



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY
DOCKET NO. 50-260
BROWNS FERRY NUCLEAR PLANT, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 186
License No. DPR-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated July 13, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9101150272 910109
PDR ADOCK 05000259
P PDR

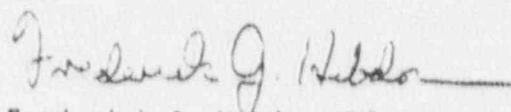
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 186, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Heddon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 9, 1991

ATTACHMENT TO LICENSE AMENDMENT NO.186

FACILITY OPERATING LICENSE NO. DPR-52

DOCKET NO. 50-260

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages* are provided to maintain document completeness.

<u>REMOVE</u>	<u>INSERT</u>
iii	iii
iv	iv*
3.9/4.9-15	3.9/4.9-15
--	3.9/4.9-15a
3.9/4.9-19	3.9/4.9-19*
3.9/4.9-20	3.9/4.9-20
3.9/4.9-21	3.9/4.9-21
3.9/4.9-22	3.9/4.9-22*

Section

Page No.

E.	Jet Pumps	3.6/4.6-11
F.	Recirculation Pump Operation	3.6/4.6-12
G.	Structural Integrity	3.6/4.6-13
H.	Snubbers	3.6/4.6-15
3.7/4.7	Containment Systems	3.7/4.7-1
A.	Primary Containment.	3.7/4.7-1
B.	Standby Gas Treatment System	3.7/4.7-13
C.	Secondary Containment.	3.7/4.7-16
D.	Primary Containment Isolation Valves	3.7/4.7-17
E.	Control Room Emergency Ventilation	3.7/4.7-19
F.	Primary Containment Purge System	3.7/4.7-21
G.	Containment Atmosphere Dilution System (CAD)	3.7/4.7-22
H.	Containment Atmosphere Monitoring (CAM) System H ₂ Analyzer	3.7/4.7-24
3.8/4.8	Radioactive Materials	3.8/4.8-1
A.	Liquid Effluents	3.8/4.8-1
B.	Airborne Effluents	3.8/4.8-3
C.	Radioactive Effluents - Dose	3.8/4.8-6
D.	Mechanical Vacuum Pump	3.8/4.8-6
E.	Miscellaneous Radioactive Materials Sources.	3.8/4.8-7
F.	Solid Radwaste	3.8/4.8-9
3.9/4.9	Auxiliary Electrical System	3.9/4.9-1
A.	Auxiliary Electrical Equipment	3.9/4.9-1
B.	Operation with Inoperable Equipment.	3.9/4.9-8
C.	Operation in Cold Shutdown	3.9/4.9-15
D.	Unit 3 Diesel Generators Required for Unit 2 Operation	3.9/4.9-15a

<u>Section</u>	<u>Page No.</u>
3.10/4.10 Core Alterations	3.10/4.10-1
A. Refueling Interlocks	3.10/4.10-1
B. Core Monitoring	3.10/4.10-4
C. Spent Fuel Pool Water	3.10/4.10-7
D. Reactor Building Crane	3.10/4.10-8
E. Spent Fuel Cask	3.10/4.10-9
F. Spent Fuel Cask Handling-Refueling Floor . . .	3.10/4.10-10
3.11/4.11 Fire Protection Systems	3.11/4.11-1
A. Fire Detection Instrumentation	3.11/4.11-1
B. Fire Pumps and Water Distribution Mains	3.11/4.11-2
C. Spray and/or Sprinkler Systems	3.11/4.11-7
D. CO ₂ Systems	3.11/4.11-8
E. Fire Hose Stations	3.11/4.11-9
F. Yard Fire Hydrants and Hose Houses	3.11/4.11-11
G. Fire-Rated Assemblies	3.11/4.11-12
H. Open Flames, Welding and Burning in the Cable Spreading Room	3.11/4.11-13
5.0 Major Design Features	5.0-1
5.1 Site Features	5.0-1
5.2 Reactor	5.0-1
5.3 Reactor Vessel	5.0-1
5.4 Containment	5.0-1
5.5 Fuel Storage.	5.0-1
5.6 Seismic Design	5.0-2

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.C. Operation in Cold Shutdown

Whenever the reactor is in COLD SHUTDOWN CONDITION with irradiated fuel in the reactor, the availability of electric power shall be as specified in Section 3.9.A except as specified herein.

