

TROJAN NUCLEAR PLANT
CYCLE 5 STARTUP AND POWER ESCALATION PHYSICS TESTING REPORT

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8212070319 821130
PDR ADOCK 05000344
P PDR

Portland General Electric Company
121 S.W. Salmon Street
Portland, Oregon 97204

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CYCLE 5 STARTUP AND POWER
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1.0 ABSTRACT

Startup physics tests were performed at the beginning of the fifth fuel cycle at the Trojan Nuclear Power Plant in order to verify that the basic static and kinetic characteristics of the core were as expected and that values used in the safety analysis were conservative.

Due to fuel failure resulting from reactor vessel internals baffle-gap water-jetting (see LTR 82-06), a total of 22 fuel assemblies in the Cycle 5 core were modified to contain either three or five stainless steel pins in place of fuel rods (see License Amendment 75, July 29, 1982). In addition, eight of those fuel assemblies also have 2 x 8 partial grids at the seven midspans on the baffle-jet side of the assembly to reduce the likelihood of jetting fuel failure.

The effect on power distribution of the fuel assembly modification was expected to be small. Startup physics tests confirmed the validity of this nuclear design prediction.

This startup report is being submitted in order to fulfill the requirements of Amendment 75 of Facility Operating License No. NPF-1. The subject matter of this report is limited to the performance of the reactor core.

2.0 INTRODUCTION

The Trojan Nuclear Power Plant, located in northwest Oregon on the Columbia River, is a high-power-density 4-loop Westinghouse pressurized water reactor rated at 3423 Mwt gross using 17 x 17 array nuclear fuel.

The Trojan Nuclear Power Plant completed its fourth cycle of operation on March 26, 1982. The fourth cycle core contained two assemblies that were modified to include three stainless steel pins in response to a baffle-gap water-jet impingement problem (see Topical Report PGE-1029, "Cycle 4 Startup and Power Escalation Testing Report", for additional details). Within 3 months after the commencement of Cycle 4, evidence of fuel failure was observed.

During a fuel inspection program at Trojan at the end of Cycle 4, 17 assemblies were found to have degraded fuel cladding. The inspection was planned to locate fuel assemblies that were believed to be leaking, since dose-equivalent iodine coolant activity had reached 80 to 85 percent of the Technical Specification limit near the end-of-cycle prior to the shutdown for refueling. Severe damage to eight fuel assemblies was found by visual inspection; portions of rodlets were missing and loose fuel pellets were found. After fuel sipping, nine additional assemblies had indications of failure.

The apparent cause of most fuel damage was water-jet-induced vibration of fuel rods in fuel assemblies that were adjacent to baffle plate joint locations with enlarged gaps (see Figure 2-1). The apparent degradation of the balance of the assemblies was due to minor clad defects or fuel pellet contamination from nearby grossly failed fuel bundles which was detected by the sipping technique. Two types of baffle-gap-related failures were present. Type 1 is the outside corner or center injection jetting failure. In this case, the water-jet impinges directly on the third rod from the corner and causes it to fail in the lower axial regions from direct water impingement combined with induced rod whirling/vibration.

Type 2 of baffle-gap-related failure is the inside corner or corner injection jetting failure, whereby a jet of water flows parallel to the fuel bundle perimeter face between the fuel and the baffle plate. The flow causes fuel rod whirling to occur at the first few rod locations and leads to severe rod failure in the upper axial regions.

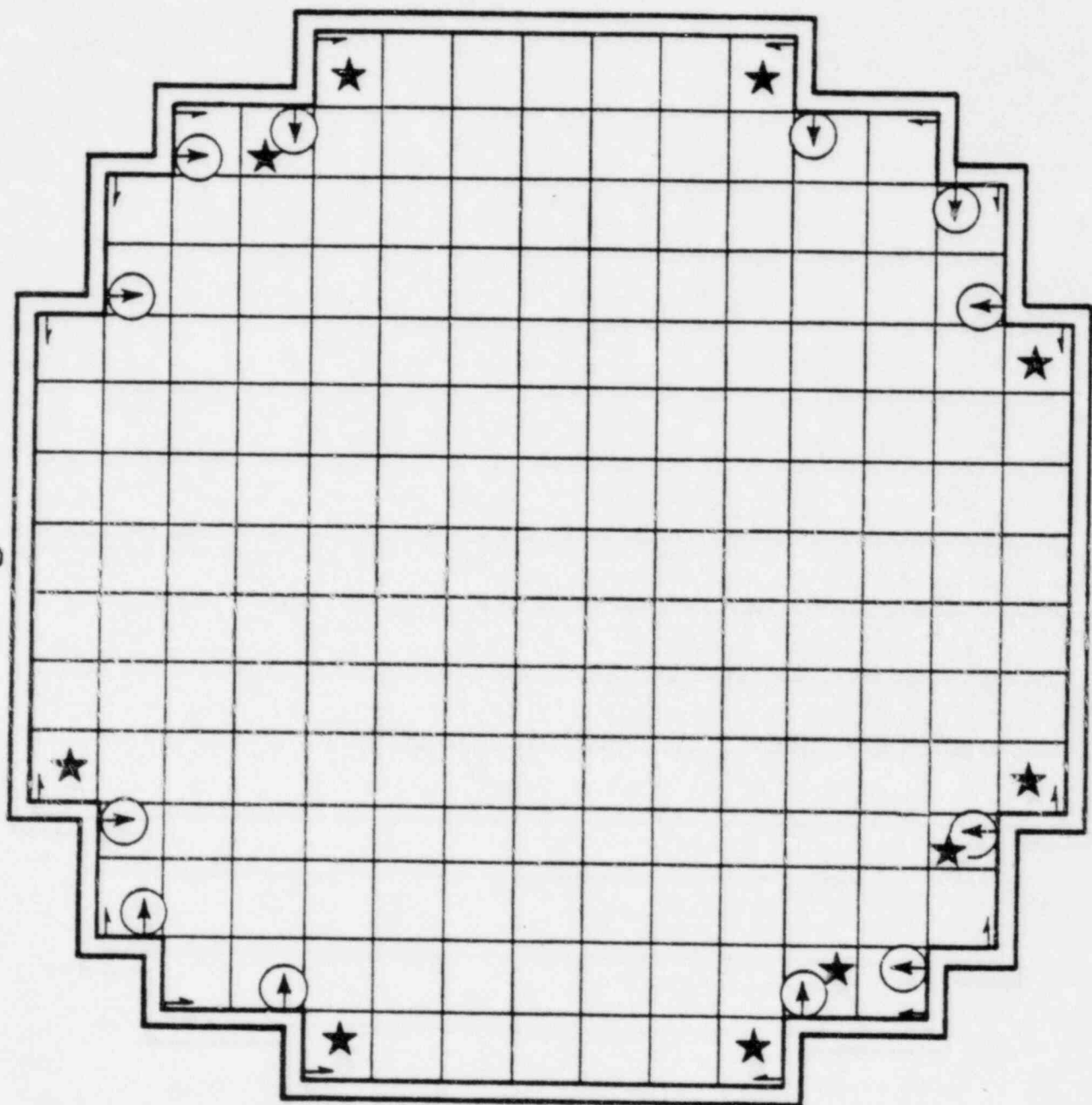
Abnormal degradation had been previously discovered at the end of Cycle 2 in two fuel rods at Trojan. The apparent cause of this fuel damage was Type 1 water-jet impingement on the fuel rod via an enlarged baffle plate joint gap. Two fuel assemblies were modified with stainless steel pins and were located adjacent to the baffle gaps during Cycle 3. Corrective peening action followed at the end of Cycle 3 to close the outside corner baffle gaps, and the previously modified fuel assemblies were relocated for Cycle 4. It appears that the Type 2 jetting problem with the inside corner joints that occurred during Cycle 4 was caused by the peening operation that was performed at the end of Cycle 3. Although additional assemblies failed during Cycle 4 by the Type 1 mechanism, they appeared to have sustained less damage than those failing during Cycle 2 before the peening operation. No failures were observed in the two previously modified fuel assemblies with stainless steel pins.

Trojan conducted an augmented fuel inspection program at the end of Cycle 4 in which all fuel assemblies to be used in the subsequent cycle were leak checked by fuel sipping and/or visually inspected to be damage free (see Attachment A). Accessible loose pellets and debris were retrieved from the reactor vessel internals and refueling cavity. Damaged assemblies adjacent to the baffle were replaced with new modified fuel assemblies using three to five stainless steel pins in place of fuel rods in order to ensure fuel integrity. In addition, 2 x 8 partial grids were inserted at midspans for eight corner injection assemblies as an interim solution to the baffle jetting problem.

The effect of the stainless steel rods and partial grids on the nuclear design of the core was expected to be small. Startup physics tests were performed to confirm the analytical predictions as described in the following sections.

R P N M L K J H G F E D C B A

180



- ★ = Failed assembly locations
- ⊙ = Outside corner, center injection
- ↖ = Inside corner, corner injection

FIGURE 2-1

GRID OF TROJAN CORE

3.0 DISCUSSION

The reload startup physics tests at the beginning of Cycle 5 resulted in good agreement between predicted and measured values as shown in Table 3-1. The results were much improved over those at the start of Cycle 3 and improved over those at the start of Cycle 4.

Only the quadrant power tilt at low power was measured to be outside its acceptance criteria. The magnitudes of Cycle 5 power distribution prediction errors were not excessive and posed no safety concerns. The high power quadrant was observed to be the one predicted to be the high power quadrant. The quadrant power tilt diminished with power ascension as the full power quadrant tilt decreased to one-half percent.

The other nuclear design parameters were well within acceptance criteria and were more accurately predicted for this reload startup than for past reload startups.

PREDICTED AND MEASURED PHYSICS PARAMETERS

<u>Parameter</u>	<u>Trojan Test Acceptance Criteria</u>	<u>Measured</u>	<u>Predicted</u>
<u>Critical Boron (ppm)</u>			
All-rods-out	1340 \pm 50	1369	1340
D bank in	1255 \pm 18	1256	1226
D + C banks in	1165 \pm 15	1160	1135
All-rods-in less one rod	760 \pm 85	755	732
<u>Isothermal Temperature Coefficient (pcm/$^{\circ}$F)</u>			
All-rods-out	-3.3 \pm 3	-3.8	-3.3
D bank in	-6.6 \pm 3	-7.6	-6.6
Moderator temperature coefficient, ARO	< \pm 5.0 \geq -53.6	-1.6	-1.0
<u>Boron Worth (pcm/ppm)</u>			
Differential boron worth, over D	-9.2 \pm 0.9	-9.6	-9.2
Differential boron worth, over C	-9.3 \pm 0.9	-8.4	-9.3
<u>Integral Rod Worth (pcm except as noted)</u>			
Control D	1054 \pm 105	1085	1054
Control C	846 \pm 85	807	846
Control B	1220 \pm 122 -244	1161	1220
Control A	280 \pm 140 -28	307	280
All-rods-in less one rod	5604 \pm 2000 -500	5320	5604
All-rods-in less one rod (ppm)	608 \pm 200 -50	614	608
<u>Doppler Coefficient (pcm/% Power)</u>			
At 30%	-17.5 to -9.5	-12.6	-10.6
At 45%	-16.0 to -8.4	-9.9	-9.9
At 69%	-13.8 to -7.7	-7.7	-9.1
At 97%	-12.8 to -6.7	-9.3	-8.2

Parameter	Trojan Test Acceptance Criteria	Measured	Predicted
<u>Low-Power Core Power Distribution, ARO</u>			
F_Q	$\leq 4.64 * K(z)$	2.48	2.55
$F_{\Delta H}$	≤ 1.523	1.45	1.43
F_{xy}	≤ 1.97	1.60	1.47
Quadrant tilt	≤ 1.02	1.038	1.012
Axial offset	$\leq 48+5\%$ -15%	42%	48%
$F_{\Delta H}$	$\leq 15\%$	9.7%	
<u>Full Power Core Power Distribution, ARO</u>			
$F_{\Delta H}$	≤ 1.55	1.424	1.435
Quadrant tilt	≤ 1.02	1.005	~1.003
F_Q	$\leq 2.32 * K(z)$	1.725	~1.686

4.0 LOW POWER PHYSICS TESTING

The purpose of the low power physics testing program is to assure Plant management that the reactor core is operating properly within design limits and safety assumptions, and to redetermine operating parameters related to the reactor system. The measurements of critical boron concentrations, rod worths, temperature coefficients, boron worths, and core power distribution provide sufficient nuclear design parameter confirmations to approve an overall nuclear design and extrapolate or infer all necessary reactor core operating information not explicitly measured.

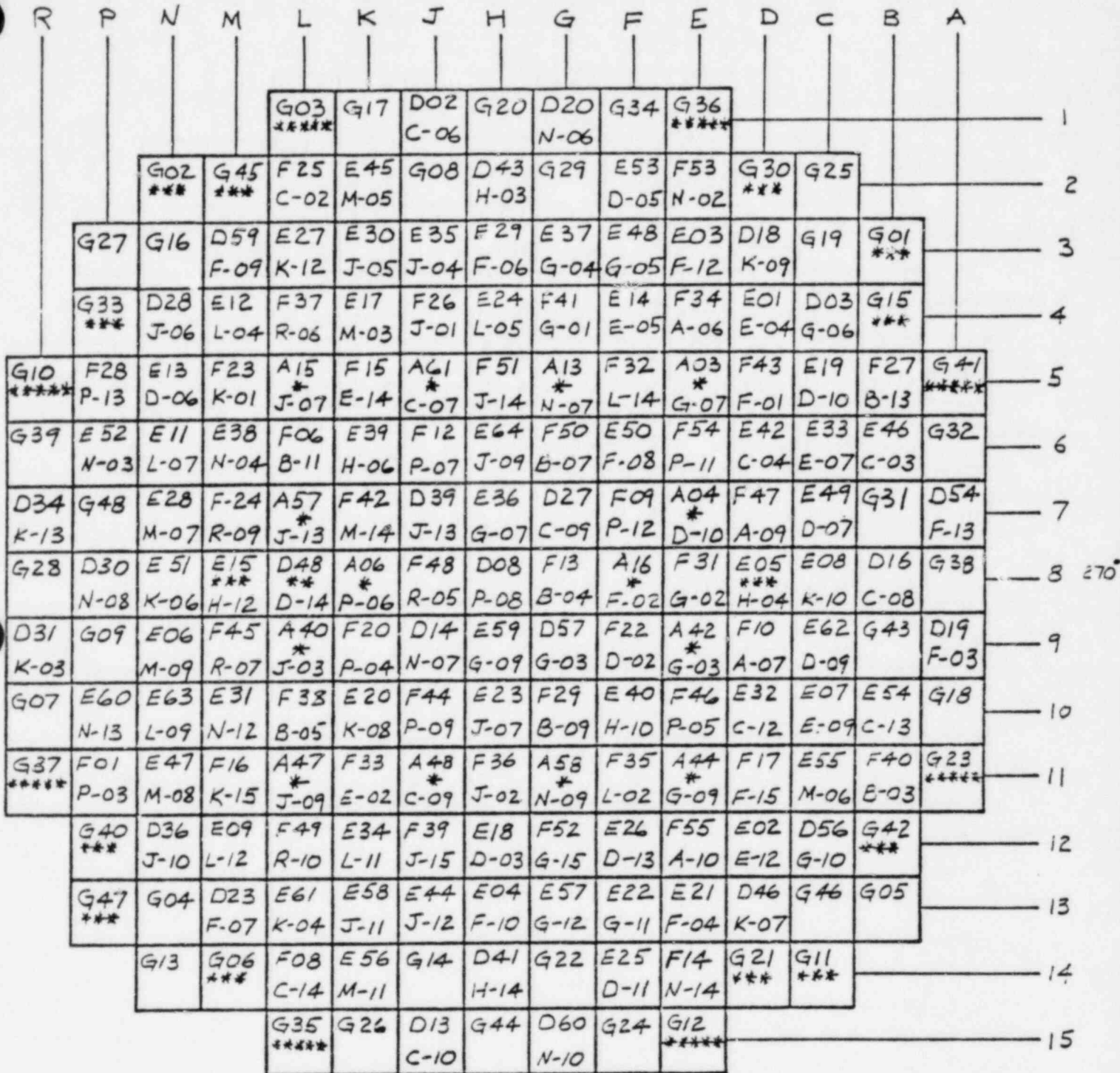
The Trojan Cycle 5 Core Loading Plan is shown in Figure 4-1, and the low power flux map results are presented in Figure 4-2. The Cycle 5 startup physics test results continued the improvement trend begun at the start of Cycle 4. The improved nuclear design methods were responsible for the improvement in predicted vs measured parameters. Most of the improvement was due to a more accurate assessment and handling of individual fuel assembly burnup values.

The low power flux map power depression that occurred at the perimeter of the upper left (90 to 180 degrees) quadrant during the start of Cycles 3 and 4 has disappeared since the quadrant is now loaded as the high power quadrant.

WESTINGHOUSE PROPRIETARY CLASS 2

FIGURE 4-1

Trojan (POR) Cycle 5 Core Loading



- 0°
- | | | | |
|---|--|-----|---|
| A | Region 1 (2.1 w/o)
* From Cycle 1 | F | Region 6 (3.2 w/o) |
| D | Region 4 (3.1 w/o)
* From Cycle 2 | G | Region 7 (3.2 w/o)
*** 3 Stainless Steel Rods
**** 5 Stainless Steel Rods |
| E | Region 5 (3.1 w/o)
*** 3 Stainless Steel Rods | xxx | Assembly Ident. |
| | | yyy | Position in Previous Cycle |

FIGURE 4-2

FLUX MAP 121 (3 PERCENT POWER)

TROJAN FLUX MAP #121 08/23/82 3% BU(0 1 CYS 0#217 CYS 80L -HAIR

CALCULATED POWER TILTS (NORMALIZED TO 1.000)

. 1.0393.0.9902 .
.
1.0371 . . . 0.9576
.
1.0107 . . . 0.9798
.
. 0.9877.0.9977 .

.
1.0382 . 0.9739
.
.
0.9992 . 0.9888
.
.

. 1.0147 . .
.
1.0239 . . 0.9687
.
. 0.9927 . .
.

POSITIVE "Y" VS. NEGATIVE "Y" TILT
1.0060 0.9940

POSITIVE "X" VS. NEGATIVE "X" TILT
0.9813 1.0187

FIGURE 4-2

TROJAN FLUX MAP #121 08/23/82 3% BU(0 1 CYS D#217 CYS BUL -HAIN

THE FOLLOWING CALCULATIONS ARE BASED ON A DIVISION
OF THE CORE INTO *OCTANTS*, THAT IS, QUADRANTS
DIVIDED INTO TWO AXIAL REGIONS OF EQUAL VOLUME.

