

**Florida
Power**

CORPORATION
Crystal River Unit 3
Docket No. 60-302

June 22, 1994
3F0694-01

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Subject: Status of Generic Safety Issue (GSI) Implementation

Reference: 1. NRC to FPC letter, 3N0490-29, dated April 25, 1990
2. FPC to NRC letter, 3F0690-10, dated June 21, 1990
3. NRC to FPC letter, 3N1190-11, dated November 26, 1990

Dear Sir:

Reference 1 requested Florida Power Corporation (FPC) provide information for Crystal River Unit 3 (CR-3) on the implementation status of generic safety issues (GSIs) resolved with the imposition of requirements or corrective actions. The FPC response (Reference 2) and subsequent discussions with the NRC Staff resulted in issuance of Reference 3, which contained the NRC list of unimplemented GSIs for CR-3. The purpose of this letter is to update the status of the remaining unimplemented GSIs identified in Reference 3.

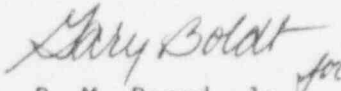
On December 20, 1993, the NRC issued Amendment 149 "Improved Technical Specifications" to the CR-3 Operating License. This Amendment incorporated the additional requirements in the CR-3 Technical Specifications necessary to close GSI No. 75 (MPA No. B090), "Item 4.3, Reactor Trip System Reliability-Technical Specification Changes", GSI No. 75 (MPA No. B091), "Item 4.4, Reactor Trip System Reliability", and GSI No. A-35 (MPA No. B023) "Adequacy of Offsite Power Supplies".

ADD 1

The Multi-Plant Actions (MPAs) associated with GSI No. 75 requested Technical Specification changes to assure functionality of the reactor trip breaker shunt trip mechanism and Reactor Protection System silicon controlled rectifiers (SCR). These changes are addressed in ITS Specification 3.3.4 "Control Rod Drive Trip Devices." The CR-3 specific terminology for the SCRs described in the MPA is electronic trip assembly (ETA) relay. GSI No. A-35 requested Technical Specifications be added for degraded voltage relays and the associated instrumentation. These changes are addressed in ITS Specification 3.3.8 "Emergency Diesel Generator Loss of Power Start."

The other GSIs identified in Reference 3 have been previously implemented and this information transmitted to you under separate cover letter. Refer to the Attachment to this letter. Therefore, FPC considers the implementation status of all GSIs requested in Reference 1 to be complete for CR-3 with the issuance of Amendment 149. Future updates to NUREG 1435 should reflect this status.

Sincerely,



P. M. Beard, Jr.
Senior Vice President
Nuclear Operations

PMB/BPW:ff

Attachment

xc: Regional Administrator, Region II
Senior Resident Inspector
NRR Project Manager

Attachment to 3F0694-01

Status of Generic Safety Issues (GSI) Identified
in NRC to FPC letter dated November 26, 1990

<u>GSI NO.</u>	<u>TITLE</u>	<u>CLOSURE DOCUMENTATION REFERENCES</u>
43	Reliability of Air Systems	FPC to NRC letter, 3F0591-08, dated May 30, 1991. NRC to FPC letter, 3N0891-05, dated August 8, 1991.
75	Salem ATWS Item 2.2.2, Vendor Interface for Safety-Related Components	FPC to NRC letter, 3F0491-10, dated April 30, 1991. NRC to FPC letter, 3N1090-20, dated October 30, 1990.
75	Item 4.3, Reactor Trip System Reliability-TS Changes	FPC to NRC letter, 3F1093-12, dated October 15, 1993. NRC to FPC letter, 3N1293-30, dated December 20, 1993.
75	Item 4.4, Reactor Trip System Reliability	FPC to NRC letter, 3F1093-12, dated October 15, 1993. NRC to FPC letter, 3N1293-30, dated December 20, 1993.
124	Auxiliary Feedwater System Reliability ¹	FPC to NRC letter, 3F0692-22, dated June 25, 1992.
A-35	Adequacy of Offsite Power Supplies	FPC to NRC letter, 3F1093-12, dated October 15, 1993. NRC to FPC letter, 3N1293-30, dated December 20, 1993.

¹ FPC considers GSI 124 implementation to be complete for CR-3. NRC's June 6, 1994 request for information relating to the current functional status of Feedwater Pump 7 are being addressed separately.