1. At least two Units 1 and 2 diesel generators and their associated 4-kV shutdown boards shall be OPERABLE.
2. An additional source of power energized and capable of supplying power to the Units 1 and 2 shutdown boards consisting of at least one of the following:
 - a. One of the offsite power sources specified in 3.9.A.1.c.
 - b. A third OPERABLE diesel generator.
3. At least one 480-V shutdown board for each unit must be OPERABLE.
4. One 480-V RMOV board mg set is required for each RMOV board (2D or 2E) required to support operation of the RHR system in accordance with 3.5.B.9.

4.9.C Operation in Cold Shutdown

1. No additional surveillance is required.

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.D Unit 3 Diesel Generators
Required for Unit 2 Operation

1. Whenever any of the following equipment is required to be OPERABLE in accordance with the corresponding section of these technical specifications, the Unit 3 diesel generator aligned to supply emergency power to that equipment shall be OPERABLE.
 - a. Standby gas treatment train C in accordance with T.S. 3.7.B (diesel generator 3D).
 - b. Control room emergency ventilation train B in accordance with T.S. 3.7.E (diesel generator 3B or 3C).
2. When the diesel generator aligned to supply emergency power to the equipment in 3.9.D.1.a or b is inoperable, the equipment may be considered OPERABLE for the purpose of satisfying the corresponding technical specification during the succeeding 30 days provided that the redundant train(s) of equipment and their normal and emergency power supplies are OPERABLE.
3. If Specification 3.9.D.2 cannot be met, the affected equipment shall be declared inoperable.

4.9.D Unit 3 Diesel Generators
Required for Unit 2 Operation

- 1.a Diesel Generators

Surveillance requirements are as specified in T.S. 4.9.A.1.a, 4.9.A.1.c, 4.9.A.1.d and 4.9.A.1.e.
- 1.b DC Power System

Surveillance requirements are as specified in T.S. 4.9.A.2.
- 1.c Logic Systems

Both divisions of the common accident signal logic system shall be tested every 6 months to demonstrate that it will function on actuation of the core spray system of the reactor to provide an automatic start signal to each diesel generator.
- 1.d Undervoltage Relays

Surveillance requirements are as specified in T.S. 4.9.A.4.
2. No surveillance required.
3. No surveillance required.

3.9 BASES

The objective of this specification is to assure an adequate source of electrical power to operate facilities to cool the plant during shutdown and to operate the engineered safeguards following an accident. There are three sources of alternating current electrical energy available, namely, the 161-kV transmission system, the 500-kV transmission system, and the diesel generators.

The unit station-service transformer B for unit 1 or the unit station-service transformer B for unit 2 provide noninterruptible sources of offsite power from the 500-kV transmission system to the units 1 and 2 shutdown boards. Auxiliary power can also be supplied from the 161-kV transmission system through the common station-service transformers or through the cooling tower transformers by way of the bus tie board. The 4-kV bus tie board may remain out of service indefinitely provided one of the required offsite power sources is not supplied from the 161-kV system through the bus tie board.

The minimum fuel oil requirement of 103,300 gallons is sufficient for seven days of full load operation of three diesels and is conservatively based on availability of a replenishment supply.

The degraded voltage sensing relays provide a start signal to the diesel generators in the event that a deteriorated voltage condition exists on a 4-kV shutdown board. This starting signal is independent of the starting signal generated by the complete loss of voltage relays and will continue to function and start the diesel generators on complete loss of voltage. should the loss of voltage relays become inoperable. The 15-day inoperable time limit specified when one of the three phase-to-phase degraded voltage relays is inoperable is justified based on the two-out-of-three permissive logic scheme provided with these relays.