RELATIVE POWER IN UPPER HALF OF CORE	RELATIVE POWER IN LOWER HALF OF CORE	PERCENT AXIAL OFFSET TOWARD TOP OF CORE	CORE AVERAGE AXIAL OFFSET
(-++) . (++)	(-++) . (++)	(-++) . (++)	
1.4698 . 1.3878	0.6066 . 0.5599	41.571 . 42.507	42.088
.	
1.4209 . 1.4047	0.5775 . 0.5728	42.206 . 42.066	
(--+) . (+--)	(--+) . (+--)	(--+) . (+--)	
POWER TILT IN UPPER HALF OF CORE	POWER TILT IN LOWER HALF OF CORE		
(-++) . (++)	(-++) . (++)	(-++) . (++)	
1.0345 . 0.9768	1.0473 . 0.9667	THESE . EDITS	
.	
1.0001 . 0.9887	0.9970 . 0.9890	ADDED . JAN., 1969	
(--+) . (+--)	(--+) . (+--)	(--+) . (+--)	

FIGURE 4-2

TROJAN FLUX MAP #121 08/23/82 3% BU10 1 CYS D=217 CYS BOL -BAIR

TOP TWENTY NUCLEAR F-DELTA-H

TOP TWENTY NUCLEAR FQ

258M 5MD	FDHN=1.4516	324K 5EN	FQN=2.4788
287L 4DM	FDHN=1.4438	325K 5FN	FQN=2.4777
225N 3LK	FDHN=1.4268	358J 4FE	FQN=2.4760
204P 7MF	FDHN=1.4266	359J 4DE	FQN=2.4693
209P 7LE	FDHN=1.4240	272M11MN	FQN=2.4585
210P 7LD	FDHN=1.4197	513E12NE	FQN=2.4575
226N 3K1	FDHN=1.4193	292L 6NE	FQN=2.4556
352J 2FM	FDHN=1.3992	258M 5MD	FQN=2.4554
272M11MN	FDHN=1.3985	208P 7MF	FQN=2.4497
353J 2EL	FDHN=1.3963	209P 7LE	FQN=2.4451
513E12NE	FDHN=1.3958	287L 4DM	FQN=2.4421
214P 9ML	FDHN=1.3947	210P 7LD	FQN=2.4376
354J 2DL	FDHN=1.3921	444G12NM	FQN=2.4372
215P 9LM	FDHN=1.3903	225N 3LK	FQN=2.4343
216P 9LN	FDHN=1.3851	326K 5ED	FQN=2.4327
324K 5EN	FDHN=1.3834	381J14FE	FQN=2.4292
325K 5FN	FDHN=1.3828	268M 9EL	FQN=2.4267
448G14LE	FDHN=1.3766	226N 3K1	FQN=2.4218
358J 4FF	FDHN=1.3748	382J14EF	FQN=2.4212
449G14MF	FDHN=1.3723	383J14DF	FQN=2.4114

NOTE= VALUES ARE BEST ESTIMATE AND DO NOT INCLUDE ENGINEERING OR NUCLEAR UNCERTAINTY.

FIGURE 4-2

AXIAL POINT	LOCA FQ ENVELOPE LIMIT		AXIALLY
	FQ(Z) LIMIT	MEAS. FQ(Z)	SOURCE NO. ID
1	1.9952	1.4010	282L 2NM
2	2.4035	1.4706	272M11MN
3	2.8026	1.7810	272M11MN
4	3.2062	2.0445	513E12NE
5	3.6053	2.2242	272M11MN
6	4.0043	2.2991	513E12NE
7	4.1760	2.2887	358J 4FE
8	4.3802	2.5430	513E12NE
9	4.3894	2.6219	513E12NE
10	4.4034	2.6778	356J 4FE
11	4.4080	2.6808	324K 5EN
12	4.4266	2.6778	358J 4FE
13	4.4358	2.6555	258M 5MD
14	4.4498	2.6325	258M 5MD
15	4.4590	2.4584	225N 3LK
16	4.4730	2.5071	208P 7MF
17	4.4822	2.5945	225N 3LK
18	4.4962	2.5561	225N 3LK
19	4.5101	2.5394	225N 3LK
20	4.5194	2.4995	225N 3LK
21	4.5286	2.5689	252M 2NM
22	4.5426	2.3952	225N 3LK
23	4.5565	2.3191	225N 3LK
24	4.5658	2.0988	352J 2FM
25	4.5797	2.1949	282L 2NM
26	4.5890	2.1295	225N 3LK
27	4.6029	2.0555	258M 5MD
28	4.6168	2.0372	225N 3LK
29	4.6261	1.9342	258M 5MD
30	4.6354	1.8978	225N 3LK

FIGURE 4-2

AXIAL POINT	LOCA FU ENVELOPE LIMIT F ₀ (Z)	MEAS. F ₀ (Z)	AXIALLY SOURCE NO. 10
31	4.6400	1.8385	225N 3LK
32	4.6400	1.6718	282L 2NM
33	4.6400	1.7652	282L 2NM
34	4.6400	1.6251	282L 2NM
35	4.6400	1.5179	258M 5MD
36	4.6400	1.4949	225N 3LK
37	4.6400	1.4031	258M 5MD
38	4.6400	1.3437	225N 3LK
39	4.6400	1.3387	252M 2NM
40	4.6400	1.2292	225N 3LK
41	4.6400	1.1955	282L 2NM
42	4.6400	1.1021	282L 2NM
43	4.6400	1.0425	258M 5MD
44	4.6400	1.0089	225N 3LK
45	4.6400	0.9620	282L 2NM
46	4.6400	0.9252	252M 2NM
47	4.6400	0.8425	258M 5MD
48	4.6400	0.8096	225N 3LK
49	4.6400	0.7378	282L 2NM
50	4.6400	0.6360	258M 5MD
51	4.6400	0.6767	225N 3LK
52	4.6400	0.6360	258M 5MD
53	4.6400	0.6032	258M 5MD
54	4.6400	0.5737	194R 50H
55	4.6400	0.5137	492E 4NM
56	4.6400	0.4590	258M 5MD
57	4.6400	0.4390	282L 2NM
58	4.6400	0.3526	214P 94L
59	4.6400	0.2802	282L 2NM
60	4.6400	0.2989	282L 2NM

FIGURE 4-2

	H	P	N	M	L	K	J	M	G	F	HU(0	D	C	B	A
1	0.728	0.819	0.505	0.658	0.502	0.818	0.723	0.5	1.9	2.1	4.7	5.7			
2	0.791	1.282	0.955	1.119	1.032	0.969	0.988	0.976	1.013	0.887	1.182	0.721			
3	4.1	4.1	4.1	5.5	2.6	2.6	1.5	-0.5	1.4	3.3	2.1	0.9			
4	1.069	0.943	1.122	1.336	1.118	1.288	1.067	1.249	1.030	1.235	1.019	0.864	0.963		
5	2.7	2.7	2.7	2.4	1.5	2.9	2.0	0.4	-0.1	-1.2	-0.2	-1.7			
6	0.796	1.220	1.073	1.317	0.980	1.306	0.942	1.243	0.927	1.231	0.889	1.205	0.970	1.066	0.656
7	9.7	3.5	0.5	0.5	2.4	2.0	3.1	3.3	0.9	-2.9	-3.7	-4.5	-4.5		
8	0.897	1.084	1.037	1.286	1.193	1.231	1.100	1.185	1.126	1.198	1.020	0.957	0.455	0.739	
9	9.7	4.0	-0.2	2.0	2.8	2.9	4.0	1.6	0.0	-2.2	-3.4	-3.6	-4.5		
10	0.528	1.243	1.017	1.252	0.904	1.243	0.930	0.959	0.886	1.212	0.887	1.209	0.962	1.144	0.477
11	6.7	3.2	0.4	0.0	0.9	2.4	3.6	4.2	1.9	1.8	0.4	-1.6	-1.1	-0.5	0.5
12	0.671	0.810	0.977	1.055	1.125	0.910	1.194	0.816	1.087	0.894	1.172	1.028	0.930	0.754	0.624
13	2.7	2.1	1.4	0.5	-0.1	2.3	4.3	2.9	1.5	1.7	0.4	-1.6	-1.0	-1.1	-1.5
14	0.503	1.216	1.021	1.258	0.899	1.232	0.908	0.933	0.900	1.237	0.892	1.212	0.976	1.171	0.480
15	2.5	2.1	1.4	-0.0	-1.4	0.7	1.7	1.2	2.6	2.9	-1.5	-3.2	-2.1	-0.8	-1.7
16	0.811	1.037	1.040	1.096	1.282	1.150	1.144	1.038	1.148	1.157	1.220	1.040	0.953	1.007	0.791
17	1.5	1.5	1.4	1.0	1.0	-0.4	-4.6	-3.3	-3.9	-0.3	-4.0	-4.3	-7.2	-2.2	-2.2
18	0.722	1.173	1.048	1.296	0.941	1.250	0.892	1.190	0.888	1.269	0.925	1.250	0.985	1.160	0.713
19	1.5	1.5	1.0	1.0	-0.5	-1.0	-4.2	-4.6	-5.7	-1.8	-3.6	-4.0	-7.2	-0.9	-0.9
20	1.018	0.884	1.058	1.263	1.057	1.234	1.044	1.252	1.086	1.294	1.080	0.864	1.002		
21	0.1	-1.3	-1.1	-1.4	-1.5	-3.6	-3.8	-3.8	-1.9	-0.8	-0.8	-0.8	-5.2	-3.1	
22	0.740	1.200	0.886	1.017	1.005	0.971	1.015	1.018	1.037	0.899	1.184	0.715			
23	-0.4	-0.4	-1.2	-2.7	-2.2	-1.3	-1.4	-2.2	-3.2	-2.1	-0.9	-3.1	-5.2		
24	0.744	1.000	1.121	1.004	1.195	0.800	1.199	1.017	1.154	1.022	0.733				
25	-0.3	-1.9	-3.6	-2.8	-0.9	-0.8	-1.8	-3.3	-2.1	-0.9	-3.0				
26	0.687	0.787	0.491	0.655	0.489	0.796	0.712								
27	-4.2	-2.4	-1.4	-1.4	-3.3	-3.3	-2.1								
28															
29															
30															
31															
32															
33															
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92															
93															
94															
95															
96															
97															
98															
99															
100															

MEAS
DIFF

FIGURE 4-2

TROJAN FLUX MAP #121 08/23/82 3% HUCO 1 CY5 DW217 CY5 RUL -BAIR

POINT	SOURCE	NUCLEAR FXV	POINT	SOURCE	NUCLEAR FXV
1	282 L 2NM	2.0251	31	225 N 3LK	1.5652
2	272 M11MN	1.6403	32	282 L 2NM	1.5445
3	272 M11MN	1.5995	33	282 L 2NM	1.7392
4	513 E12NE	1.5787	34	282 L 2NM	1.5843
5	272 M11MN	1.5361	35	258 M 5MD	1.5440
6	513 E12NE	1.5434	36	225 N 3LK	1.5756
7	358 J 4FE	1.4804	37	258 M 5MD	1.5392
8	513 E12NE	1.4797	38	225 N 3LK	1.5335
9	513 E12NE	1.4603	39	252 M 2NM	1.6008
10	358 J 4FE	1.4629	40	225 N 3LK	1.5779
11	324 K 5EN	1.4399	41	282 L 2NM	1.7479
12	358 J 4FE	1.4265	42	282 L 2NM	1.6039
13	258 M 5MD	1.4254	43	258 M 5MD	1.5709
14	258 M 5MD	1.4457	44	225 N 3LK	1.5485
15	225 N 3LK	1.4712	45	282 L 2NM	1.5570
16	208 P 74F	1.4563	46	252 M 2NM	1.5448
17	225 N 3LK	1.4631	47	258 M 5MD	1.5223
18	225 N 3LK	1.4693	48	225 N 3LK	1.5556
19	225 N 3LK	1.4777	49	282 L 2NM	1.5400
20	225 N 3LK	1.4826	50	258 M 5MD	1.4826
21	252 M 2NM	1.5566	51	225 N 3LK	1.5385
22	225 N 3LK	1.4973	52	258 M 5MD	1.5297
23	225 N 3LK	1.5395	53	258 M 5MD	1.5407
24	352 J 2FM	1.5533	54	194 R 50M	1.5628
25	282 L 2NM	1.5288	55	492 F 4NM	1.5102
26	225 N 3LK	1.5169	56	258 M 5MD	1.4444
27	258 M 5MD	1.5165	57	282 L 2NM	1.6263
28	225 N 3LK	1.5359	58	214 P 9ML	1.5807
29	258 M 5MD	1.5075	59	282 L 2NM	1.6045
30	225 N 3LK	1.5463	60	282 L 2NM	2.3045

NOTE = VALUES ARE BEST ESTIMATE AND DO NOT INCLUDE ENGINEERING OR NUCLEAR UNCERTAINTY.

5.0 POWER ESCALATION TESTING

The purpose of the power escalation physics testing program is primarily to monitor core power distribution at elevated power levels and to recalibrate instrumentation settings for important Plant parameters that may vary with core loading, power level, or other Plant changes.

In addition, the nuclear doppler/power coefficients are measured to further confirm the nuclear design.

The following measurements were performed during the power escalation testing:

- (1) Incore thermocouple correction factors were obtained (see Table 5-1).
- (2) Doppler coefficient was measured (see Figure 5-1).
- (3) Delta temperature correlation was realigned (60°F equals 100 percent power, see Figure 5-2).
- (4) Feedwater flow correlation was confirmed linear (see Figure 5-3).
- (5) Incore-excore detector calibration relationship was established for incore and excore axial offsets (see Table 5-2 and Figures 5-4 through 5-15).
- (6) Core power distributions were taken (see Figures 5-16 and 5-17 for Flux Maps 122 and 125). Quadrant tilting decreased with increasing power level and burnup. Hot channel factors limits were met.

TABLE 5-1

INCORE THERMOCOUPLE CORRECTION FACTORS

DATA SHEET 1a
T/C CALCULATIONS
Final results

True RTD
Correct Temperature: 557.4 °F

Hot Junction Box Temperatures: 159.5
(Computer U0058, U0059)

T/C No.	Measured Temp		(Meas-Truth) Difference (°F)		T/C No.	Measured Temp		Difference (°F)	
	Toggle	Computer	Toggle	Computer		Toggle	Computer	Toggle	Computer
(A8) 1	553.7	552.1	-3.7	-5.3	(A9) 34	557.3	555.3	-0.1	-2.1
(B3) 2	555.7	554.6	-1.7	-2.8	(A11) 35	558.0	555.7	+0.6	-1.7
(B10) 3	555.3	554.5	-2.1	-2.9	(B5) 36	557.5	555.6	+0.1	-1.8
(C8) 4	555.7	554.4	-1.7	-3.0	(C3) 37	559.0	556.5	+1.6	-0.9
(C13) 5	555.0	553.6	-2.4	-3.8	(C12) 38	558.5	555.9	+1.1	-1.5
(D3) 6	556.0	554.6	-1.4	-2.8	(D7) 39	556.8	554.8	-0.6	-2.6
(E4) 7	556.3	554.8	-1.1	-2.6	(D9) 40	557.8	555.4	+0.4	-2.0
(E6) 8	553.2	552.3	-4.2	-5.1	(E2) 41	557.5	555.1	+0.1	-2.3
(E8) 9	555.0	554.2	-2.4	-3.2	(E14) 42	554.0	552.0	-3.4	-5.4
(E10) 10	558.2	554.0	-2.2	-3.4	(F5) 43	557.8	556.0	+0.4	-1.4
(F12) 11	554.9	553.8	-2.5	-3.6	(F9) 44	558.5	555.9	+1.1	-1.5
(F15) 12	555.0	554.1	-2.4	-3.3	(F11) 45	554.5	552.3	-2.9	-5.1
(G2) 13	555.8	554.4	-1.6	-3.0	(G4) 46	558.0	555.4	+0.6	-2.0
(G9) 14	555.2	554.4	-2.2	-3.0	(G8) 47	558.3	555.9	+0.9	-1.5
(G11) 15	555.2	554.4	-2.2	-3.0	(H5) 48	557.5	555.6	+0.1	-1.8
(H01) 16	552.8	551.6	-4.6	-5.8	(H9) 49	558.8	556.2	+1.4	-1.2
(H3) 17	553.3	552.2	-4.1	-5.2	(H11) 50	557.3	554.3	-0.1	-3.1
(H8) 18	555.3	554.8	-2.1	-2.6	(H14) 51	557.5	555.8	+0.1	-1.6
(H13) 19	554.8	553.4	-2.6	-4.0	(J7) 52	557.5	555.4	+0.1	-2.0
(J10) 20	555.5	554.3	-1.9	-3.1	(K11) 53	557.5	555.1	+0.1	-2.3
(K3) 21	552.8	549.6	-4.6	-7.8	(K13) 54	557.0	555.0	-0.4	-2.4
(K5) 22	555.3	554.2	-2.1	-3.2	(L2) 55	-	-		
(K15) 23	553.8	552.4	-3.6	-5.0	(L9) 56	557.5	555.8	+0.1	-1.6
(L1) 24	555.5	554.2	-1.9	-3.2	(L14) 57	557.3	555.1	-0.1	-2.3
(L8) 25	555.3	554.2	-2.1	-3.2	(M5) 58	558.3	556.1	+0.9	-1.3
(L12) 26	555.5	553.9	-1.9	-3.5	(M12) 59	-	-		
(M10) 27	557.3	556.4	-0.1	-1.0	(N2) 60	558.0	555.6	+0.6	-1.8
(M13) 28	555.3	553.3	-2.1	-4.1	(N8) 61	559.0	556.4	+1.6	-1.0
(N3) 29	555.8	554.3	-1.6	-3.1	(N13) 62	557.3	555.3	-0.1	-2.1
(N9) 30	555.0	554.1	-2.4	-3.3	(P7) 63	558.6	556.0	+1.2	-1.4
(P5) 31	555.7	554.3	-1.7	-3.1	(R5) 64	559.0	556.3	+1.6	-1.1
(P11) 32	555.8	554.6	-1.6	-2.8	(R10) 65	558.0	555.2	+0.6	-2.2
(R08) 33	555.5	554.0	-1.9	-3.4					

Completed By: Jim Kessler

Date 8/20/82 Time 1800

Engineering Supervisor: Alan D. Cully

Date 4/31/82

TABLE 5-2

INCORE/EXCORE DETECTOR CALIBRATION AND POINTS ANALYSIS SHEET

TROJAN NUCLEAR PLANT
 FIDELITY OF CALIBRATION AND POINTS ANALYSIS SHEET
 GARY HAIR

TROJAN MAPS 122, 123, 124, 508, 1121A, 258, 10, 35810, 758, 1125, 100% HAIR 8/30/82

INPUT DATA

AD	TOP CHANNEL CURRENT				BOTTOM CHANNEL CURRENT				
	41	42	43	44	41	42	43	44	
15.4	115.0	124.0	112.0	124.0	104.0	107.0	100.0	112.0	MEASURED
15.4	326.7	359.1	327.0	358.9	297.3	309.9	292.0	324.1	NORMALIZED
20.0	85.0	91.0	82.0	91.0	74.0	75.0	79.0	79.0	MEASURED
20.0	334.7	360.7	333.9	365.6	291.3	302.3	285.1	317.4	NORMALIZED
2.4	154.0	166.0	152.0	163.0	154.0	159.0	151.0	168.0	MEASURED
2.4	313.0	341.7	310.5	336.3	313.0	327.3	308.5	346.7	NORMALIZED
12.0	160.0	174.0	157.0	169.0	147.0	152.0	144.0	160.0	MEASURED
12.0	326.3	357.1	322.9	350.8	294.7	311.9	296.1	332.2	NORMALIZED
7.4	157.0	170.0	155.0	166.0	150.0	156.0	147.0	165.0	MEASURED
7.4	320.1	344.9	317.7	342.5	305.9	320.1	301.3	340.5	NORMALIZED
2.2	241.0	250.0	227.0	243.0	230.0	239.0	224.0	254.0	MEASURED
2.2	313.7	342.0	311.6	333.9	312.3	327.0	307.4	349.1	NORMALIZED
-1.4	308.0	336.0	304.0	328.0	318.0	333.0	315.0	355.0	MEASURED
-1.4	308.0	336.0	304.0	328.0	318.0	333.0	315.0	355.0	NORMALIZED

