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Docket Number 50-346

License Number NPF-3

Serial Number 2229

June 20, 1994

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: Supplemental Information For License Amendment Application to Revise Technical Specifications and Applicable Bases to Relocate Certain Cycle-Specific Protective Limits and Core Operating Limits to the Core Operating Limits Report (COLR)

Gentlemen:

By letter dated March 18, 1994 (Serial Number 2197), Toledo Edison (TE) submitted an application for an amendment to the Davis-Besse Nuclear Power Station (DBNPS), Unit Number 1 Operating License Number NPF-3, Appendix A, Technical Specifications proposing changes to Technical Specification (TS) 2.1.2 (Reactor Core), TS 2.2.1 (Reactor Protection System Setpoints), Bases 2.1.1 and 2.1.2 (Reactor Core), Bases 2.2.1 (Reactor Protection System Instrumentation Setpoints), TS 3.2.2 (Power Distribution Limits, Nuclear Heat Flux Hot Channel Factor - F_Q), TS 3.2.3 (Power Distribution Limits, Nuclear Enthalpy Rise Hot Channel Factor - $F_{\Delta H}^N$), Bases 3/4.2 (Power Distribution Limits), TS 6.5.1.6 (Safety Review Board, Responsibilities), TS 6.7 (Safety Limit Violation), and TS 6.9.1.7 (Reporting Requirements, Core Operating Limits Report).

The purpose of this letter is to request the incorporation of several minor clarifications to the changes previously proposed by TE letter Serial No. 2197 for TS 6.9.1.7, as described below. These changes were requested by the NRC Staff during telephone discussions with Toledo Edison in May and June, 1994.

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Operating Companies
Cleveland Electric Illuminating
Toledo Edison

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Toledo Edison letter Serial No. 2197 proposed that references to TS 2.1.2 and 2.2.1 be included in TS 6.9.1.7 since these Technical Specifications will refer to information included in the Core Operating Limits Report (COLR). The references to be added were to read:

- 2.1.2 Reactor Core
- 2.2.1 Reactor Protection System Setpoints

It is proposed that these references be changed to read:

- 2.1.2 AXIAL POWER IMBALANCE Protective Limits for Reactor Core Specification 2.1.2
- 2.2.1 Trip Setpoint for Flux -- Δ Flux/Flow for Reactor Protection System Setpoints Specification 2.2.1

In addition, TE letter Serial No. 2197 proposed that the present TS 6.9.1.7 discussion of analytical methods and listing of topical reports be replaced with the following:

The analytical methods used to determine the core operating limits addressed by the individual Technical Specifications shall be: those previously reviewed and approved by the NRC, as described in BAW-10179P-A, "Safety Criteria and Methodology for Acceptable Cycle Reload Analyses", or any other new NRC-approved analytical methods used to determine core operating limits that are not yet referenced in the applicable BAW-10179P-A revision. The applicable BAW-10179P-A revision (the approved revision at the time the reload analyses are performed) shall be listed in the CORE OPERATING LIMITS REPORT. The CORE OPERATING LIMITS REPORT shall also list any new NRC-approved analytical methods used to determine core operating limits that are not yet referenced in the applicable BAW-10179P-A revision.

It is proposed that this insert be reworded as follows:

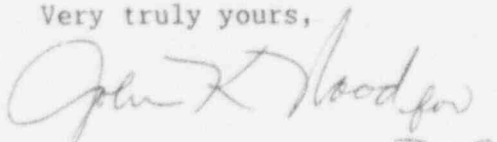
The analytical methods used to determine the core operating limits addressed by the individual Technical Specifications shall be: those previously reviewed and approved by the NRC, as described in BAW-10179P-A, "Safety Criteria and Methodology for Acceptable Cycle Reload Analyses", or any other new NRC-approved analytical methods used to determine core operating limits that are not yet referenced in the applicable approved revision of BAW-10179P-A. The applicable approved revision number for BAW-10179P-A at the time the reload analyses are performed shall be identified in the CORE OPERATING LIMITS REPORT. The CORE OPERATING LIMITS REPORT shall also list any new NRC-approved analytical methods used to determine core operating limits that are not yet referenced in the applicable approved revision of BAW-10179P-A.

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This letter provides supplemental information and the conclusion stated in TE letter Serial No. 2197, that the license amendment application does not involve a significant hazards consideration, remains unaffected.

If you have any questions, please contact Mr. William T. O'Connor, Manager - Regulatory Affairs, at (419) 249-2366.

Very truly yours,



DCS

MKL/laj

cc: J. B. Martin, Regional Administrator, NRC Region III
S. Stasek, NRC Region III, DB-1 Senior Resident Inspector
G. West, Jr., NRC/NRR DB-1 Project Manager
J. R. Williams, Chief of Staff, Ohio Emergency Management Agency,
State of Ohio (NRC Liaison)
Utility Radiological Safety Board