December 2, 1982

Carolina Power and Light Company ATTN: Mr. E. E. Utley Executive Vice President 411 Fayetteville Street Raleigh, NC 27602

Gentlemen:

Subject: Meeting Summary-Brunswick Steam Electric Plant Units 1 and 2 Docket Nos. 50-325 and 50-324

This refers to the meeting held with members of your staff at the Brunswick Steam Electric Plant on November 17. 1982. This meeting related to activities authorized by NRC Operating License Nos. DPR-17 and DPR-62. The subjects discussed are detailed in the anclosed meeting summary.

It is our opinion that this meeting was beneficial and has provided for a mutually better understanding of these issues. Your cooperation with us is appreciated. In accordance with Section 2.790 of NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and enclosure will be placed in the NRC's Public Document Room.

Should you have any questions concerning this matter, we will be pleased to discuss them.

Sincerely,

James P. O'Reilly Regional Administrator

Enclosure: Meeting Summary

cc w/encl:

C. R. Dietz, Plant Manager

bcc w/encl: NRC Resident Inspector Document Management Branch State of North Carolina

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ENCLOSURE

MEETING SUMMARY

Subjects Discussed

The primary purpose of the meeting, held on November 17, 1982, at the Brunswick Steam Electric Plant, was to discuss the integrated and local leak test programs (10 CFR 50, Appendix J) being performed at the site and the cracked weld in the residual heat removal (RHR) shutdown cooling suction piping and the subsequent repair/evaluation. (For background see RII Report 50-324/82-41.) With regard to the cracked weld, the licensee had completed the weld repair and was in the process of preparing the weld for the appropriate non-destructive examinations. The licensee is performing a metallurgical examination to assist in determining the mode of failure and plans to submit a detailed report to NRC by November 30, 1982.

The discussion on 10 CFR 50, App. J, centered on resolving questions by CP&L concerning compliance with the requirements. The question concerning premaintenance leak testing of isolation valves was determined to not be a requirement. This matter is resolved. Also resolved were questions on leak test requirements on reactor instrument valves, lines which are submerged below the torus water level, and venting and draining of systems within the containment building.

Two issues yet to be completely resolved deal with adjusting the past ILRT with each LLRT, and the venting and draining requirement for nonsafety systems outside the containment building. Resolution of these issues are considered to be of low priority and will be discussed by CP&L at a future BWR Owner's Group meeting.

In addition, the following topics were discussed briefly:

- a) Maintenance and test program for the MSIV's.
- b) Experience and history of safety-relief valves, and the participation by CP&L personnel in the BWR Owner's group, established to resolve the setpoint variability problem.
- c) Discussion of the requirements and performance history of the secondary containment test program.

2. Meeting Attendance

Licensee Representatives

- P. Howe, Vice President, Brunswick Nuclear Project
- C. R. Dietz, General Manager, Brunswick Steam Electric Plant S. A. Bishop, Manager of Technical Support

D. Warren, Sentor Engineer

J. Boone, Engineering Supervisor W. L. Johnson, Operations Engineer

G. Locklear, Project Specialist-Mechanical

F. Repass, Maintenance Technician I

B. E. Hinkley, NSSS Project Specialist E. Betz, Project QA/QC Specialist

C. R. Osman, Principal QA/QC Specialist

NRC

A. R. Herdt, Chief, Engineering Inspection Branch, DETP F. Jape, Chief, Test Programs Section, Engineering Inspection Branch, DETP