FLUX DIFFERENCE AFTER VALUES

AD	TOP CHANNEL				BOTTOM CHANNEL				
	41	42	43	44	41	42	43	44	
10.0	322.0	352.7	320.6	346.3	303.2	316.3	298.4	334.7	NORMALIZED
-10.0	298.5	324.7	294.3	313.5	327.5	344.3	324.7	369.5	NORMALIZED
20.0	335.0	366.7	333.7	365.7	291.0	302.3	285.3	317.3	NORMALIZED
-20.0	289.3	310.7	241.2	276.1	339.7	358.3	337.8	386.9	NORMALIZED
30.0	347.2	380.8	346.4	383.1	274.8	288.2	272.2	299.9	NORMALIZED
-30.0	274.1	299.6	268.0	276.7	351.9	372.3	351.0	404.3	NORMALIZED

TABLE 5-2

BUCHAN MICLFAR PLANT
 DELTA 03 CALIBRATION AND POINTS ANALYSIS SHEET
 GARY HAIN
 BUCHAN MAPS 122,123,124,5081121A,2581C,3581D,758125,1008 HAIN 8/30/82

FACTOR DEFECTOR	LOCDEF AXIAL OFFSET	I-TOPT MICROAMP 100S	I-BOIT MICROAMP 100S	V-TOP VOLTS	V-BOIT VOLTS	DELTA V VOLTS	I TOTAL MICRO A	V/I TOP K OHMS	V/I BOIT K OHMS
CH-41	0.0	310.6	317.4	8.333	8.333	0.0	626.0	0.02683	0.02642
CH-41	13.0	326.5	341.4	8.758	7.915	0.843	626.0	0.02683	0.02642
CH-41	3.0	314.3	317.1	8.431	8.430	0.195	626.0	0.02683	0.02642
CH-41	-3.0	307.0	308.4	8.235	8.430	-0.195	626.0	0.02683	0.02642
CH-41	-45.0	255.8	307.0	6.863	9.781	-2.918	626.0	0.02683	0.02642
CH-41	-50.0	249.7	299.7	6.708	9.942	-3.242	626.0	0.02683	0.02642
CH-41	59.4	383.0	459.6	10.273	6.422	3.852	626.0	0.02683	0.02642
CH-42	0.0	338.7	405.4	8.333	8.333	0.0	669.0	0.02460	0.02523
CH-42	13.0	356.9	428.3	8.781	7.873	0.908	669.0	0.02460	0.02523
CH-42	3.0	342.9	411.5	8.436	8.427	0.210	669.0	0.02460	0.02523
CH-42	-3.0	334.5	401.4	8.230	8.430	-0.210	669.0	0.02460	0.02523
CH-42	-45.0	275.6	340.7	6.781	9.924	-3.143	669.0	0.02460	0.02523
CH-42	-50.0	268.6	322.3	6.609	10.101	-3.493	669.0	0.02460	0.02523
CH-42	59.4	422.0	506.4	10.382	6.232	4.149	669.0	0.02460	0.02523
CH-43	0.0	307.4	368.9	8.333	8.333	0.0	619.0	0.02710	0.02675
CH-43	13.0	324.5	389.4	8.796	7.876	0.919	619.0	0.02710	0.02675
CH-43	3.0	311.4	313.7	8.440	8.428	0.212	619.0	0.02710	0.02675
CH-43	-3.0	303.5	304.2	8.226	8.438	-0.212	619.0	0.02710	0.02675
CH-43	-45.0	248.3	298.0	6.731	9.914	-3.182	619.0	0.02710	0.02675
CH-43	-50.0	241.8	290.1	6.553	10.089	-3.536	619.0	0.02710	0.02675
CH-43	59.4	385.4	462.5	10.447	6.247	4.201	619.0	0.02710	0.02675
CH-44	0.0	330.9	397.0	8.333	8.333	0.0	683.0	0.02518	0.02360
CH-44	13.0	353.5	424.2	8.902	7.748	1.155	683.0	0.02518	0.02360
CH-44	3.0	330.1	403.3	8.404	8.210	0.255	683.0	0.02518	0.02360
CH-44	-3.0	325.7	390.4	8.282	8.456	-0.255	683.0	0.02518	0.02360
CH-44	-45.0	252.6	303.1	6.362	10.145	-3.824	683.0	0.02518	0.02360
CH-44	-50.0	243.4	292.7	6.143	10.391	-4.248	683.0	0.02518	0.02360
CH-44	59.4	434.2	521.0	10.935	5.888	5.047	683.0	0.02518	0.02360

DELTA I SETPOINT QUALITY HAIN FOR AD < -45 IS 2.208/8
 DELTA I SETPOINT QUALITY HAIN FOR AD > 35 IS 2.658/8

TABLE 5-2

TRUCKS AND TRAILERS
 DELTA FINANCIAL CORPORATION

TRUCKS AND TRAILERS DELTA FINANCIAL CORPORATION MAIN 01/30/67

NO	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60
941	0.06941	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000
942	0.06943	3.433	1.709	0.900	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709
943	0.07017	3.100	1.000	0.210	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000
944	0.08496	1.500	1.000	-0.210	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000
		1.333	1.293	-3.143	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293
		4.980	5.001	4.149	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001
		5.000	5.043	4.191	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043
		1.000	1.000	-4.191	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000

NO	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60
941	0.06941	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000
942	0.06943	3.433	1.709	0.900	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709
943	0.07017	3.100	1.000	0.210	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000
944	0.08496	1.500	1.000	-0.210	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000
		1.333	1.293	-3.143	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293
		4.980	5.001	4.149	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001
		5.000	5.043	4.191	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043
		1.000	1.000	-4.191	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000

INCOME TAX CALCULATION

NO	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60	FIN	EO	F60
941	0.06941	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000	0.0	3.000	1.000
942	0.06943	3.433	1.709	0.900	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709	0.919	3.433	1.709
943	0.07017	3.100	1.000	0.210	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000	0.212	3.100	1.000
944	0.08496	1.500	1.000	-0.210	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000	-0.212	1.500	1.000
		1.333	1.293	-3.143	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293	-3.142	1.333	1.293
		4.980	5.001	4.149	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001	4.201	4.980	5.001
		5.000	5.043	4.191	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043	4.243	5.000	5.043
		1.000	1.000	-4.191	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000	-4.243	1.000	1.000

COMPUTER ADDRESSES FOR USE IN PAGES 1, 2, 3, 4, 5, 6, 7, 8, 9, 10, 11, 12, 13, 14, 15, 16, 17, 18, 19, 20, 21, 22, 23, 24, 25, 26, 27, 28, 29, 30, 31, 32, 33, 34, 35, 36, 37, 38, 39, 40, 41, 42, 43, 44, 45, 46, 47, 48, 49, 50, 51, 52, 53, 54, 55, 56, 57, 58, 59, 60, 61, 62, 63, 64, 65, 66, 67, 68, 69, 70, 71, 72, 73, 74, 75, 76, 77, 78, 79, 80, 81, 82, 83, 84, 85, 86, 87, 88, 89, 90, 91, 92, 93, 94, 95, 96, 97, 98, 99, 100

TRUJAH NUCLEAR PLANT
 TRUJAH MAPS 122x124x129x50x1121A, 25x10, 15x10, 75x1125, 100x 441R 8/30/82

.....NIS INTERMEDIATE RANGE CHANNEL CALCULATIONS.....

N35 FULL POWER MICROAMPS = 379.

N35 22% TRIP CURRENT MICROAMPS = 83. (+- 8.)

N35 30% OVERCURRENT MICROAMPS = 114.

N35 16% RESET CURRENT MICROAMPS = 61. (+- 8.)

N35 22% TRIP VOLTAGE = 8.651 (+0.047 -0.052)

N35 30% OVERVOLTAGE = 8.819

N35 16% RESET VOLTAGE = 8.478 (+0.064 -0.072)

N36 FULL POWER MICROAMPS = 449.

N36 22% TRIP CURRENT MICROAMPS = 99. (+- 9.)

N36 30% OVERCURRENT MICROAMPS = 135.

N36 16% RESET CURRENT MICROAMPS = 72. (+- 9.)

N36 22% TRIP VOLTAGE = 8.744 (+0.047 -0.052)

N36 30% OVERVOLTAGE = 8.912

N36 16% RESET VOLTAGE = 8.571 (+0.064 -0.072)

FIGURE 5-1

DOPPLER COEFFICIENT

6937-244

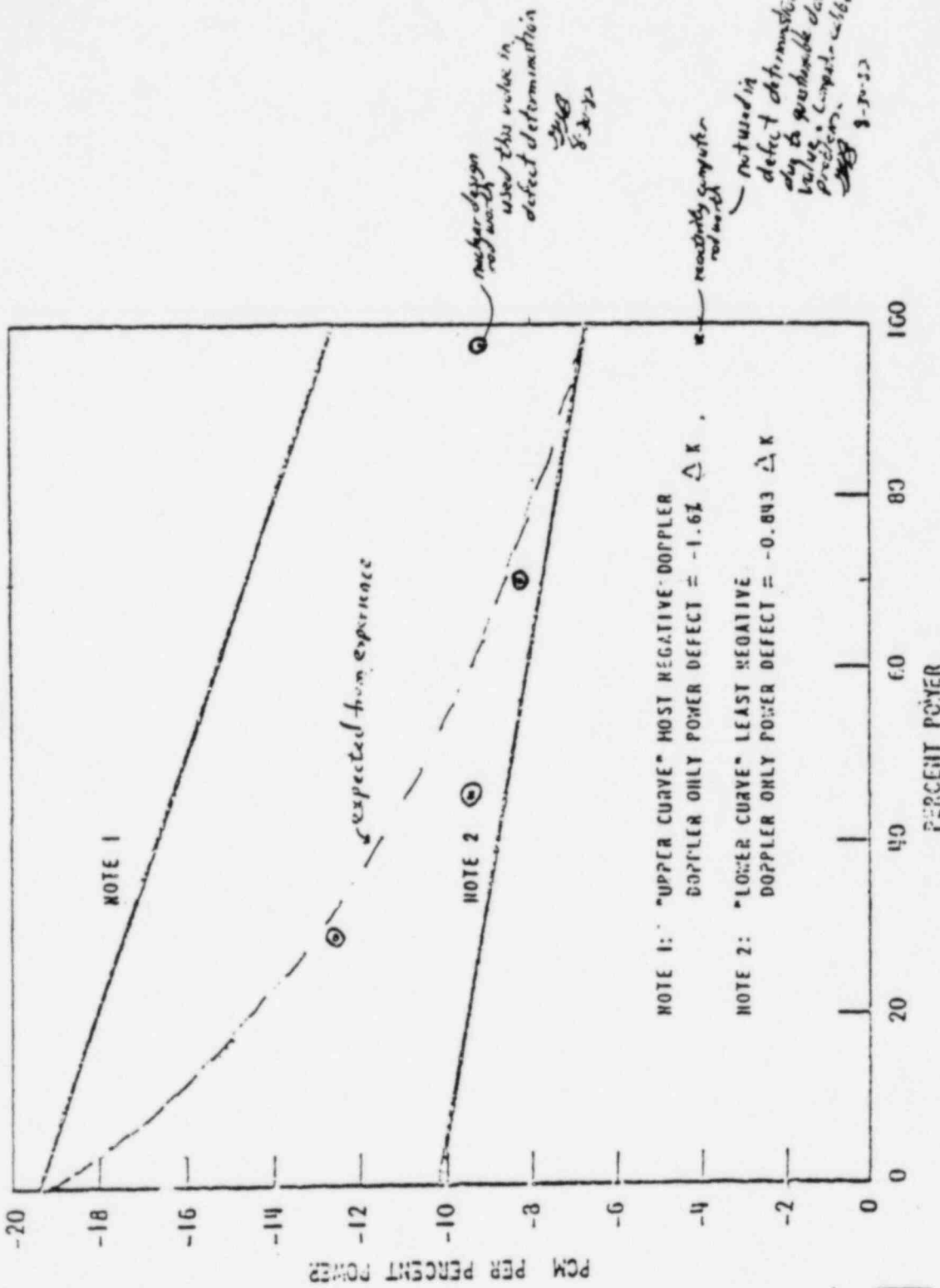


Figure 15.0-5 Doppler Power Coefficient Used In Accident Analysis

use nuclear design rod worth for high power data point because reactivity computer data questionable at this point.

By W. S. Nicholson

Date 8/29/82

Engineering Supervisor Alan S. Cuddy

Date 8/31/82

FIGURE 5-2

LOOP DELTA-T (T-HOT MINUS T-COLD) VS POWER

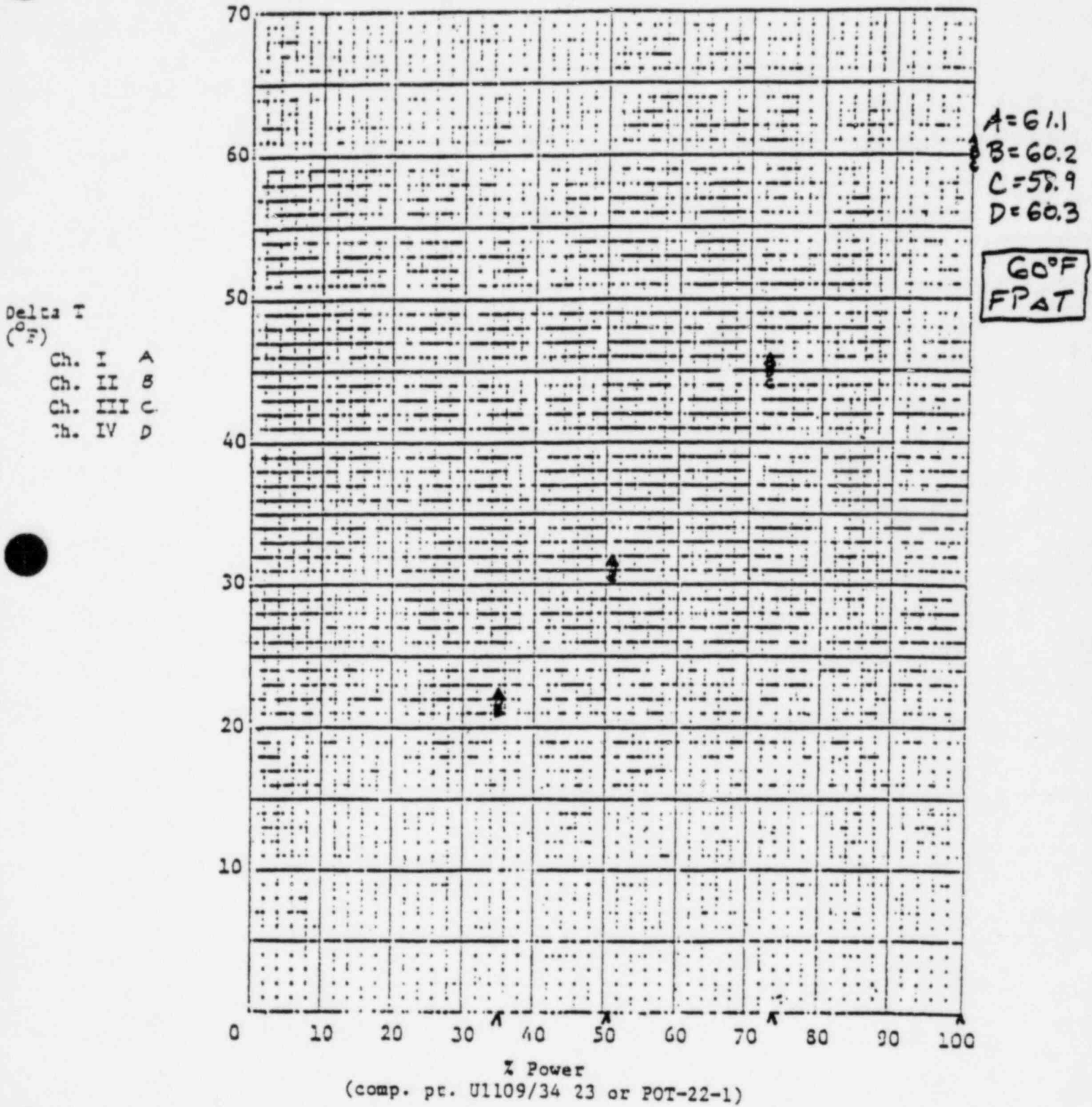
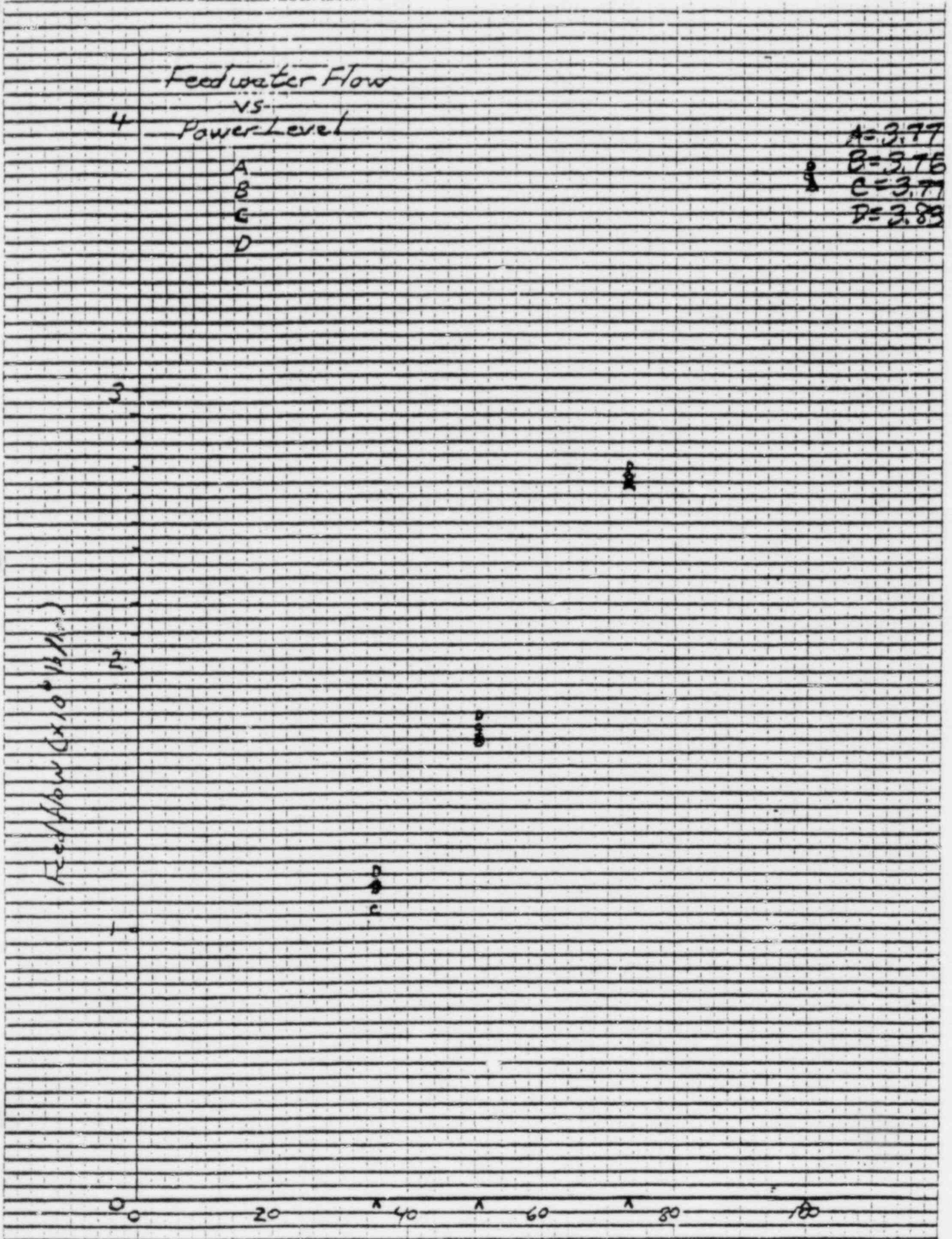
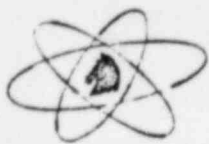