A 4-kV shutdown board is allowed to be out of operation for a brief period to allow for maintenance and testing, provided all remaining 4-kV shutdown boards and associated diesel generators, CS, RHR, (LPCI and containment cooling) systems supplied by the remaining 4-kV shutdown boards, and all emergency 480-V power boards are operable.

There are eight 250-V dc battery systems, each of which consists of a battery, battery charger, and distribution equipment. Three of these systems provide power for unit control functions, operative power for unit motor loads, and alternative drive power for a 115-V ac unit-preferred mg set. One 250-V dc system provides power for common plant and transmission system control functions, drive power for a 115-V ac plant-preferred mg set, and emergency drive power for certain unit large motor loads. The four remaining systems deliver control power to the 4,160-V shutdown boards.

3.9 BASES (Cont'd)

Each 250-V dc shutdown board control power supply can receive power from its own battery, battery charger, or from a spare charger. The chargers are powered from normal plant auxiliary power or from the standby diesel-driven generator system. Zero resistance short circuits between the control power supply and the shutdown board are cleared by fuses located in the respective control power supply. Each power supply is located in the reactor building near the shutdown board it supplies. Each battery is located in its own independently ventilated battery room.

The 250-V dc system is so arranged, and the batteries sized so that the loss of any one unit battery will not prevent the safe shutdown and cooldown of all three units in the event of the loss of offsite power and a design basis accident in any one unit. Loss of control power to any engineered safeguard control circuits is annunciated in the main control room of the unit affected. The loss of one 250-V shutdown board battery affects normal control power for the 480-V and 4,160-V shutdown boards which it supplies. The station battery supplies loads that are not essential for safe shutdown and cooldown of the nuclear system. This battery was not considered in the accident load calculations.

There are two 480-V ac RMOV boards that contain mg sets in their feeder lines. These 480-V ac RMOV boards have an automatic transfer from their normal to alternate power source (480-V ac shutdown boards). The mg sets act as electrical isolators to prevent a fault from propagating between electrical divisions due to an automatic transfer. The 480-V ac RMOV boards involved provide motive power to valves associated with the LPCI mode of the RHR system. Having an mg set out of service reduces the assurance that full RHR (LPCI) capacity will be available when required. Since sufficient equipment is available to maintain the minimum complement required for RHR (LPCI) operation, a 7-day servicing period is justified. Having two mg sets out of service can considerably reduce equipment availability; therefore, the affected unit shall be placed in Cold Shutdown within 24 hours.

The offsite power source requirements are based on the capacity of the respective lines. The Trinity line is limited to supplying two operating units because of the load limitations of CSST's A and B. The Athens line is limited to supplying one operating unit because of the load limitations of the Athens line. The limiting conditions are intended to prevent the 161-kV system from supplying more than two units in the event of a single failure in the offsite power system.

Specification 3.9.D provides the operability requirements for the Unit 3 diesel generators when they serve as emergency power supplies to standby gas treatment train C and control room emergency ventilation train B when they are being considered operable for Unit 2 technical specifications. The allowable out of service time of 30 days is commensurate with the importance of the affected systems when Unit 3 is in cold shutdown, the low probability of a LOCA/Loss of offsite power and availability of onsite power to redundant trains.

4.9 BASES

The monthly tests of the diesel generators are primarily to check for failures and deterioration in the system since last use. The diesels will be loaded to at least 75 percent of rated power while engine and generator temperatures are stabilized (about one hour). The minimum 75-percent load will prevent soot formation in the cylinders and injection nozzles. Operation up to an equilibrium temperature ensures that there is no overheating problem. The tests also provide an engine and generator operating history to be compared with subsequent engine-generator test data to identify and to correct any mechanical or electrical deficiency before it can result in a system failure.

