(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | | | L | L | S | C | 1 (2) | 0 0 - 0 0 0 0 - 0 0 (3) | 4 1 0 0 0 (4) | (5)

7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE JO 57 CAT 58

CON'T

0 1 | REPORT SOURCE | L (6) | 0 5 0 0 0 3 7 3 (7) | 1 1 0 2 8 2 (8) | 1 2 0 1 8 2 (9)

7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On November 2, 1982 at 0100 hrs. during the performance of LOS-RI-Q1 the RCIC

0 3 | Inboard Testable Check Valve 1E51-F066 failed to indicate closed following a close

0 4 | signal from the control room. At 0130 hrs. on 11/2/82 the RCIC Outboard Testable

0 5 | Check Valve and Equalizing Valve, 1E51-F065 and 1E51-F354, were deactivated in the

0 6 | closed position pursuant to Tech. Spec. 3.6.3. The RCIC System was placed in the

0 7 | degraded equipment log.

0 8 |

7 8 9 80

0 9 | SYSTEM CODE: S D (11) | CAUSE CODE: E (12) | CAUSE SUBCODE: B (13) | COMPONENT CODE: V A L U 0 P (14) | COMP. SUBCODE: J (15) | VALVE SUBCODE: A (16)

7 8 9 10 11 12 13 18 19 20

(17) LER/RO REPORT NUMBER: EVENT YEAR 8 2 | — | SEQUENTIAL REPORT NO. 1 4 1 | / | OCCURRENCE CODE 0 3 | REPORT TYPE L | — | REVISION NO. 0

21 22 23 24 26 27 28 29 30 31 32

ACTION TAKEN: Z (18) | FUTURE ACTION: X (19) | EFFECT ON PLANT: Z (20) | SHUTDOWN METHOD: Z (21) | HOURS (22): 0 0 0 0 | ATTACHMENT SUBMITTED: Y (23) | NPRD-4 FORM SUB.: N (24) | PRIME COMP. SUPPLIER: A (25) | COMPONENT MANUFACTURER: A 3 9 2

33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | During the Unit 1 Startup Phase the station has experienced reoccurring problems

1 1 | with RCIC Testable Check Valve closed indication when at rated reactor pressure:

1 2 | ref. LER #82-077 and LER #82-097. Causes are under investigation with the Anchor

1 3 | Darling Valve Co. Alternative limit switch configurations are being evaluated for

1 4 | applicability.

7 8 9 80

1 5 | FACILITY STATUS: B (28) | % POWER: 0 1 8 (29) | OTHER STATUS (30): NA | METHOD OF DISCOVERY: B (31) | DISCOVERY DESCRIPTION (32): LOS-RI-Q1

7 8 9 10 12 13 44 45 46 80

1 6 | ACTIVITY RELEASED: Z (33) | CONTENT OF RELEASE: Z (34) | AMOUNT OF ACTIVITY (35): NA | LOCATION OF RELEASE (36): NA

7 8 9 10 11 44 45 80

1 7 | PERSONNEL EXPOSURES: NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39): NA

7 8 9 11 12 13 80

1 8 | PERSONNEL INJURIES: NUMBER 0 0 0 (40) | DESCRIPTION (41): NA

7 8 9 11 12 80

1 9 | LOSS OF OR DAMAGE TO FACILITY: TYPE Z (42) | DESCRIPTION (43): NA

7 8 9 10 80

2 0 | PUBLICITY: ISSUED N (44) | DESCRIPTION (45): NA

7 8 9 10 68 69 80

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NRC USE ONLY

NAME OF PREPARER: R. W. Houston PHONE: (815) 357-6761 ext. 323

I. LER NUMBER: 82-141/03L-0

II. LASALLE COUNTY STATION: Unit 1

III. DOCKET NUMBER: 050-373

IV. EVENT DESCRIPTION:

On November 2, 1982 at 0100 hours during the performance of LOS-RI-Q1, "RCIC System Pump Operability and Valve Inservice Tests in Conditions 1, 2, and 3," the RCIC Inboard Testable Check Valve 1E51-F066 failed to indicate closed following a close signal from the control room.

V. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

At the time of the occurrence the reactor was in the RUN mode producing 780 MWT, 193 MWE. At 0130 hours on 11/2/82 the RCIC Outboard Testable Check Valve and Equalizing Valve, 1E51-F065 and 1E51-F354, were deactivated in the closed position pursuant to Technical Specificaion 3.6.3. The RCIC System was placed in the degraded equipment log. Safe plant operation was maintained at all times.

VI. CAUSE:

During the Unit 1 Startup Phase the station has experienced reoccurring problems with RCIC Testable Check Valve closed indication when at rated reactor pressure: ref. LER #82-077 and LER #82-097. These valves, manufactured by Anchor Darling Valve Co., operate properly at reduced reactor pressure. Causes for improper indication at rated reactor pressure are currently under discussion with Anchor Darling representatives. Most probable causes of the indication problem are position cam slippage or stiff limit switch linkage that prevents tight valve closure unless there is a differential pressure across the valve disk.

VII. CORRECTIVE ACTION:

Following a reactor SCRAM on 11/9/82 a drywell entry was made to observe the 1E51-F066 valve and correct any problems associated with the indication. At this reduced reactor pressure the valve was observed to operate correctly, therefore, no work was performed.

Alternative limit switch configurations are being evaluated for applicability to the RCIC Testable Check Valves.

Prepared by: R. Houston