FIGURE 5-3

FEEDWATER FLOW VS POWER LEVEL



% Power



TROJAN NUCLEAR PLANT

TOTAL CHANNEL CURRENT VS POWER LEVEL

CHANNEL 800

41

TOTAL CHANNEL CURRENT μ a

700

600

500

400

300

200

100

0

20

40

60

80

100

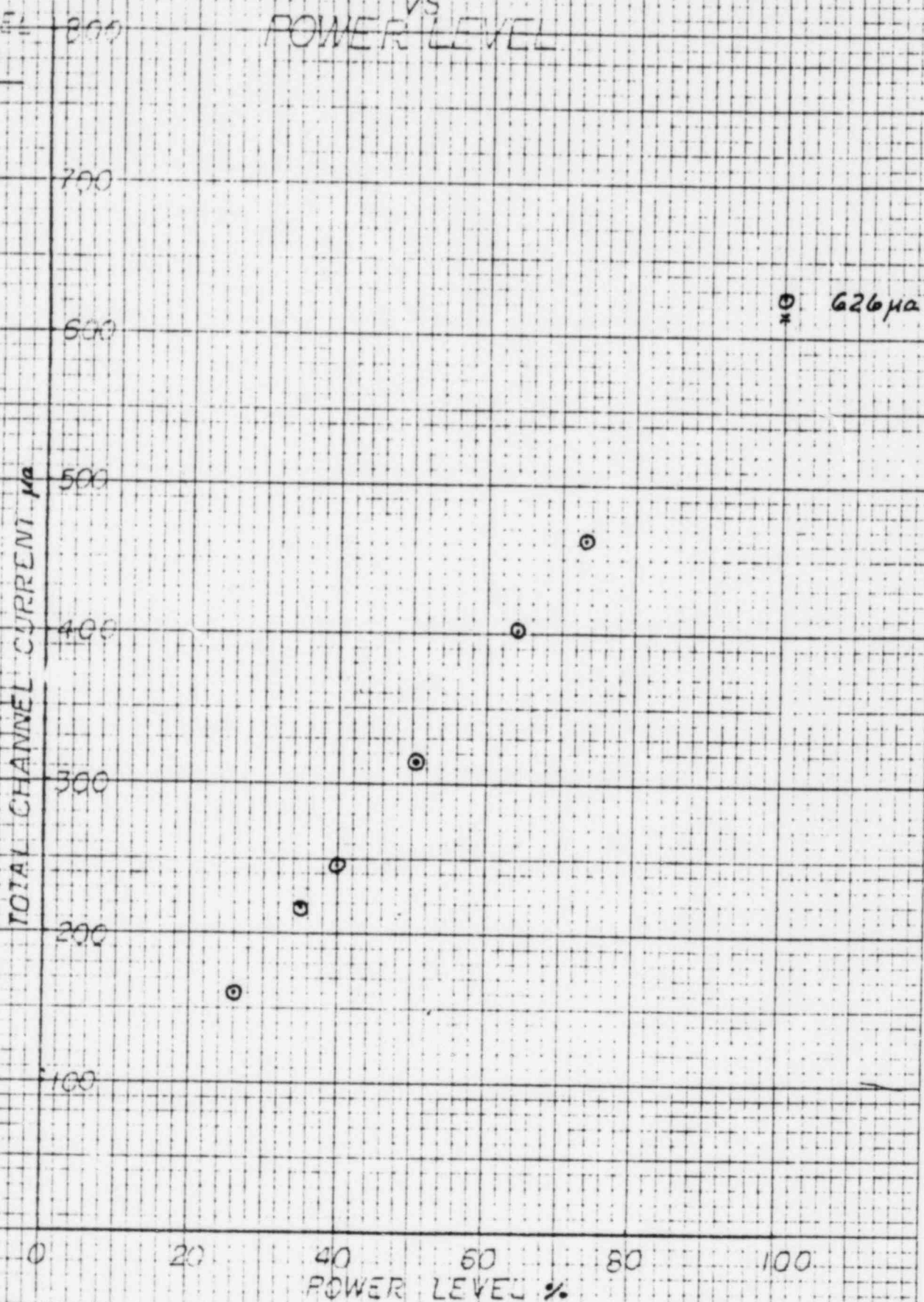
POWER LEVEL %

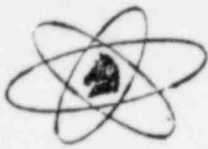
626 μ a

FIGURE 5-4

BY *Saylan*

DATE 8-31-82





TROJAN NUCLEAR PLANT

TOTAL CHANNEL CURRENT VS POWER LEVEL

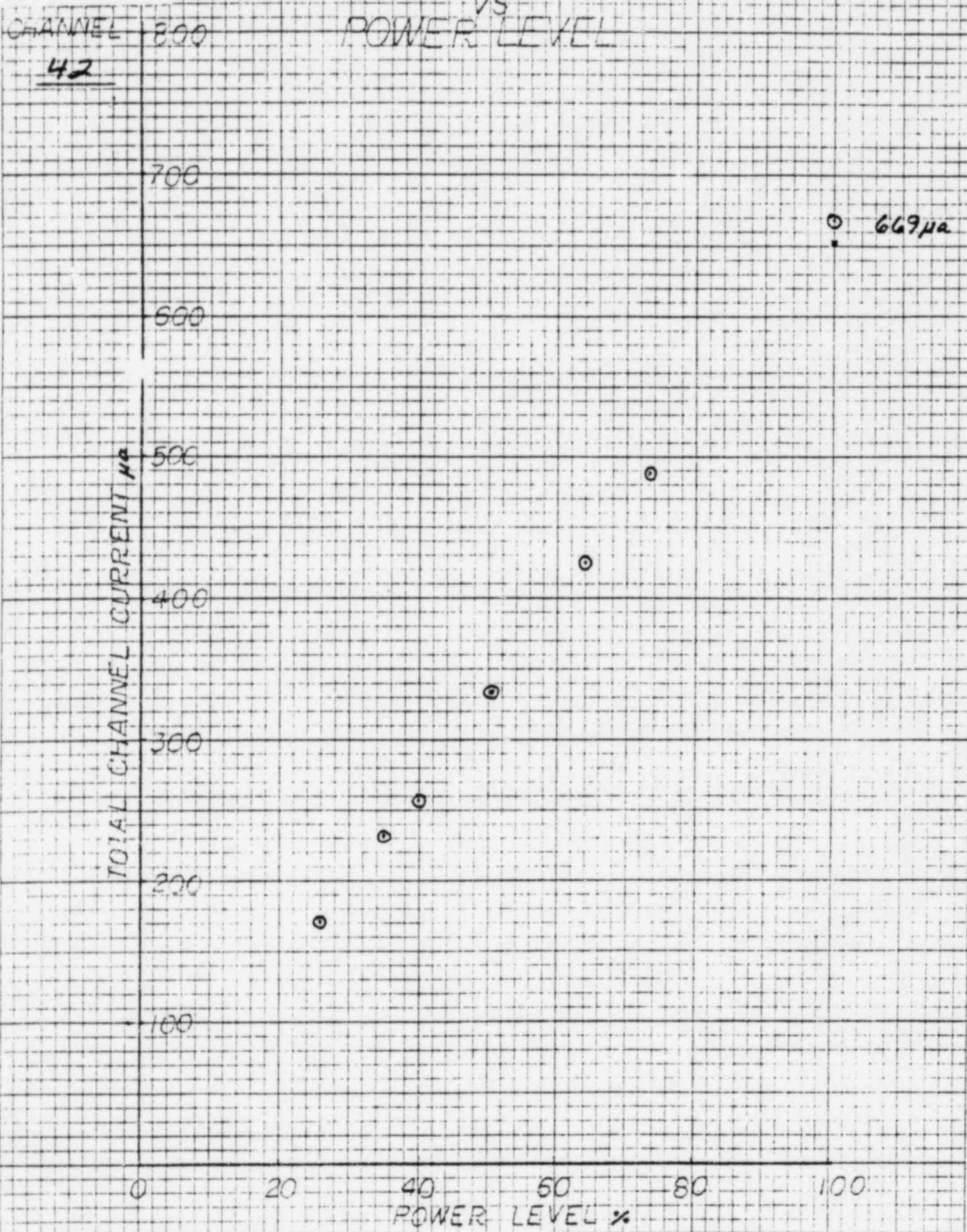
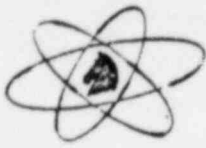


FIGURE 5-5

BY *Ray Bair*

DATE 8-31-82



TROJAN NUCLEAR PLANT

TOTAL CHANNEL CURRENT VS POWER LEVEL

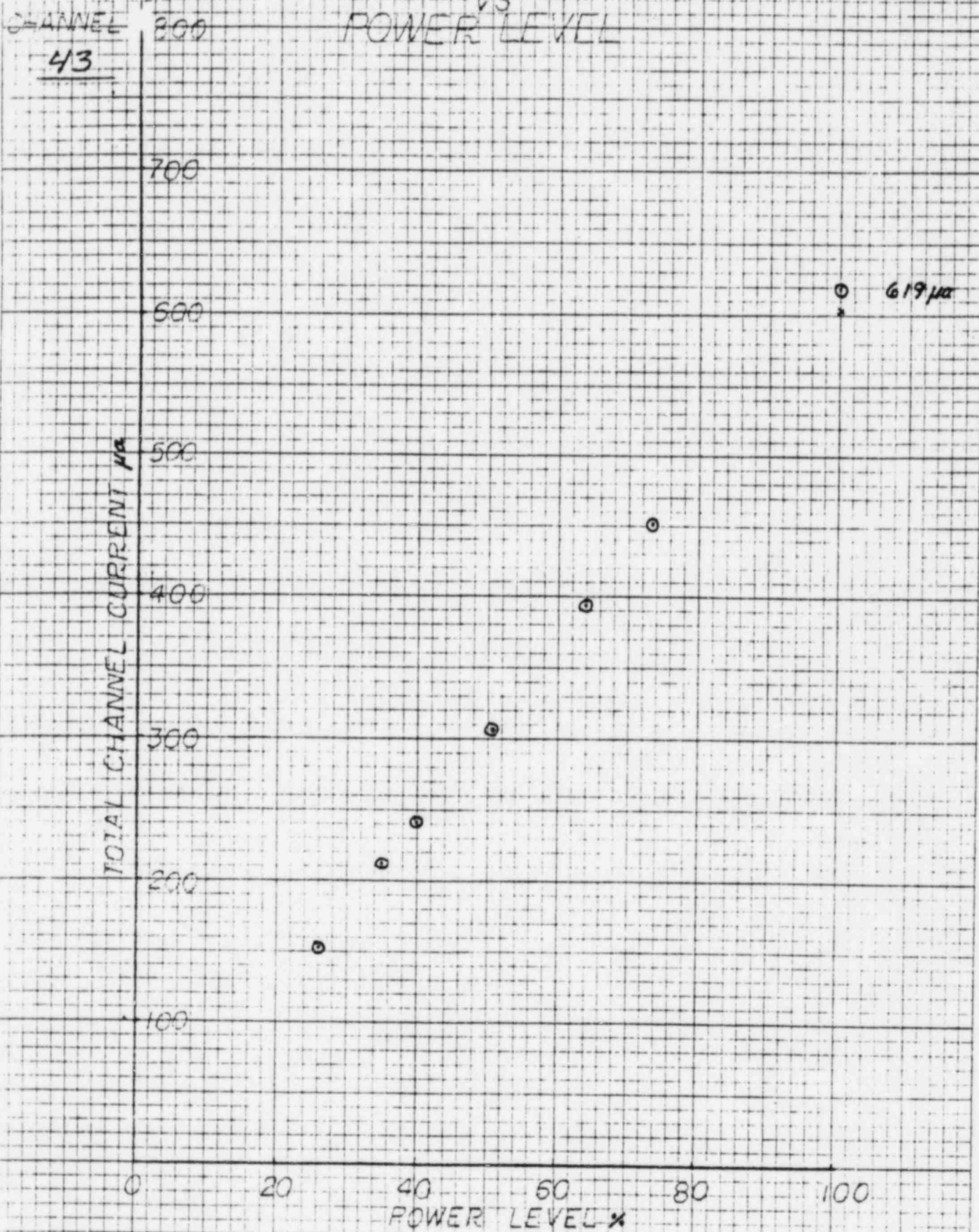


FIGURE 5-6

BY *Hayden*

DATE 8-31-82



TROJAN NUCLEAR PLANT

TOTAL CHANNEL CURRENT

VS
POWER LEVEL

CHANNEL 800

44

TOTAL CHANNEL CURRENT μ a

700

600

500

400

300

200

100

0

20

40

60

80

100

POWER LEVEL %

⊙ 683 μ a
*

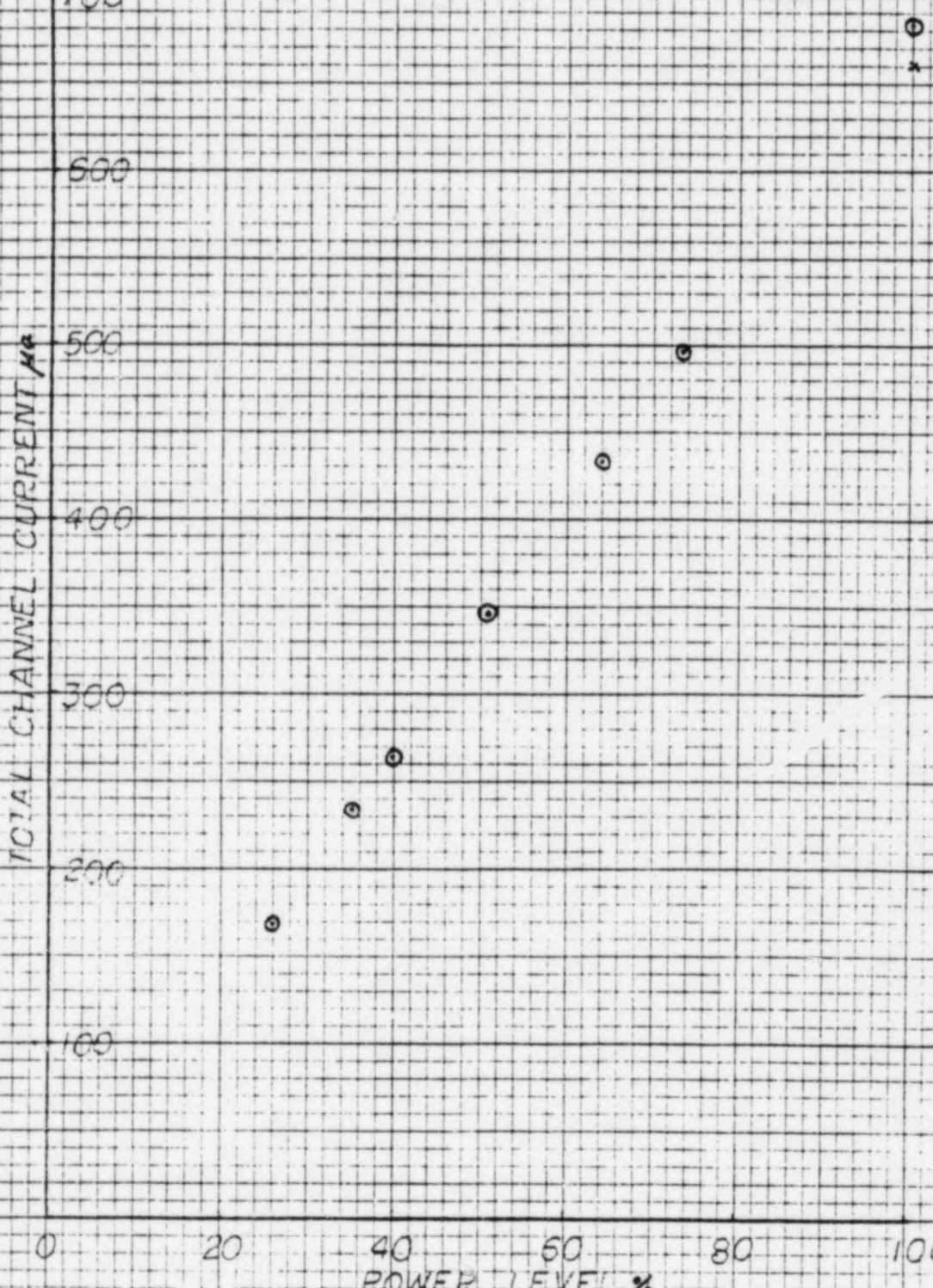
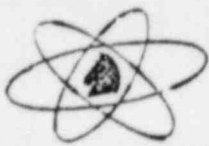