The test during refueling outages is more comprehensive, including procedures that are most effectively conducted at that time. These include automatic actuation and functional capability tests to verify that the generators can start and be ready to assume load in 10 seconds. The annual inspection will detect any signs of wear long before failure.

Battery maintenance with regard to the floating charge, equalizing charge, and electrolyte level will be based on the manufacturer's instruction and sound maintenance practices. In addition, written records will be maintained of the battery performance. The plant batteries will deteriorate with time but precipitous failure is unlikely. The type of surveillance called for in this specification is that which has been demonstrated through experience to provide an indication of a cell becoming irregular or unserviceable long before it becomes a failure.

The equalizing charge, as recommended by the manufacturer, is vital to maintaining the ampere-hour capacity of the battery, and will be applied as recommended.

The testing of the logic systems will verify the ability of the logic systems to bring the auxiliary electrical system to running standby readiness with the presence of an accident signal from any reactor or an undervoltage signal on the 4-kV shutdown boards.

The periodic simulation of accident signals in conjunction with diesel-generator voltage available signals will confirm the ability of the 480-V load shedding logic system to sequentially shed and restart 480-V loads if an accident signal were present and diesel-generator voltage were the only source of electrical power.

Specification 4.9.D provides surveillance requirements for Unit 3 diesel generators for the purpose of satisfying Specification 3.9.D. It contains less stringent testing requirements for the Unit 3 diesel generators when they are only being used to support Unit 2 equipment.

REFERENCES

1. Normal Auxiliary Power System (BFNP FSAR Subsection 8.4)
2. Standby AC Power Supply and Distribution (BFNP FSAR Subsection 8.5)
3. 250-Volt DC Power Supply and Distribution (BFNP FSAR Subsection 8.6)
4. Memorandum from Gene M. Wilhoite to H. J. Green dated December 4, 1981 (LOO 811208 664) and memorandum from C. E. Winn to H. J. Green dated January 10, 1983 (G02 830112 002)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296

BROWNS FERRY NUCLEAR PLANT, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150
License No. DPR-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated July 13, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

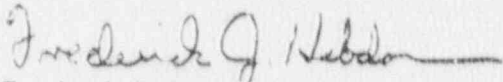
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-68 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 150, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Frederick J. Heddon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 9, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 150

FACILITY OPERATING LICENSE NO. DPR-68

DOCKET NO. 50-296

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages* are provided to maintain document completeness.

<u>REMOVE</u>	<u>INSERT</u>
iii	iii
iv	iv*
3.9/4.9-13	3.9/4.9-13*
3.9/4.9-14	3.9/4.9-14
--	3.9/4.9-14a
--	3.9/4.9-14b
3.9/4.9-18	3.9/4.9-18*
3.9/4.9-19	3.9/4.9-19
3.9/4.9-20	3.9/4.9-20
3.9/4.9-21	3.9/4.9-21*

<u>Section</u>	<u>Page No.</u>
F. Recirculation Pump Operation	3.6/4.6-12
G. Structural Integrity	3.6/4.6-13
H. Snubbers	3.6/4.6-15
3.7/4.7 Containment Systems	3.7/4.7-1
A. Primary Containment.	3.7/4.7-1
B. Standby Gas Treatment System	3.7/4.7-13
C. Secondary Containment.	3.7/4.7-16
D. Primary Containment Isolation Valves	3.7/4.7-17
E. Control Room Emergency Ventilation	3.7/4.7-19
F. Primary Containment Purge System	3.7/4.7-21
G. Containment Atmosphere Dilution System (CAD)	3.7/4.7-22
H. Containment Atmosphere Monitoring (CAM) System H ₂ Analyzer	3.7/4.7-23
3.8/4.8 Radioactive Materials	3.8/4.8-1
A. Liquid Effluents	3.8/4.8-1
B. Airborne Effluents	3.8/4.8-3
C. Radioactive Effluents - Dose	3.8/4.8-6
D. Mechanical Vacuum Pump	3.8/4.8-6
E. Miscellaneous Radioactive Materials Sources	3.8/4.8-7
F. Solid Radwaste	3.8/4.8-9
3.9/4.9 Auxiliary Electrical System	3.9/4.9-1
A. Auxiliary Electrical Equipment	3.9/4.9-1
B. Operation with Inoperable Equipment.	3.9/4.9-8
C. Operation in Cold Shutdown Condition	3.9/4.9-14
D. Unit 3 Diesel Generators Required for Unit 2 Operation	3.9/4.9-14a

