. C. Schwemm, VP - Atlantic Electric

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