FIGURE 5-7

BY *Stylian*

DATE 8-31-82



TROJAN NUCLEAR PLANT NORMALIZED EXCORE F.P. CURRENT VS INCORE AXIAL OFFSET

CHANNEL 41

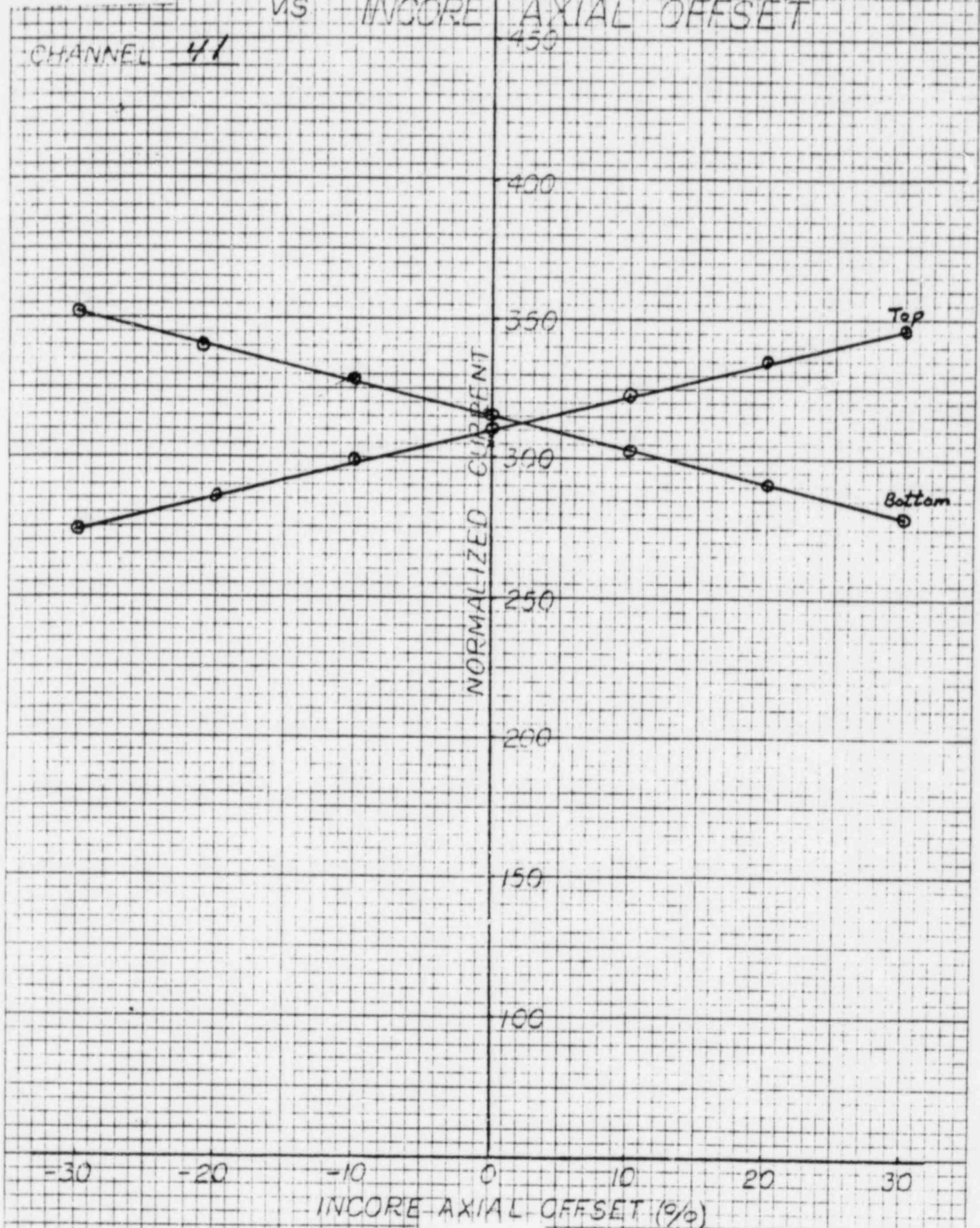
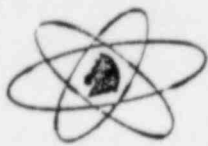


FIGURE 5-8

BY *Hayden*

DATE 8-31-82



TROJAN NUCLEAR PLANT

NORMALIZED EXCORE F.P. CURRENT

VS INCORE AXIAL OFFSET

CHANNEL 42

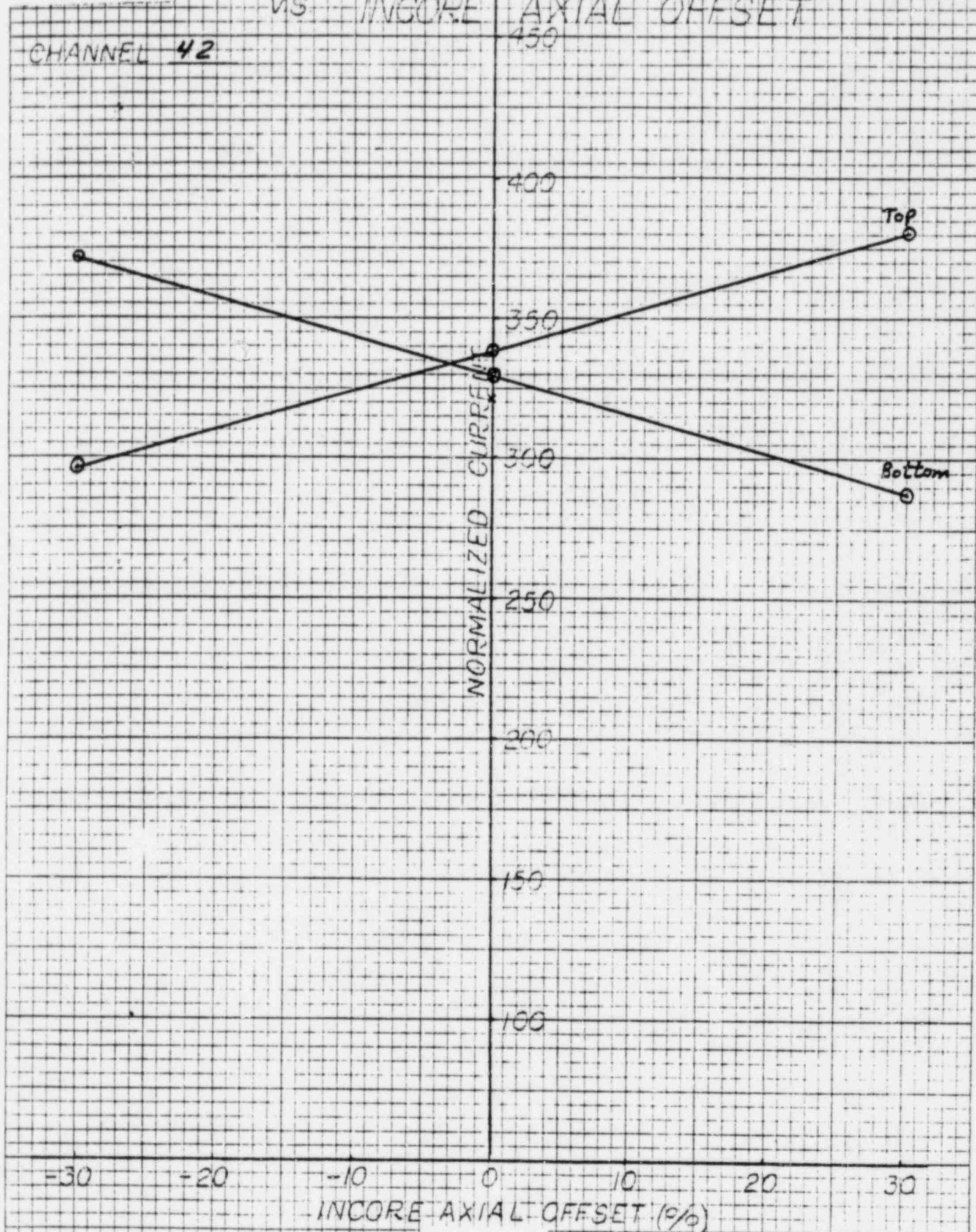
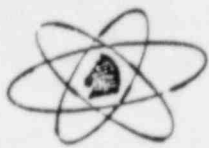


FIGURE 5-9

BY *Daylin*

DATE 8-31-82



TROJAN NUCLEAR PLANT NORMALIZED EXCORE FP. CURRENT VS INCORE AXIAL OFFSET

CHANNEL 43

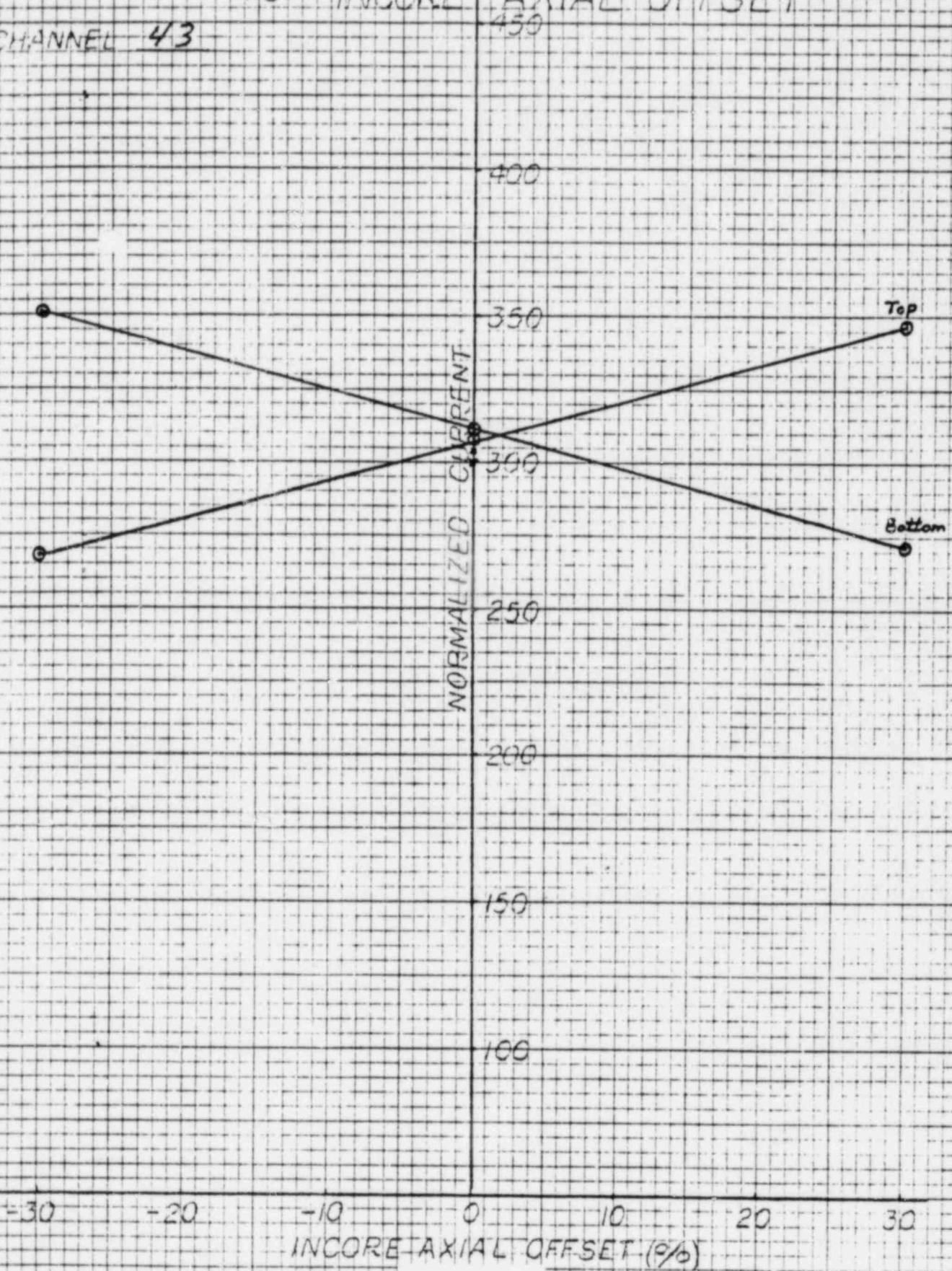
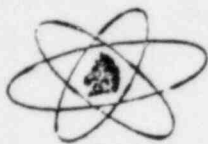


FIGURE 5-10

BY *Gaylin*

DATE 8-31-92



TROJAN NUCLEAR PLANT

NORMALIZED EXCORE F.F. CURRENT VS INCORE AXIAL OFFSET

CHANNEL 44

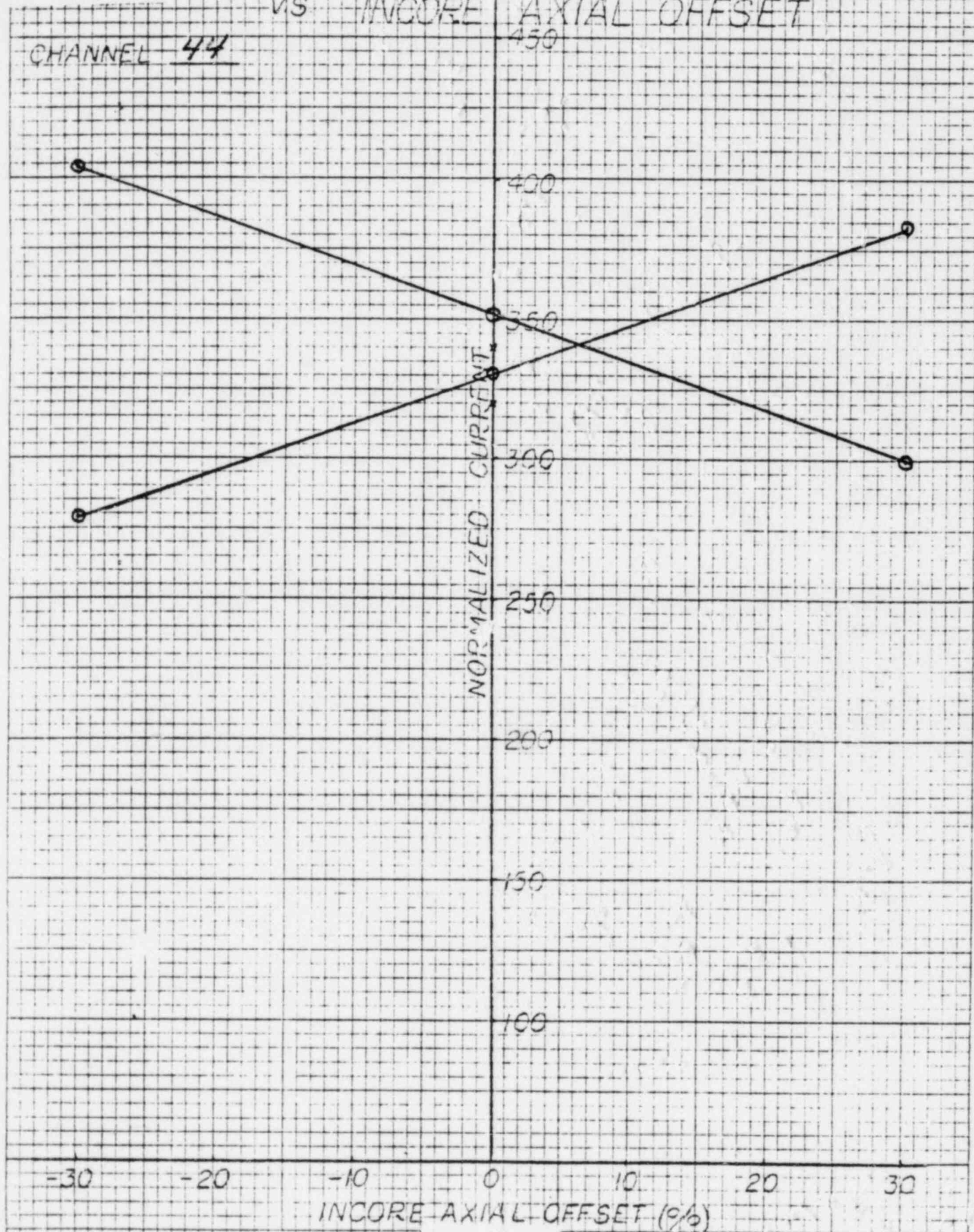
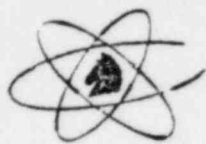


FIGURE 5-11

BY *Hayden*

DATE 8-31-82



TROJAN NUCLEAR PLANT

INCORE vs EXCORE OFFSETS

CHANNEL 41

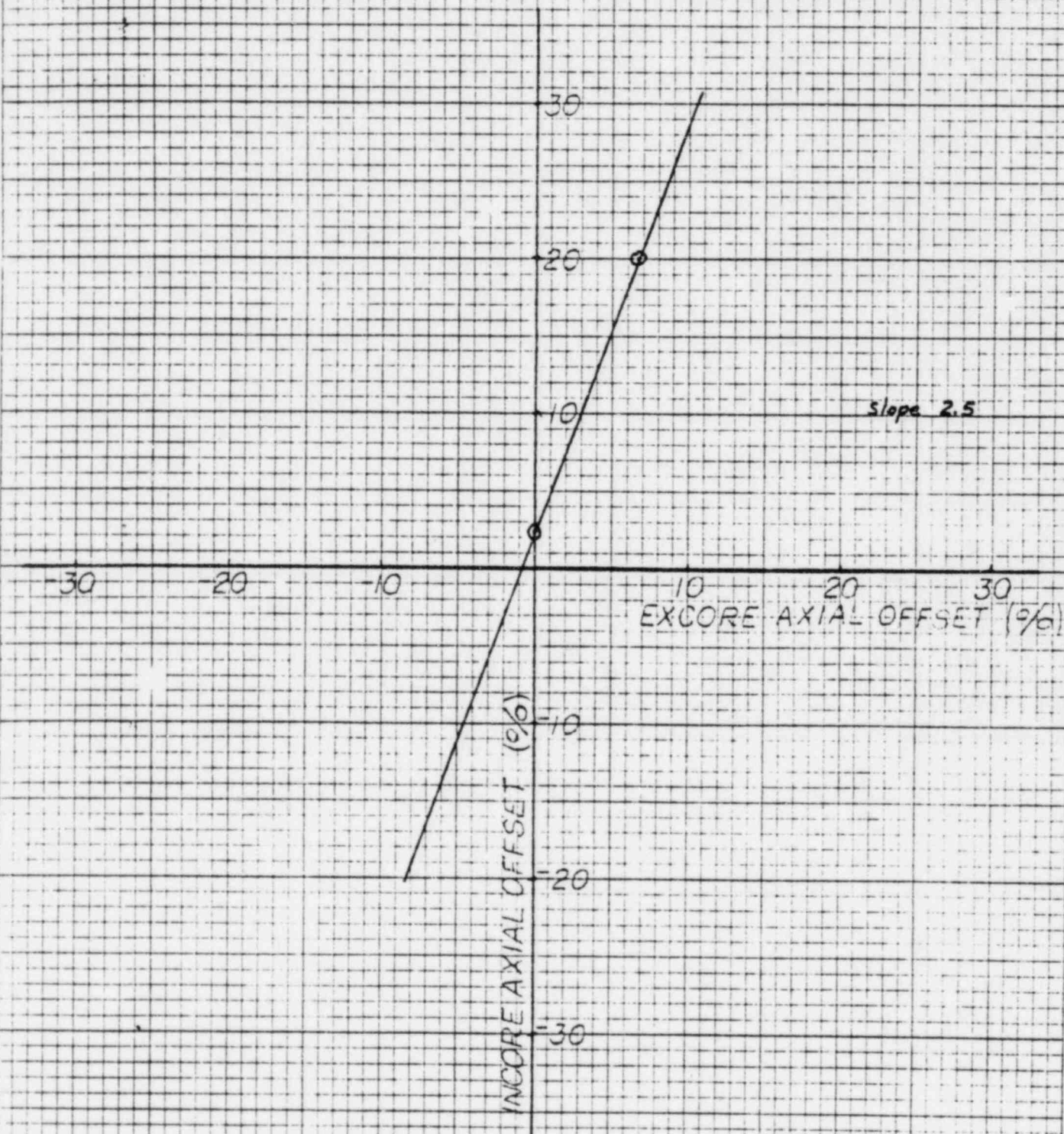
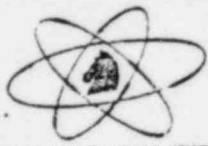


