	CONTROL BLOCK:                       (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1	P A 3 7 S 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 6 1 5 CAT 58
GON'T 0 1 8	REPORT L 6 0 5 0 0 0 3 3 4 7 0 8 2 3 8 1 3 1 1 1 1 8 8 2 9  EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10  1 On 8/23/81 at 0145 hours with reactor power at 99%, a surveillance test
0 2	
0 3	conducted to check for leakage through the containment airlock door seals
0 4	revealed leakage past the inner door seals. The airlock was declared in-
0 5	operable. Power operations continued as permitted by the action statement
0 6	of Technical Specification 3.6.1.3. There were no safety implications
0 7	as the outer airlock door tested satisfactorily.
0 8	
7 8 0 9 7 3	SYSTEM CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCOD
10	TAKEN ACTION FUTURE EFFECT SHUTDOWN METHOD HOURS 22 ATTACHMENT FORM SUB. SUPPLIER COMPONENT MANUFACTURER SUPPLIER SUPPLI
	regreased and the surveillance test was successfully completed at 0505
	hours on 8/23/81. Because of the frequent problems with the airlock,
U 121	
	a design change procedure, DCP 322 "Miscellancous Airlock Modifications"
7 8	was approved and completed on 6/5/82 during the second refueling outage.
1 5	STATUS SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32    E 28   0   9   9   29   N/A   B 31   Scheduled Surveillance Test   80
	CTIVITY CONTENT ELEASED OF RELEASE  AMOUNT OF ACTIVITY (35)  N/A  SO  AND  N/A  N/A
1 7	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 0 0 0 37 Z 38 N/A
1 3	PERSONNEL INJURIES NUMBER DESCRIPTION 41 0 0 0 40 N/A
1 9	LOSS OF OR DAMAGE TO FACILITY 43  TYPE DESCRIPTION N/A 8212080440 821126 PDR ADOCK 05000334 PDR ADOCK 05000334 PDR ADOCK 05000334
20	SSUED DESCRIPTION 45  NRC USE ONLY

Attachment To LER 81-078/03X-1
Beaver Valley Power Station
Duquesne Light Company
Docket No. 50-334

On 8/23/81, at 0145 hours, a surveillance test conducted to test for leakage through the containment airlock door seals, revealed leakage past the inner door seals. The inner door was then declared inoperable. Power operations continued as permitted by the action statement of Technical Specification 3.6.1.3. There were no safety implications as the outer airlock door tested satisfactorily.

At approximately 0400 hours, a containment entry was made to inspect the inner door 0-ring. During the entry, the inner door 0-rings, the door mating surfaces, and the locking ring mating surfaces were wiped and regreased. The inner door was then retested at 0505 hours and declared operable.

In order to preclude future problems with the door seal leakage, a design change procedure, DCP 322, "Miscellaneous Airlock Modification" was initiated on 11/5/81. This design change, completed on 6/5/82, modified the O-ring grooves to provide positive retention of the O-rings. This modification consisted of welding a 3/32 inch diameter stainless steel wire around the outer circumference of each of the four main door O-ring grooves. As a result of the addition of the wire, a smaller diameter (½ inch) O-ring than before (9/16 inch diameter) is required for proper operation of the seals. Also, to compensate for the reduction in diameter of the O-rings, the thickness of the locking wedges was increased.

Upon the resumption of normal plant operations following the refueling outage, no additional problems with 0-ring seal leakage have been experienced.