<u>Section</u>	<u>Page No.</u>
3.10/4.10 Core Alterations	3.10/4.10-1
A. Refueling Interlocks	3.10/4.10-1
B. Core Monitoring	3.10/4.10-4
C. Spent Fuel Pool Water	3.10/4.10-7
D. Reactor Building Crane	3.10/4.10-8
E. Spent Fuel Cask	3.10/4.10-9
F. Spent Fuel Cask Handling-Refueling Floor.	3.10/4.10-9
3.11/4.11 Fire Protection Systems	3.11/4.11-1
A. Fire Detection Instrumentation	3.11/4.11-1
B. Fire Pumps and Water Distribution Mains	3.11/4.11-2
C. Spray and/or Sprinkler Systems	3.11/4.11-7
D. CO ₂ System	3.11/4.11-8
E. Fire Hose Stations.	3.11/4.11-9
F. Yard Fire Hydrants and Hose Houses	3.11/4.11-11
G. Fire-Rated Assemblies	3.11/4.11-12
H. Open Flames, Welding and Burning in the Cable Spreading Room.	3.11/4.11-13
5.0 Major Design Features	5.0-1
5.1 Site Features	5.0-1
5.2 Reactor	5.0-1
5.3 Reactor Vessel	5.0-1
5.4 Containment	5.0-1
5.5 Fuel Storage	5.0-1
5.6 Seismic Design	5.0-2

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.B Operation With Inoperable Equipment

10. When one 480-V shutdown board is found to be inoperable, the reactor will be placed in HOT STANDBY CONDITION within 12 hours and COLD SHUTDOWN CONDITION within 24 hours.
11. If one 480-V RMOV board mg set is inoperable, REACTOR POWER OPERATION may continue for a period not to exceed seven days, provided the remaining 480-V RMOV board mg sets and their associated loads remain OPERABLE.
12. If any two 480-V RMOV board mg sets become inoperable, the reactor shall be placed in the COLD SHUTDOWN CONDITION within 24 hours.
13. If the requirements for operation in the conditions specified by 3.9.B.1 through 3.9.B.12 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the COLD SHUTDOWN CONDITION within 24 hours.

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.C. Operation in COLD SHUTDOWN
CONDITION

Whenever the reactor is in the COLD SHUTDOWN CONDITION with irradiated fuel in the reactor, the availability of electric power shall be as specified in Section 3.9.A except as specified herein.

1. At least two Unit 3 diesel generators and their associated 4-kV shutdown boards shall be OPERABLE.
2. An additional source of power energized and capable of supplying power to the Unit 3 shutdown boards consisting of at least one of the following:
 - a. One of the offsite power sources specified in 3.9.A.1.c.
 - b. A third OPERABLE diesel generator.
3. At least one Unit 3 480-V shutdown board must be OPERABLE.
4. One 480-V RMOV board motor generator (mg) set is required for each RMOV board (3D or 3E) required to support operation of the RHR system in accordance with 3.5.B.9.

4.9.C Operation in COLD SHUTDOWN
CONDITION

1. No additional surveillance is required.

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.D Unit 3 Diesel Generators
Required for Unit 2 Operation

1. Whenever any of the following equipment is required to be OPERABLE in accordance with the corresponding section of any units technical specifications, the Unit 3 diesel generator aligned to supply emergency power to that equipment shall be OPERABLE.
 - a. Standby gas treatment train C in accordance with T.S. 3.7.B (diesel generator 3D).
 - b. Control room emergency ventilation train B in accordance with T.S. 3.7.E (diesel generator 3B or 3C).
2. When the diesel generator aligned to supply emergency power to the equipment in 3.9.D.1.a or b is inoperable, the equipment may be considered OPERABLE for the purpose of satisfying the corresponding technical specification during the succeeding 30 days provided that the redundant train(s) of equipment and their normal and emergency power supplies are OPERABLE.
3. If Specification 3.9.D.2 cannot be met, the affected equipment shall be declared inoperable.