FIGURE 5-12

BY *Hayden*

DATE 8-31-82



TROJAN NUCLEAR PLANT

INCORE vs EXCORE OFFSETS

CHANNEL 42

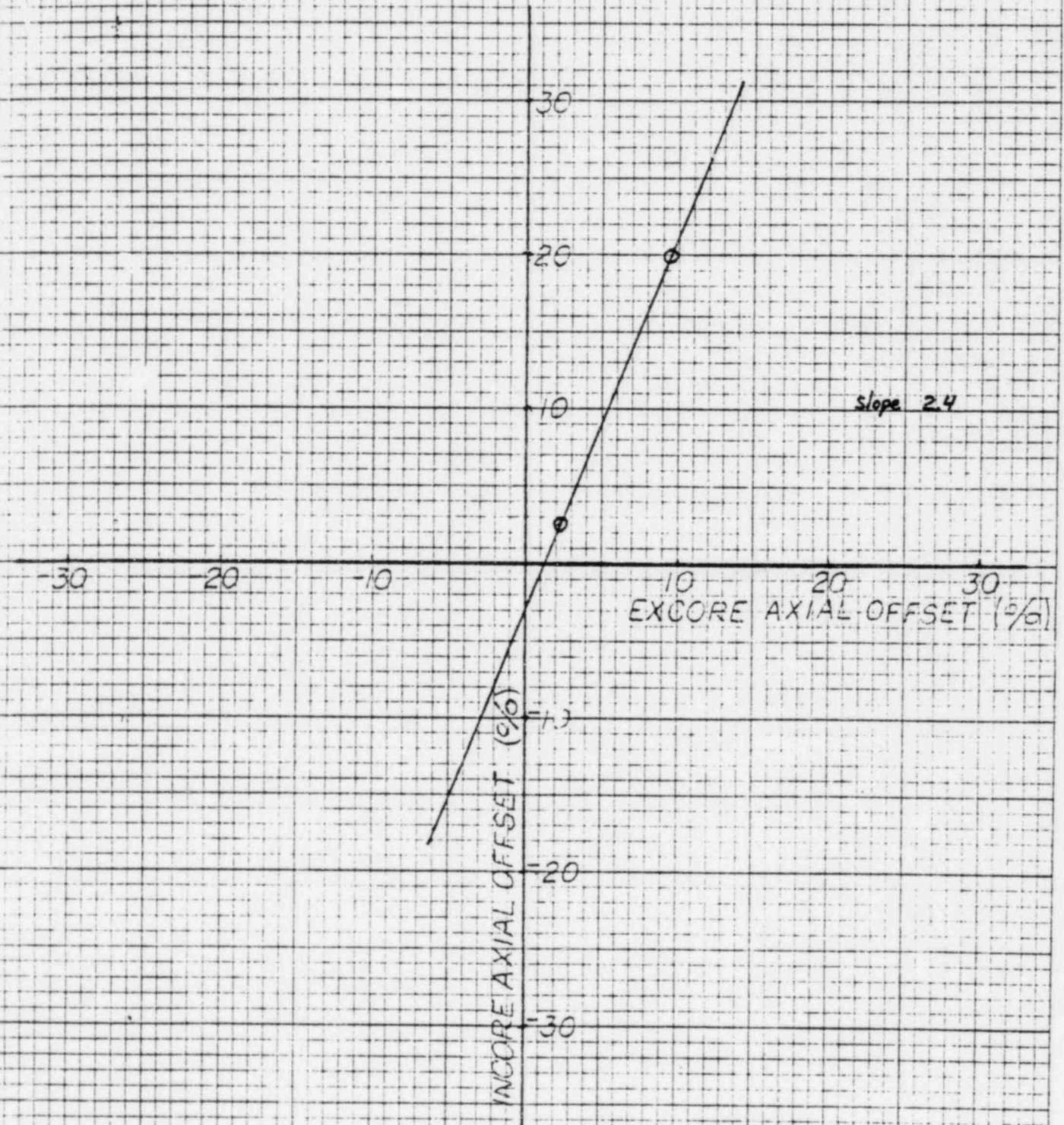
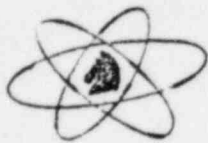


FIGURE 5-13

BY *Day Bair* DATE 8-31-82



TROJAN NUCLEAR PLANT

INCORE vs EXCORE OFFSETS

CHANNEL 43

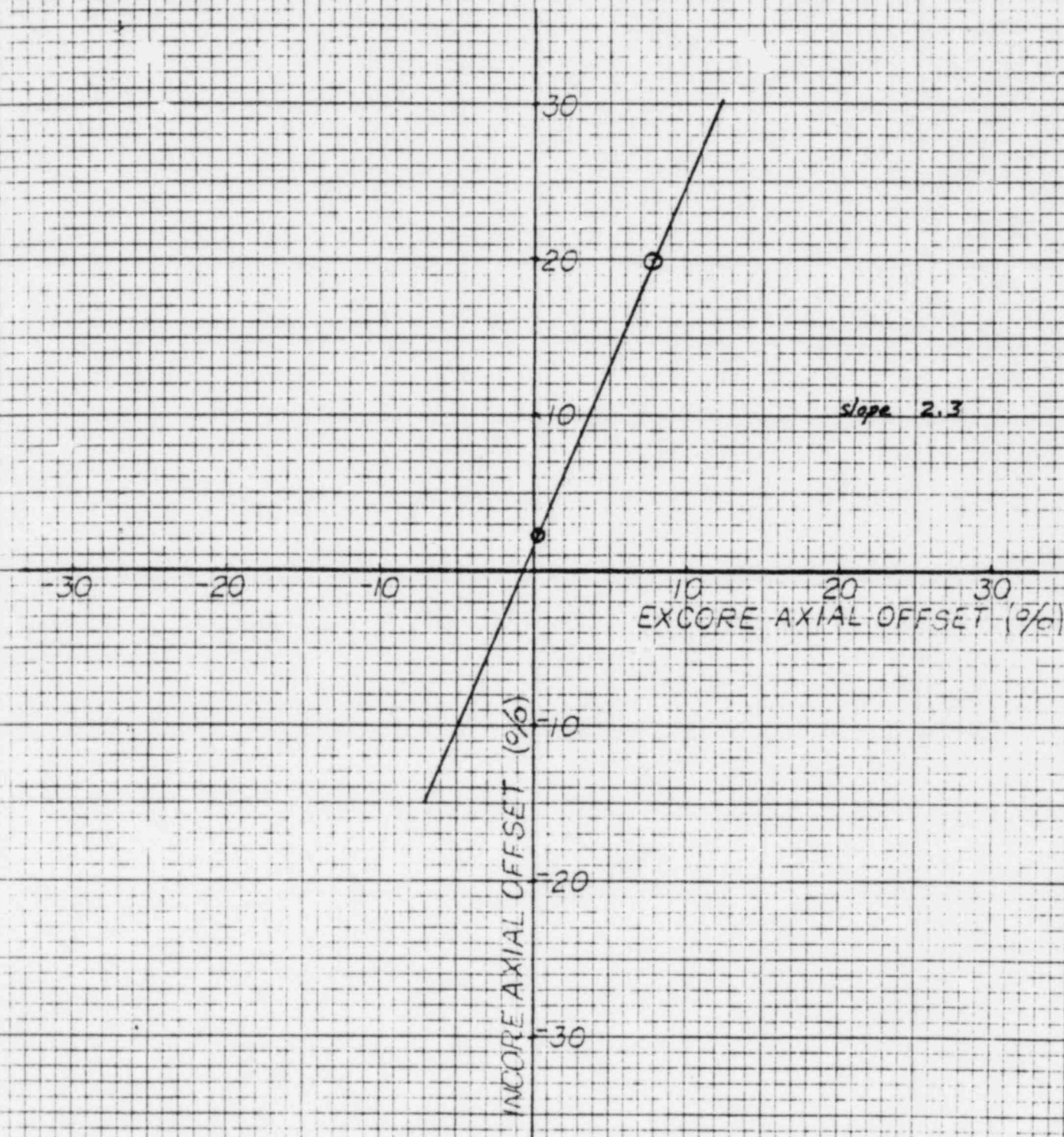
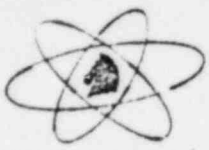


FIGURE 5-14

BY *Ray Bair* DATE 8-31-82



TROJAN NUCLEAR PLANT

INCORE vs EXCORE OFFSETS

CHANNEL 44

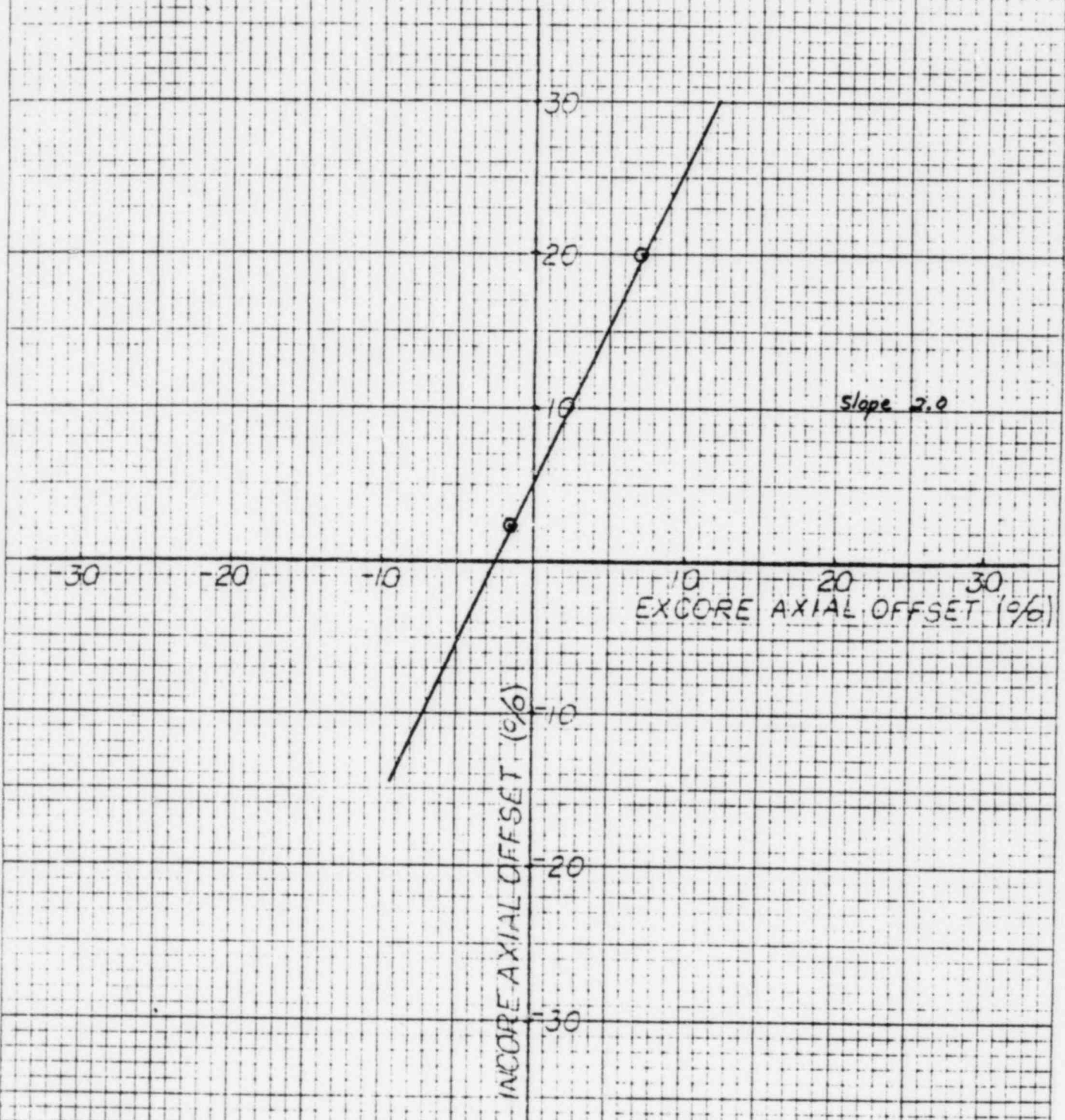


FIGURE 5-15

By *Jay Bain*

DATE 8-31-82

FIGURE 5-16

FLUX MAP 122 (50 PERCENT POWER)

TRJAN FLUX MAP #122 08/26/82 50% HUI25 1 CYS 0W201 MIDDLE AO -BAIR
CALCULATED POWER TILTS (NORMALIZED TO 1.000)

. 1.0085 . 0.9878 .
 . . .
 1.0112 . . . 0.9929

 1.0076 . . . 0.9991

 . 0.9970 . 0.9960 .

.
 .
 1.0099 . 0.9903
 .

 .
 1.0023 . 0.9975
 .
 .

.
 . 0.9981 .
 . . .
 1.0094 . 0.9960
 . . .
 . 0.9965 .
 .

POSITIVE "Y" VS. NEGATIVE "Y" TILT
1.0001 0.9999

POSITIVE "X" VS. NEGATIVE "X" TILT
0.9939 1.0061

FIGURE 5-16

TROJAN FLUX MAP #122 08/26/82 50% RU(25) CY5 D=201 MIDDLE AU -RAIN

THE FOLLOWING CALCULATIONS ARE BASED ON A DIVISION OF THE CORE INTO *OCTANTS*, THAT IS, QUADRANTS DIVIDED INTO TWO AXIAL REGIONS OF EQUAL VOLUME.

RELATIVE POWER IN UPPER HALF OF CORE	RELATIVE POWER IN LOWER HALF OF CORE	PERCENT AXIAL OFFSET TOWARD TOP OF CORE	CORE AVERAGE AXIAL OFFSET
(-++) . (+++)	(-++) . (+++)	(-++) . (+++)	
1.0831 . 1.0618	0.9366 . 0.9189	7.252 . 7.215	7.297
.	
1.0789 . 1.0681	0.9256 . 0.9270	7.645 . 7.074	
(--+) . (+-)	(--+) . (+-)	(--+) . (+-)	
POWER TILT IN UPPER HALF OF CORE	POWER TILT IN LOWER HALF OF CORE		
(-++) . (+++)	(-++) . (+++)	(-++) . (+++)	
1.0094 . 0.9896	1.0104 . 0.9912	THESE . EDITS	
.	
1.0055 . 0.9955	0.9985 . 0.9999	ADDED . JAN., 1969	
(--+) . (+-)	(--+) . (+-)	(--+) . (+-)	

TROJAN FLUX MAP #122 08/26/82 50% BU(25) 1 CY5 DW201 MIDDLE AU -RAIN

TOP TWENTY NUCLEAR F-DELTA-H

TOP TWENTY NUCLEAR F0

313J 2EM	F0HN=1.3543	313J 2EM	F0N=1.6953
253M 5MD	F0HN=1.3519	314J 2EL	F0N=1.6898
314J 2EL	F0HN=1.3498	213P 9NM	F0N=1.6641
276L 4DM	F0HN=1.3458	209P 7NE	F0N=1.6636
213P 9NM	F0HN=1.3430	253M 5MD	F0N=1.6626
362G 2LM	F0HN=1.3423	362G 2LM	F0N=1.6556
364G 2MN	F0HN=1.3420	210P 7LE	F0N=1.6555
209P 7NE	F0HN=1.3400	364G 2MN	F0N=1.6550
210P 7LE	F0HN=1.3387	276L 4DM	F0N=1.6548
363G 2HL	F0HN=1.3379	214P 9LM	F0N=1.6531
214P 9LM	F0HN=1.3355	436D 7MF	F0N=1.6522
333J14ED	F0HN=1.3335	437D 7MD	F0N=1.6516
487H 9DM	F0HN=1.3292	363G 2ML	F0N=1.6500
488H 9FM	F0HN=1.3250	487H 9DM	F0N=1.6455
433D 5ED	F0HN=1.3237	222N 3ML	F0N=1.6432
257M 7ED	F0HN=1.3227	257M 7ED	F0N=1.6417
256M 7DE	F0HN=1.3227	256M 7DE	F0N=1.6417
334J14EF	F0HN=1.3222	440D 9ML	F0N=1.6410
244N13MF	F0HN=1.3207	244N13MF	F0N=1.6407
247K 5MN	F0HN=1.3197	488H 9FM	F0N=1.6406

NOTE = VALUES ARE BEST ESTIMATE AND DO NOT INCLUDE ENGINEERING OR NUCLEAR UNCERTAINTY.

LOCA ENVELOPE LIMIT AXIALLY			
AXIAL POINT	FQ(Z) LIMIT	MEAS. FQ(Z)	SOURCE NO. ID
1	1.9952	0.6703	297K 5MN
2	2.4035	0.6700	297K 5MN
3	2.8026	1.0040	297K 5MN
4	3.2062	1.2116	297K 5MN
5	3.6053	1.3263	297K 5MN
6	4.0043	1.3536	297K 5MN
7	4.1760	1.3950	304K 9DM
8	4.3802	1.5505	304K 9DM
9	4.3894	1.6253	297K 5MN
10	4.4034	1.6705	297K 5MN
11	4.4080	1.7113	440D 9ML
12	4.4266	1.7369	436D 7MF
13	4.4358	1.7605	313J 2FN
14	4.4494	1.7606	313J 2FN
15	4.4590	1.6931	257M 7ED
16	4.4730	1.7194	333J14FD
17	4.4822	1.7825	213P 9NM
18	4.4962	1.8036	313J 2FN
19	4.5101	1.8161	313J 2FN
20	4.5194	1.6238	313J 2FN
21	4.5286	1.8334	313J 2FN
22	4.5426	1.7974	313J 2FN
23	4.5565	1.7788	313J 2FN
24	4.5658	1.6060	213P 9NM
25	4.5797	1.7345	244N13MF
26	4.5890	1.7574	244N13MF
27	4.6029	1.7661	244N13MF
28	4.6168	1.7695	244N13MF
29	4.6261	1.7558	222N 3ML
30	4.6354	1.7441	222N 3ML

FIGURE 5-16

AXIAL POINT	FOUR FIVE FLOPPY LIMIT	MEAS. FOUR	AXIALLY SOURCE
31	4.6400	1.7199	222N 3M
32	4.6400	1.6906	222N 3M
33	4.6400	1.6265	244N13MF
34	4.6400	1.6753	244N13MF
35	4.6400	1.6023	244N13MF
36	4.6400	1.6048	244N13MF
37	4.6400	1.6787	244N13MF
38	4.6400	1.6607	244N13MF
39	4.6400	1.6379	244N13MF
40	4.6400	1.5930	222N 3M
41	4.6400	1.4548	433B12FD
42	4.6400	1.5435	244N13MF
43	4.6400	1.5576	244N13MF
44	4.6400	1.5495	244N13MF
45	4.6400	1.5299	244N13MF
46	4.6400	1.5092	244N13MF
47	4.6400	1.4718	244N13MF
48	4.6400	1.4332	244N13MF
49	4.6400	1.3594	473C13FF
50	4.6400	1.2514	244N13MF
51	4.6400	1.3025	25JM 5MD
52	4.6400	1.2914	25JM 5MD
53	4.6400	1.2549	25JM 5MD
54	4.6400	1.2050	25JM 5MD
55	4.6400	1.1421	25JM 5MD
56	4.6400	1.0668	25JM 5MD
57	4.6400	0.9638	25JM 5MD
58	4.6400	0.8443	41H 5FD
59	4.6400	0.7052	434D 5FN
60	4.6400	0.5241	434D 5FN

THRUJAW FLUA MAP #122	08/26/82	508 HUC25	1 CY5 DM201 MIDDLE AD	-HAIR	
POINT	SUBJECT	NUCLEAR FAX	POINT	SOURCE	NUCLEAR FAX
1	297 K 5M1	1.5746	31	222 N 3ML	1.3747
2	297 K 5M1	1.5753	32	222 N 3ML	1.3967
3	297 K 5M1	1.5491	33	244 N13MF	1.4463
4	297 K 5M1	1.5144	34	244 N13MF	1.3474
5	297 K 5M1	1.4856	35	244 N13MF	1.3497
6	297 K 5M1	1.4372	36	244 N13MF	1.3475
7	304 K 9M1	1.4553	37	244 N13MF	1.4013
8	304 K 9M1	1.4346	38	244 N13MF	1.3997
9	297 K 5M1	1.3827	39	244 N13MF	1.3944
10	297 K 5M1	1.3683	40	222 N 3ML	1.3441
11	440 D 9M1	1.3617	41	493 H12ED	1.4022
12	436 D 7M1	1.3554	42	244 N13MF	1.4357
13	313 J 2EN	1.3576	43	244 N13MF	1.4031
14	313 J 2EN	1.3595	44	244 N13MF	1.4055
15	257 H 7ED	1.3931	45	244 N13MF	1.4011
16	333 J14ED	1.3441	46	244 N13MF	1.4009
17	213 P 9M1	1.3508	47	244 N13MF	1.3877
18	313 J 2EN	1.3602	48	244 N13MF	1.3801
19	313 J 2EN	1.3647	49	473 C13EF	1.3835
20	313 J 2EN	1.3677	50	244 N13MF	1.3968
21	313 J 2EN	1.3782	51	253 M 5MD	1.3636
22	313 J 2EN	1.3617	52	253 M 5MD	1.3705
23	313 J 2EN	1.3823	53	253 M 5MD	1.3741
24	213 P 9M1	1.3701	54	253 M 5MD	1.3772
25	244 N13MF	1.3679	55	253 M 5MD	1.3765
26	244 N13MF	1.3601	56	253 M 5MD	1.3880
27	244 N13MF	1.3664	57	253 M 5MD	1.3433
28	244 N13MF	1.3724	58	433 D 5ED	1.4175
29	222 N 3ML	1.3695	59	434 D 5EN	1.4924
30	222 N 3ML	1.3737	60	434 D 5EN	1.4986

NOTE= VALUES ARE BEST ESTIMATE AND DO NOT INCLUDE ENGINEERING OR NUCLEAR UNCERTAINTY.

