



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee) dated October 11, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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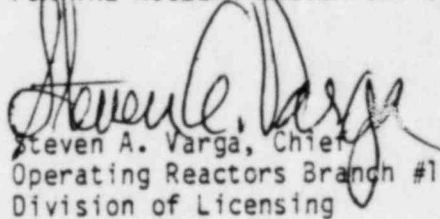
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 20, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 24, 1982

ATTACHMENT TO LICENSE AMENDMENT
AMENDMENT NO. 20 TO FACILITY LICENSE NO. NPF-8
DOCKET NO. 50-364

Revise Appendix A as follows:

Remove Pages

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Insert Pages

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REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.7.2 Reactor Coolant System leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 GPM UNIDENTIFIED LEAKAGE,
- c. 1 GPM total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,
- d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 31 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2235 ± 20 psig.
- *f. 1 GPM leakage from any Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1 at a Reactor Coolant System pressure of 2235 ± 20 psig.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.7.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by;

- a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.
- b. Monitoring the containment air cooler condensate level system or containment atmosphere gaseous radioactivity monitor at least once per 12 hours.

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.7.2 Reactor Coolant System leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 GPM UNIDENTIFIED LEAKAGE,
- c. 1 GPM total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,
- d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 31 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2235 ± 20 psig.
- *f. The maximum allowable leakage of any Reactor Coolant System Pressure Isolation Valve shall be as specified in Table 3.4-1a at a pressure of 2235 ± 20 psig.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and cold shutdown within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.7.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by;

- a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.
- b. Monitoring the containment air cooler condensate level system or containment atmosphere gaseous radioactivity monitor at least once per 12 hours.

*These leakage rates apply only to startup tests following the first refueling outage. Allowable leakage rates for this one time exception are contained in Table 3.4-1a.

TABLE 3.4-1a
REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

T P NS	DESCRIPTION	PRE-OP DATA ADJUSTED LEAKAGE	ALLOWABLE LEAKAGE FOR 1ST REFUELING RESTART ADJUSTED TO 2235±20 PSIG
QV062A,B&C	2" check	0.303 GPM	1.65 GPM
QV051A	6" check	0.00 GPM	2.5 GPM
Q2E11V066A	2" check	0.00 GPM	1.5 GPM
Q2E11V021A	6" check	0.00 GPM	2.5 GPM
Q2E11V042B	10" check	0.00 GPM	2.5 GPM
Q2E11V051B	6" check	0.00 GPM	2.5 GPM
Q2E11V066B	2" check	0.32 GPM	1.66 GPM
Q2E11V021B	6" check	0.00 GPM	2.5 GPM
Q2E11V051C	6" check	0.00 GPM	2.5 GPM
Q2E11V021C	6" check	0.00 GPM	2.5 GPM
Q2E11V066C	2" check	0.91 GPM	1.955 GPM
Q2E11V042A	10" check	0.00 GPM	2.5 GPM
Q2E21V077A	6" check	0.00 GPM	2.5 GPM
Q2E21V077B	6" check	0.00 GPM	2.5 GPM
Q2E21V077C	6" check	0.00 GPM	2.5 GPM
Q2E21V078A	2" check	0.00 GPM	1.5 GPM
Q2E21V079A	2" check	0.00 GPM	1.5 GPM
Q2E21V076A	6" check	0.00 GPM	2.5 GPM
Q2E21V078B	2" check	0.45 GPM	1.725 GPM
Q2E21V079B	2" check	0.45 GPM	1.725 GPM
Q2E21V076B	6" check	0.00 GPM	2.5 GPM
Q2E21V078C	2" check	0.69 GPM	1.845 GPM
Q2E21V079C	2" check	0.76 GPM	1.88 GPM
Q2E11V016A	12" GATE	0.00 GPM	2.5 GPM
Q2E11V001A	12" GATE	0.00 GPM	2.5 GPM
Q2E11V016B	12" GATE	0.00 GPM	2.5 GPM
Q2D11V001B	12" GATE	0.00 GPM	2.5 GPM
Q2E21V032A	12" check	0.00 GPM	2.5 GPM
Q2E21V032B	12" check	0.00 GPM	2.5 GPM
Q2E21V032C	12" check	0.00 GPM	2.5 GPM
Q2E21V037A	12" check	0.00 GPM	2.5 GPM
Q2E21V037B	12" check	0.00 GPM	2.5 GPM
Q2E21V037C	12" check	0.00 GPM	2.5 GPM