4.9.D Unit 3 Diesel Generators
Required for Unit 2 Operation

1.a Diesel Generators

Surveillance requirements are as specified in T.S. 4.9.A.1.a, 4.9.A.1.c, 4.9.A.1.d and 4.9.A.1.e.

1.b DC Power System

Surveillance requirements are as specified in T.S. 4.9.A.2.

1.c Logic Systems

Both divisions of the common accident signal logic system shall be tested every 6 months to demonstrate that it will function on actuation of the core spray system of the Unit 2 reactor to provide an automatic start signal to each diesel generator.

1.d Undervoltage Relays

Surveillance requirements are as specified in T.S. 4.9.A.4.

2. No surveillance required.

3. No surveillance required.

THIS PAGE INTENTIONALLY LEFT BLANK

AUG 19 1988

The objective of this specification is to assure an adequate source of electrical power to operate facilities to cool the unit during shutdown and to operate the engineered safeguards following an accident. There are three sources of alternating current electrical energy available, namely, the 161-kV transmission system, the 500-kV transmission system, and the diesel generators.

The unit station-service transformer B for unit 3 provides a noninterruptible source of offsite power from the 500-kV transmission system to the unit 3 shutdown boards. Auxiliary power can also be supplied from the 161-kV transmission system through the common station-service transformers or through the cooling tower transformers by way of the bus tie board. The 4-kV bus tie board may remain out of service indefinitely provided one of the required offsite power sources is not supplied from the 161-kV system through the bus tie board.

The minimum fuel oil requirement of 103,300 gallons is sufficient for seven days of full load operation of three diesels and is conservatively based on availability of a replenishment supply.

The degraded voltage sensing relays provide a start signal to the diesel generators in the event that a deteriorated voltage condition exists on a 4-kV shutdown board. This starting signal is independent of the starting signal generated by the complete loss of voltage relays and will continue to function and start the diesel generators on complete loss of voltage should the loss of voltage relays become inoperable. The 15-day inoperable time limit specified when one of the three phase-to-phase degraded voltage relays is inoperable is justified based on the two-out-of-three permissive logic scheme provided with these relays.

A 4-kV shutdown board is allowed to be out of operation for a brief period to allow for maintenance and testing, provided all remaining 4-kV shutdown boards and associated diesel generators, CS, RHR, (LPCI and containment cooling) systems supplied by the remaining 4-kV shutdown boards, and all emergency 480-V power boards are operable.

The 480-V diesel auxiliary board may be out of service for short periods for tests and maintenance. There are five 250-V dc battery systems associated with unit 3, each of which consists of a battery, battery charger, and distribution equipment. Three of these systems provide power for unit control functions, operative power for unit motor loads, and alternative drive power for a 115-V ac unit-preferred mg set. One 250-V dc system provides power for common plant and transmission system control functions, drive power for a 115-V ac plant-preferred mg set, and emergency drive power for certain unit large motor loads. The fifth battery system delivers control power to a 4-kV shutdown board.

3.5 BASES (Cont'd)

The 250-V dc system is so arranged and the batteries sized so that the loss of any one unit battery will not prevent the safe shutdown and cooldown of all three units in the event of the loss of offsite power and a design basis accident in any one unit. Loss of control power to any engineered safeguard control circuits is annunciated in the main control room of the unit affected.

The station battery supplies loads that are not essential for safe shutdown and cooldown of the nuclear system. This battery was not considered in the accident load calculations.