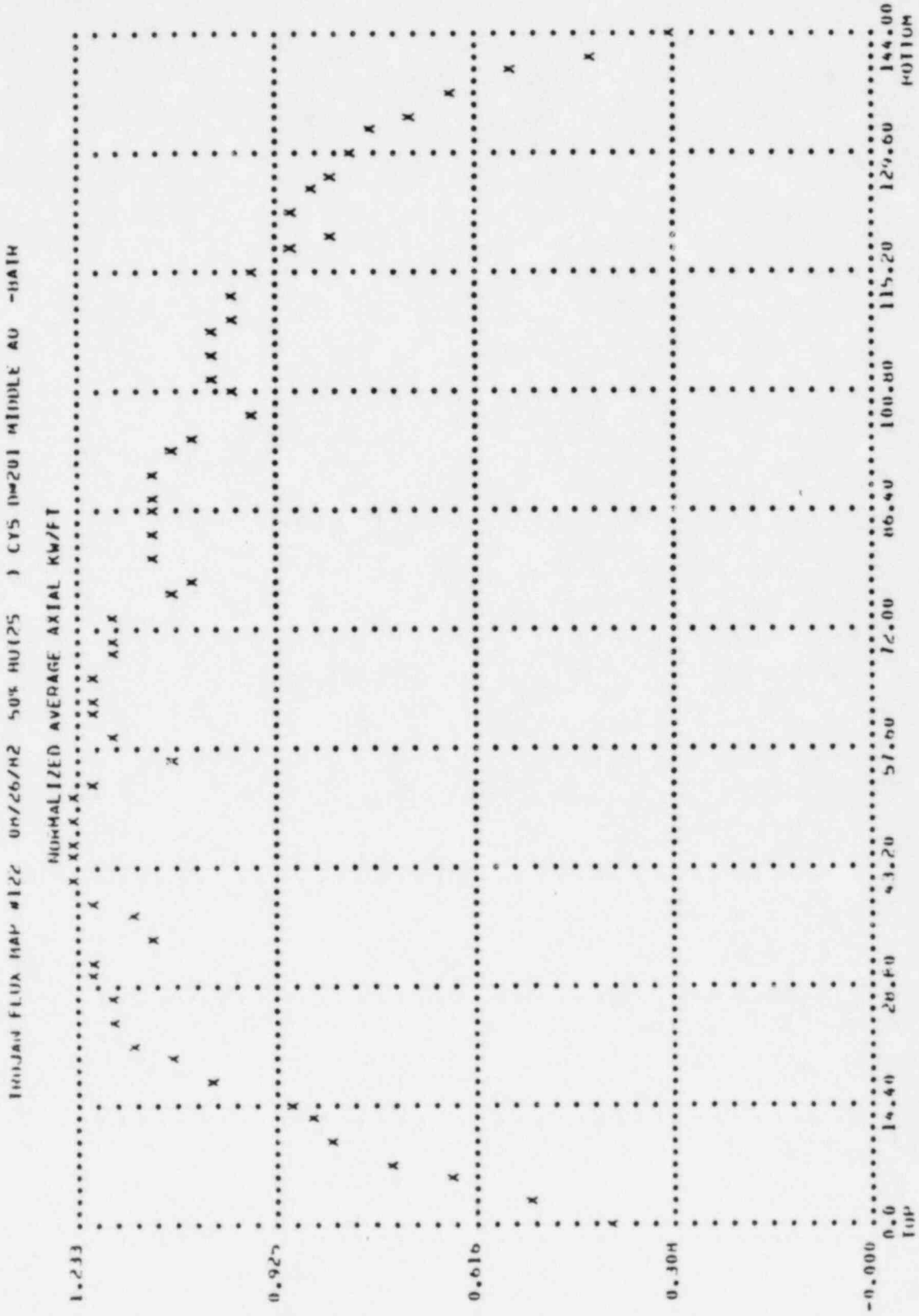


FIGURE 5-17

FLUX MAP 125 (100 PERCENT POWER)

TROJAN FLUX MAP #125 06/28/82 100% 60(75) 1 CY5 0w208 -HAIR
CALCULATED POWER TILTS (NORMALIZED TO 1.000)

. 1.0022 . 0.9961 .	.	.
.
1.0071 0.9864	1.0047 . 0.9912	. . 0.9991 . .
.
1.0099 1.0057	1.0013 . 1.0028	1.0085 . . 0.9961
.
. 0.9927 . 0.9998 0.9963 . .

POSITIVE "Y" VS. NEGATIVE "Y" TILT
0.9980 1.0020

POSITIVE "X" VS. NEGATIVE "X" TILT
0.9970 1.0030

FIGURE 5-17

TRIUMF FLUX MAP #125 08/28/82 100% HD(75) CYS DW298 -HAIR

THE FOLLOWING CALCULATIONS ARE BASED ON A DIVISION OF THE CORE INTO "OCTANTS", THAT IS, QUADRANTS DIVIDED INTO TWO AXIAL REGIONS OF EQUAL VOLUME.

RELATIVE POWER IN UPPER HALF OF CORE	RELATIVE POWER IN LOWER HALF OF CORE	PERCENT AXIAL OFFSET TOWARD TOP OF CORE	CORE AVERAGE AXIAL OFFSET
(-++) . (++)	(-++) . (++)	(-++) . (++)	
0.9483 . 0.9731	1.0210 . 1.0094	-1.626 . -1.831	-1.837
.	
0.9827 . 0.9823	1.0199 . 1.0232	-1.855 . -2.037	
(--+) . (++)	(--+) . (++)	(--+) . (++)	
POWER TILT IN UPPER HALF OF CORE	POWER TILT IN LOWER HALF OF CORE		
(-++) . (++)	(-++) . (++)	(-++) . (++)	
1.0050 . 0.9913	1.0026 . 0.9912	THESE . FDITS	
.	
1.0011 . 1.0007	1.0015 . 1.0047	ADDED . JAN., 1969	
(--+) . (++)	(--+) . (++)	(--+) . (++)	

FIGURE 5-17

TROJAN FLUX MAP #125 08/28/82 100% BU175 1 CYS 0*208

-BAIR

TOP TWENTY NUCLEAR F-DELTA-H

TOP TWENTY NUCLEAR F₀

321J 80F	F ₀ HN=1.3691	452C 3EL	F ₀ N=1.5949
214P 9LM	F ₀ HN=1.3445	453C 36I	F ₀ N=1.5769
210P 7LF	F ₀ HN=1.3418	210P 7LE	F ₀ N=1.5762
301K 7ED	F ₀ HN=1.3414	214P 9LM	F ₀ N=1.5693
300K 7EN	F ₀ HN=1.3387	487H 9DM	F ₀ N=1.5671
311J 2EN	F ₀ HN=1.3343	313J 2EN	F ₀ N=1.5628
214P 9LM	F ₀ HN=1.3365	448H 9EM	F ₀ N=1.5620
2534 540	F ₀ HN=1.3351	213P 9NM	F ₀ N=1.5606
314J 2EL	F ₀ HN=1.3340	314J 2EL	F ₀ N=1.5578
286L120F	F ₀ HN=1.3335	204P 7NE	F ₀ N=1.5533
204P 70F	F ₀ HN=1.3296	253M 5M0	F ₀ N=1.5501
276L 404	F ₀ HN=1.3294	323J 80E	F ₀ N=1.5500
395F 944	F ₀ HN=1.3283	244N13MF	F ₀ N=1.5483
487R 9DM	F ₀ HN=1.3280	362G 2LM	F ₀ N=1.5444
398F11L0	F ₀ HN=1.3277	3546 2MN	F ₀ N=1.5439
4400 94L	F ₀ HN=1.3253	473C13FF	F ₀ N=1.5429
320J 60F	F ₀ HN=1.3251	276L 40M	F ₀ N=1.5428
4410 944	F ₀ HN=1.3249	363G 2ML	F ₀ N=1.5394
418F1004	F ₀ HN=1.3243	493H12ED	F ₀ N=1.5392
379G12F1	F ₀ HN=1.3240	491H11ED	F ₀ N=1.5343

NOTE = VALUES ARE BEST ESTIMATE AND DO NOT INCLUDE ENGINEERING OR NUCLEAR UNCERTAINTY.

FIGURE 5-17

AXIAL POINT	LOCA FLOW DEVELOPMENT LIMIT	MAX. FLOW	5000rCF	AXIALLY
1	2.9976	0.6182	301K 7FD	301K 7FD
2	1.2014	0.7057	301K 7FD	301K 7FD
3	1.9011	0.9505	301K 7FD	301K 7FD
4	1.6031	1.0878	301K 7FD	301K 7FD
5	1.0026	1.2009	323J BDE	301K 7FD
6	2.0022	1.2955	301K 7FD	323J BDE
7	2.0080	1.2684	301K 7FD	301K 7FD
8	2.1901	1.4192	323J BDE	323J BDE
9	2.1947	1.4949	323J BDE	323J BDE
10	2.2017	1.5443	323J BDE	323J BDE
11	2.2040	1.5771	323J BDE	323J BDE
12	2.2133	1.6011	323J BDE	323J BDE
13	2.2179	1.6223	323J BDE	323J BDE
14	2.2249	1.6205	323J BDE	323J BDE
15	2.2295	1.5909	323J BDE	323J BDE
16	2.2365	1.5321	301K 7FD	301K 7FD
17	2.2411	1.6470	323J BDE	323J BDE
18	2.2431	1.6045	323J BDE	323J BDE
19	2.2550	1.6727	323J BDE	323J BDE
20	2.2597	1.6759	323J BDE	323J BDE
21	2.2643	1.6762	323J BDE	323J BDE
22	2.2713	1.6724	323J BDE	323J BDE
23	2.2782	1.6392	323J BDE	323J BDE
24	2.2829	1.5394	333J14FD	333J14FD
25	2.2894	1.6328	210P 7LE	210P 7LE
26	2.2945	1.6782	210P 91M	210P 91M
27	2.3014	1.6893	210P 7LE	210P 7LE
28	2.3084	1.7020	210P 7LE	210P 7LE
29	2.3130	1.7046	210P 7LE	210P 7LE
30	2.3177	1.6946	210P 7LE	210P 7LE

FIGURE 5-17

AXIAL POINT	LOCA EQ ENVELOPE LIMIT AXIALLY		SOURCE
	F ₁ (Z) LIMIT	MEAS. F ₂ (Z)	
31	2.3200	1.6816	210P 7LF
32	2.3200	1.6248	452C 3FL
33	2.3200	1.6049	333J14ED
34	2.3200	1.6702	210P 7LF
35	2.3200	1.6836	210P 7LE
36	2.3200	1.7036	452C 3FL
37	2.3200	1.7185	452C 3FL
38	2.3200	1.7249	452C 3FL
39	2.3200	1.7143	452C 3FL
40	2.3200	1.6930	452C 3FL
41	2.3200	1.5886	452C 3FL
42	2.3200	1.6241	333J14ED
43	2.3200	1.6717	452C 3FL
44	2.3200	1.6743	452C 3FL
45	2.3200	1.6823	452C 3FL
46	2.3200	1.6717	452C 3FL
47	2.3200	1.6558	452C 3FL
48	2.3200	1.6248	452C 3FL
49	2.3200	1.5737	452C 3FL
50	2.3200	1.5103	333J14ED
51	2.3200	1.5112	333J14ED
52	2.3200	1.4931	323J 80E
53	2.3200	1.4722	409E 4NM
54	2.3200	1.4353	409E 4r14
55	2.3200	1.3910	409E 4NM
56	2.3200	1.3283	409E 4NM
57	2.3200	1.2287	409E 4NM
58	2.3200	1.1196	409E 4NM
59	2.3200	0.9602	317J 4FF
60	2.3200	0.4413	333J14ED

FIGURE 5-17

	P	N	M	L	K	J	I	H	G	F	E	D	C	B	A
1	0.739	0.998	0.706	0.623	0.527	0.701	0.524	0.823	0.727						
2	0.9	0.9	-1.5	1.7	1.9	3.2	2.7	3.9	3.9						
3	0.726	1.140	0.883	1.020	1.009	0.997	0.955	1.008	1.018	1.015	1.122	1.004	0.743		
4	-0.6	-0.6	-1.1	-1.5	-1.5	-1.4	-0.8	0.8	0.8	0.8	0.8	0.8	3.6	3.6	
5	0.976	0.877	1.041	1.245	1.070	1.224	1.039	1.227	1.051	1.239	1.027	0.881	0.980		
6	-1.6	-1.8	-2.3	-1.5	-1.5	-2.8	-1.7	-1.0	0.2	-0.1	-1.4	0.8	1.0		
7	0.725	1.120	1.025	1.249	1.253	0.936	1.210	0.929	1.242	0.944	1.233	1.003	1.081	0.688	
8	1.4	-0.2	-1.3	-1.7	-1.4	-2.4	-1.7	-1.5	-0.1	-0.9	-1.4	-1.9	-1.9		
9	0.831	1.024	1.019	1.076	1.189	1.241	1.119	1.192	1.149	1.233	1.057	0.989	0.974	0.775	
10	3.0	1.2	-0.6	-0.9	-1.1	1.6	1.9	-1.0	-0.6	-1.2	-1.7	-2.0	-2.0	-1.9	
11	0.531	1.189	1.002	1.241	1.273	0.965	1.005	0.923	1.240	0.939	1.233	0.986	1.138	0.504	
12	3.8	1.4	-0.9	-0.4	-0.2	2.3	1.8	-0.4	0.3	0.5	-1.0	-0.8	-1.0	-0.2	
13	0.706	0.824	0.978	1.061	1.164	0.984	1.248	0.889	1.146	0.951	1.214	1.064	0.962	0.789	0.682
14	4.1	2.2	0.4	-0.3	0.4	3.3	4.3	3.9	1.3	1.1	0.5	-0.6	-0.4	0.0	1.8
15	0.531	1.192	1.019	1.229	1.236	0.939	0.986	0.952	1.258	0.952	1.247	1.003	1.176	0.520	
16	4.0	2.3	1.1	-1.9	-2.1	-1.2	0.8	2.6	1.9	0.5	-0.4	-0.2	0.7	1.8	
17	0.833	1.046	1.054	1.232	1.138	1.200	1.099	1.221	1.171	1.253	1.071	1.006	1.028	0.821	
18	4.5	4.4	3.5	-2.8	-2.7	-2.2	-1.4	-0.1	0.6	0.3	-1.1	-1.4	-1.4	2.0	2.0
19	0.746	1.173	1.071	1.224	1.227	0.938	1.236	0.955	1.260	0.941	1.226	1.002	1.144	0.728	
20	5.5	5.5	-2.4	-2.5	-2.3	-2.0	-1.1	-1.0	-1.4	-2.9	-3.0	-3.4	-3.4	2.0	
21	1.004	0.877	1.050	1.239	1.058	1.227	1.062	1.252	1.076	1.213	1.018	0.891	1.031		
22	2.5	-0.4	0.0	-0.9	-1.1	-2.8	-1.9	-1.8	-1.4	-3.8	-4.0	0.4	4.4		
23	0.724	1.137	0.680	1.009	1.009	0.991	0.973	1.000	1.015	1.016	0.876	1.162	0.775		
24	0.1	0.1	0.0	-1.4	-0.7	-1.9	-1.3	-2.2	-1.5	-1.3	-0.7	1.8	6.6		
25	0.726	0.981	1.114	1.012	1.161	0.806	1.154	0.992	1.128	1.021	0.765				
26	0.1	0.2	0.3	0.6	-0.8	-0.9	-2.1	-2.4	0.7	3.9	5.2				
27	0.715	0.809	0.517	0.687	0.513	0.708	0.719								
28	0.6	0.8	0.5	0.4	-1.2	-2.4	0.7								
29															
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MEAS
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FIGURE 5-17