There are two 480-V ac RMOV boards that contain mg sets in their feeder lines. These 480-V ac RMOV boards have an automatic transfer from their normal to alternate power source (480-V ac shutdown boards). The mg sets act as electrical isolators to prevent a fault from propagating between electrical divisions due to an automatic transfer. The 480-V ac RMOV boards involved provide motive power to valves associated with the LPCI mode of the RHR system. Having an mg set out of service reduces the assurance that full RHR (LPCI) capacity will be available when required. Since sufficient equipment is available to maintain the minimum complement required for RHR (LPCI) operation, a 7-day servicing period is justified. Having two mg sets out of service can considerably reduce equipment availability; therefore, the affected unit shall be placed in Cold Shutdown within 24 hours.

The offsite power source requirements are based on the capacity of the respective lines. The Trinity line is limited to supplying two operating units because of the load limitations of CSST's A and B. The Athens line is limited to supplying one operating unit because of the load limitations of the Athens line. The limiting conditions are intended to prevent the 161-kV system from supplying more than two units in the event of a single failure in the offsite power system.

Specification 3.9.D provides the operability requirements for the Unit 3 diesel generators when they serve as emergency power supplies to standby gas treatment train C and control room emergency ventilation train B when they are being considered operable for Unit 2 technical specifications. The allowable out of service time of 30 days is commensurate with the importance of the affected systems when Unit 3 is in cold shutdown, the low probability of a LOCA/Loss of offsite power and availability of onsite power to redundant trains.

4.9 BASES

The monthly tests of the diesel generators are primarily to check for failures and deterioration in the system since last use. The diesels will be loaded to at least 75 percent of rated power while engine and generator temperatures are stabilized (about one hour). The minimum 75-percent load will prevent soot formation in the cylinders and injection nozzles. Operation up to an equilibrium temperature ensures that there is no overheating problem. The tests also provide an engine and generator operating history to be compared with subsequent engine-generator test data to identify and to correct any mechanical or electrical deficiency before it can result in a system failure.

The test during refueling outages is more comprehensive, including procedures that are most effectively conducted at that time. These include automatic actuation and functional capability tests to verify that the generators can start and be ready to assume load in 10 seconds. The annual inspection will detect any signs of wear long before failure.

Battery maintenance with regard to the floating charge, equalizing charge, and electrolyte level will be based on the manufacturer's instruction and sound maintenance practices. In addition, written records will be maintained of the battery performance. The plant batteries will deteriorate with time but precipitous failure is unlikely. The type of surveillance called for in this specification is that which has been demonstrated through experience to provide an indication of a cell becoming irregular or unserviceable long before it becomes a failure.

The equalizing charge, as recommended by the manufacturer, is vital to maintaining the ampere-hour capacity of the battery, and will be applied as recommended.

The testing of the logic system will verify the ability of the logic systems to bring the auxiliary electrical system to running standby readiness with the presence of an accident signal from any reactor or an undervoltage signal on the start buses or 4-kV shutdown boards.

The periodic simulation of accident signals in conjunction with diesel-generator voltage available signals will confirm the ability of the 480-V load shedding logic system to sequentially shed and restart 480-V loads if an accident signal were present and diesel-generator voltage were the only source of electrical power.

Specification 4.9.D provides surveillance requirements for Unit 3 diesel generators for the purpose of satisfying Specification 3.9.D. It contains less stringent testing requirements for the Unit 3 diesel generators when they are only being used to support Unit 2 equipment.

4.9 BASES (Cont'd)

References

1. Normal Auxiliary Power System (BFNP FSAR Subsection 8.4)
2. Standby AC Power Supply and Distribution (BFNP FSAR Subsection 8.5)
3. 250-Volt DC Power Supply and Distribution (BFNP FSAR Subsection 8.6)
4. Memorandum from G. M. Wilhoite to H. J. Green dated December 4, 1981 (LOO 811208 664) and memorandum from C. E. Winn to H. J. Green dated January 10, 1983 (G02 830112 002)