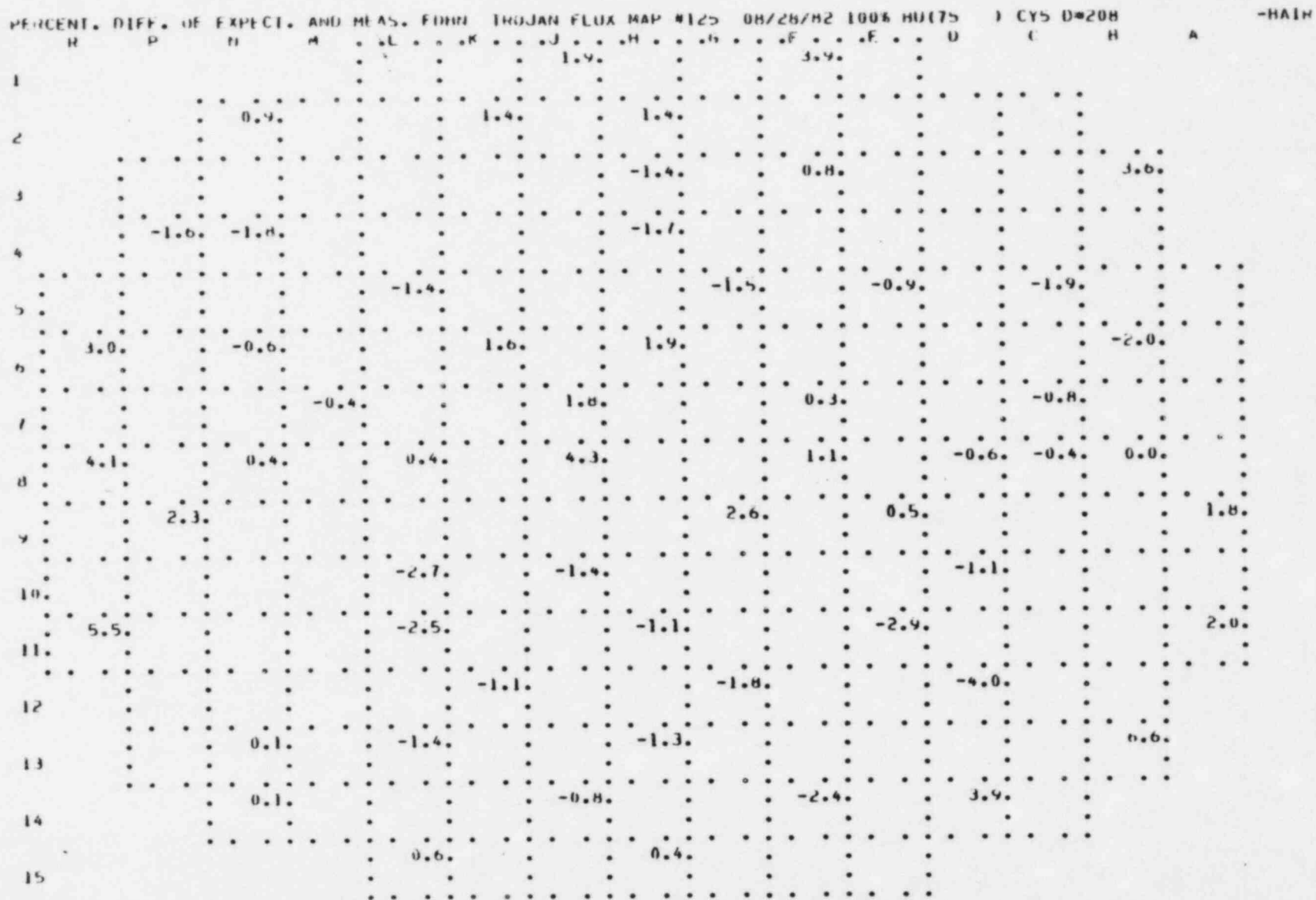


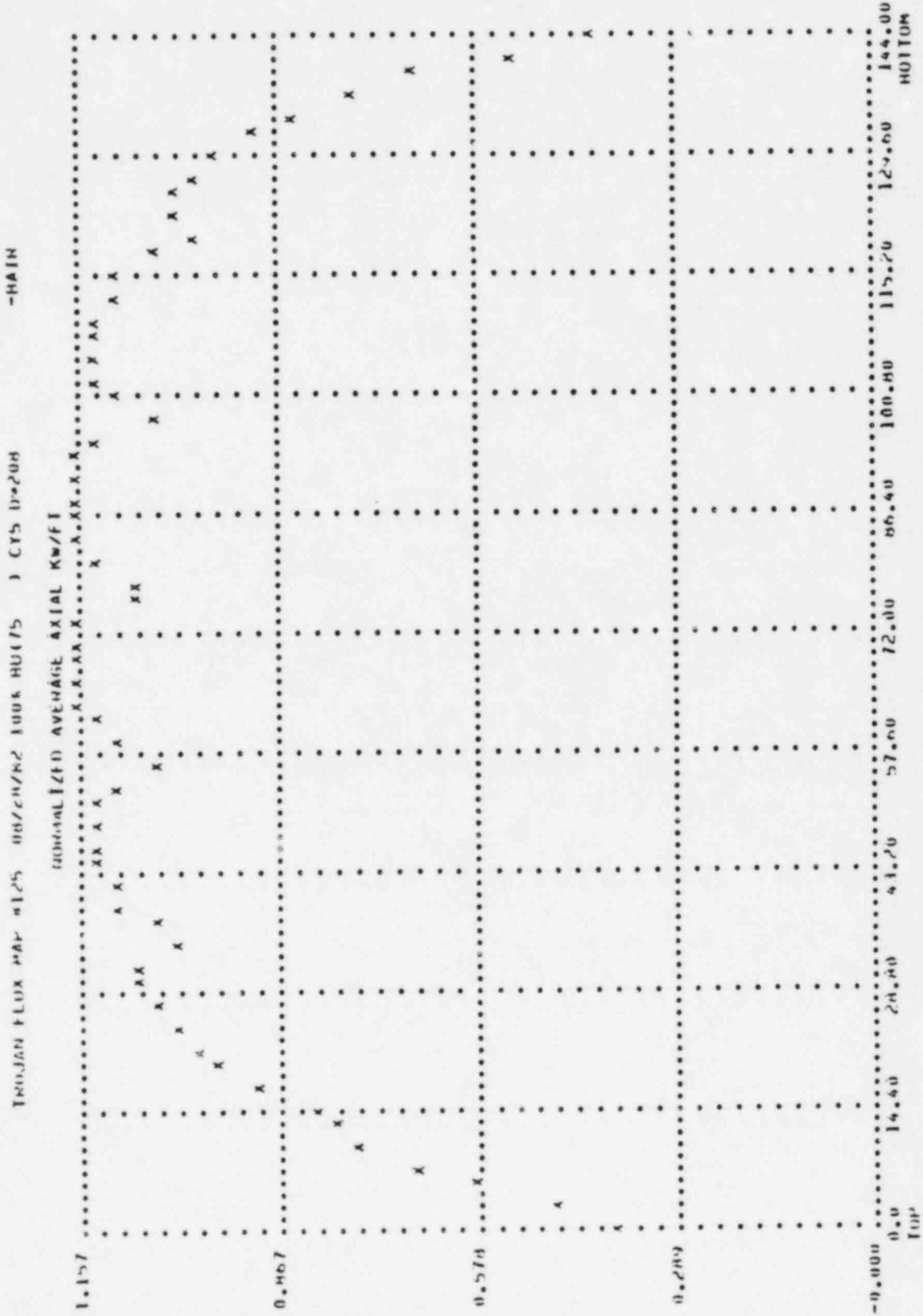
FIGURE 5-17

TROJAN FLUX MAP #125 08/28/82 1006 H0175 1 CY5 D=20H

-HAIR

POINT	SOURCE	NUCLEAR FX	POINT	SOURCE	NUCLEAR FX
1	395 F 9M	1.5095	31	210 P 7LE	1.3633
2	301 K 7ED	1.5716	32	452 C 3EL	1.3417
3	301 K 7ED	1.5473	33	333 J14ED	1.3457
4	301 K 7ED	1.5247	34	210 P 7LE	1.3577
5	301 K 7ED	1.4989	35	210 P 7LE	1.3583
6	323 J 8DE	1.4803	36	452 C 3EL	1.3659
7	301 K 7ED	1.4684	37	452 C 3EL	1.3739
8	323 J 8DE	1.4448	38	452 C 3EL	1.3809
9	323 J 8DF	1.4433	39	452 C 3EL	1.3779
10	323 J 8DF	1.4334	40	452 C 3EL	1.3867
11	323 J 8DF	1.4226	41	452 C 3EL	1.4160
12	323 J 8DF	1.4160	42	333 J14ED	1.3774
13	323 J 8DE	1.4151	43	452 C 3EL	1.3715
14	323 J 8DE	1.4112	44	452 C 3EL	1.3770
15	323 J 8DE	1.4330	45	452 C 3EL	1.3798
16	301 K 7ED	1.3779	46	452 C 3EL	1.3773
17	323 J 8DE	1.3968	47	452 C 3EL	1.3800
18	323 J 8DE	1.3944	48	452 C 3EL	1.3758
19	323 J 8DE	1.3849	49	452 C 3EL	1.4130
20	323 J 8DE	1.3763	50	333 J14ED	1.4381
21	323 J 8DE	1.3726	51	333 J14ED	1.3622
22	323 J 8DF	1.3694	52	323 J 8DE	1.3632
23	323 J 8DE	1.3724	53	409 F 4NM	1.3764
24	333 J14ED	1.3843	54	409 F 4NM	1.3878
25	210 P 7LE	1.3571	55	409 E 4NM	1.4151
26	214 P 9LM	1.3624	56	409 F 4NM	1.4527
27	210 P 7LE	1.3613	57	409 F 4NM	1.4253
28	210 P 7LF	1.3638	58	409 F 4NM	1.5561
29	210 P 7LF	1.3639	59	317 J 4FE	1.6824
30	210 P 7LF	1.3613	60	333 J14ED	1.9466

NOTE= VALUES ARE BEST ESTIMATE AND DO NOT INCLUDE ENGINEERING OR NUCLEAR UNCERTAINTY.



6.0 CONCLUSIONS

The Cycle 5 startup test results were satisfactory and represent a significant improvement over the Cycle 3 results and continued improvement over Cycle 4 results. The more accurate treatment of individual fuel assembly burnups, along with other improvements to the nuclear design methods, have reduced substantially the amount of nuclear design prediction error.

The startup tests also showed that the modified stainless steel pin fuel assemblies, with and without 2 x 8 partial grids, do not have a significant impact on core power distribution and that they can be accounted for in normal fuel management schemes.

ATTACHMENT A

TROJAN EOC-4 ONSITE EXAMINATION

1.0 Summary of Examinations Performed

1.1 Binocular Examinations

Binocular examinations were performed on 180 assemblies during core unload to assess the overall fuel condition. Examinations were performed on 9 Region 1 assemblies, 1 Region 3 assembly, 62 Region 4 assemblies, 64 Region 5 assemblies and 44 Region 6 assemblies.

1.2 Leak Testing

- 1.2.1 Leak testing (sipping) was performed by Nuclear Assurance Corporation on 164 assemblies from Regions 1, 4, 5 and 6. 12 assemblies (A09, A45, D07, D58, E10, E16, E41, E43, F02, F11, F21, F30) failed leak testing.

1.3 TV Visual Examinations

1.3.1 Baffle Joint Fuel Assemblies

A total of 24 baffle joint assemblies from Region 6 were TV examined at high and low magnification on the critical (baffle joint) and adjacent faces respectively. Of these assemblies, 12 were adjacent to corner injection joints, 8 were adjacent to center injection joints and 4 were adjacent to both types of joints.

1.3.2 Fuel Assemblies Located at Baffle Joints in Cycle 3

12 Region 5 assemblies that were located adjacent to baffle joints in the previous Cycle 3 were TV examined at high and low magnification on the critical and adjacent faces respectively.

1.3.3 Other Assemblies (Non-Baffle Joint)

- a) Assembly F33 face 2 which was adjacent to F/A F22 (suspect sipping failure).
- b) Assemblies F30, E10, E16, E41, D07, D58, A09 and A45 all classified as sipping failures.
- c) Assemblies A25, A49 and A65 heavily crudded from Cycle 1.
- d) Assembly A64 used as channel spacing standard.

1.4 Channel Spacing Examination

38 fuel assemblies from Regions 5 and 6 were probed for rod spacing/movement measurements. The 16 Region 5 assemblies probed were originally adjacent to baffle joints in Cycle 3. 8 were adjacent to center injection

joints, 4 were adjacent to inside corner joints and 4 were adjacent to both center injection and inside corner joints. 22 Region 6 assemblies, consisting of 6 assemblies adjacent to center injection joints, 12 assemblies adjacent to inside corner joints and 4 assemblies adjacent to both types of baffle joints, were probed.

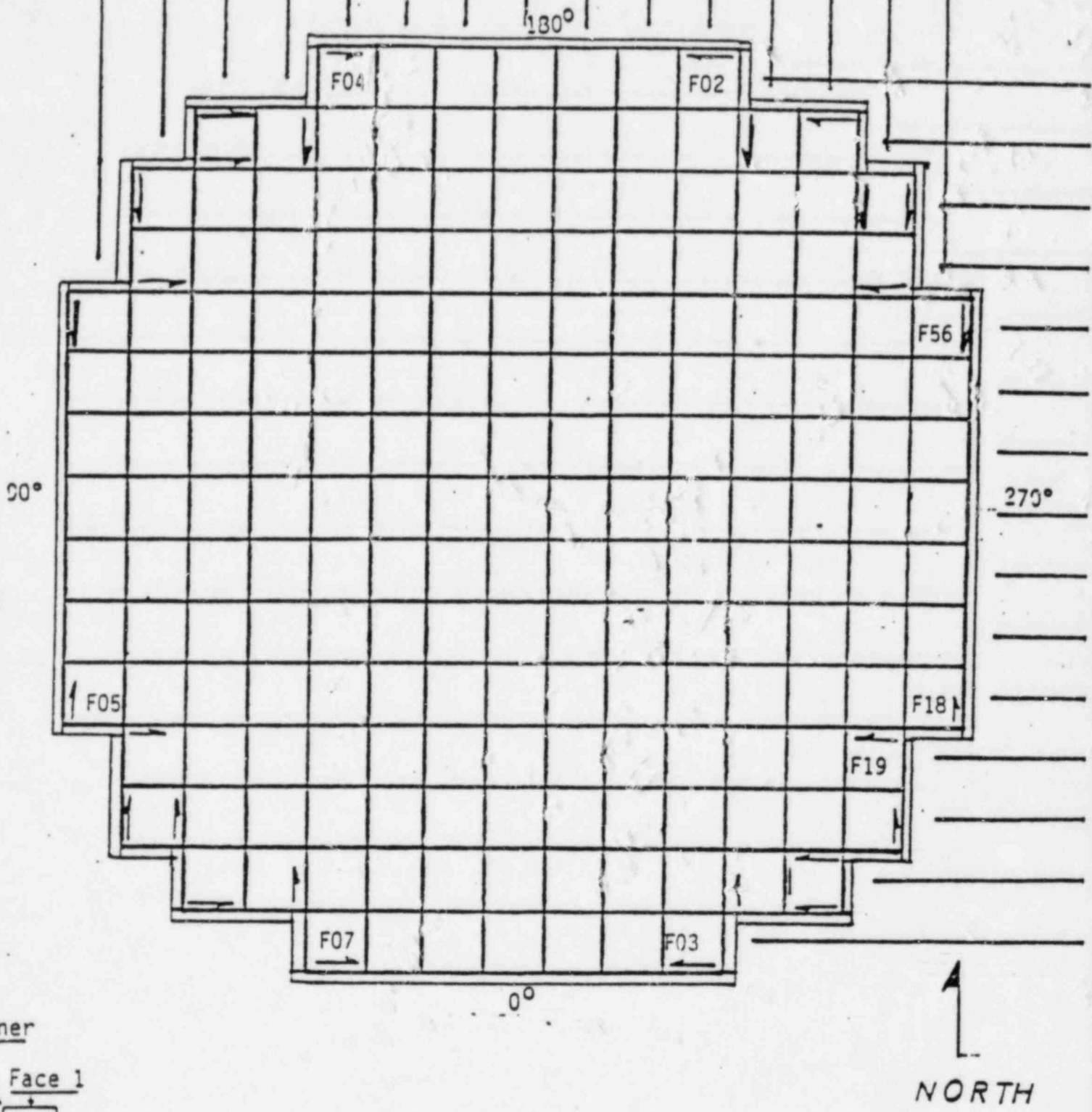
2.0 Examination Results

- 2.1 Of the 24 Region 6 baffle joint assemblies which were examined, 7 inside corner joint assemblies (F56, F18, F07, F05, F04, F03, F02) and 1 center injection joint assembly F19 exhibited severe fuel rod damage. The damaged area for 7 inside corner joint assemblies occurred between grid levels 5 to 8. For F19, the center injection joint assembly, the damage was between grids 1 and 2. Figure 1 shows the core locations for these assemblies. The remaining 16 baffle joint assemblies had no apparent baffle jetting related visual damage. However, F21 had one interior rod (#12 in the second row on non-baffle face 4) which was suspect since the lower end plug was slightly cocked and debris was sighted on the bottom nozzle in that area.
- 2.2 12 assemblies from Region 5 which were located adjacent to baffle joints in Cycle 3 (prior cycle) were TV examined on their critical faces (see Figure 2). Assembly E43, a sipping failure, was shown by TV examinations to have failed rod 2 and a suspected failure of rod 3. Assembly E47 exhibited a reduction in channel spacing during probing at the bottom of grid 2 between rods 1 and 2 on face 4. This channel reduction was present at EOC-3 and there was no change in this phenomenon after Cycle 4. Two assemblies E05 and E15 each containing three stainless steel rods were in good condition with the exception of light baffle spray on the grids on face for E15. Two additional assemblies E53 and E56 exhibited a reduction in channel spacing at grid span 1 on their critical face. For the remaining six assemblies (E02, E04, E08, E12, E29 and E51), the only anomalies detected were light baffle marks on the critical faces for assemblies E04, E29 and E51.
- 2.3 F33 was examined on face 2 for visual anomalies because this assembly was located adjacent to F22. Assembly F22 was classified as a suspect sipping failure early in the site exam but was later classified as a non-leaker by PGE. The TV examination of face 2 revealed no anomalies.
- 2.4 Assembly F30 (sipping failure) was TV examined at low magnification on all four faces and no anomalies were noted.
- 2.5 Three assemblies E10, E16 and E41, all classified as sipping failures, were TV examined at low magnification on all four faces for visual defects. Assembly E10 had visible pellet chips on face 1 and metallic debris on the bottom nozzle of face 2. Fuel assembly E16 had a failed peripheral fuel rod (#15) on face 1. No anomalies were observed on assembly E41.

- 2.6 Assemblies D07 and D58, slated for reuse in Cycle 5, failed leak testing. Both assemblies were TV examined at low magnification on all four faces. The only anomaly detected was F/Rs 2 and 3 on face 3 of D58 which were approximately 1/4" from the bottom nozzle. These assemblies were previously in baffle locations in Cycle 2 but were not located at joint locations.
- 2.7 Assemblies A09 and A45 which were stored in the Spent Fuel Pit (SFP) were designated for Cycle 5 reinsertion prior to failing leak testing. Both assemblies were TV examined on all faces at low magnification and no visual anomalies were found except for some non-metallic debris (approx. 1/4" in length) on face 3 above grid 8 on assembly A09.
- 2.8 A65 from Region 1 was examined at the request of PGE. The assembly was TV examined at low magnification on all four faces. Other than being heavily crudded from Cycle 1, no visual anomaly was detected.
- 2.9 Assembly A64 which had been discharged at EOC-1 and was stored in the SFP was used as probe spacing standard at the beginning of the site exam. This assembly was TV examined at high and low magnification on faces 1 and 2. Some non-metallic debris was detected in the top nozzle on face 1. Other than being heavily crudded from Cycle 1, no apparent visual anomalies were found.

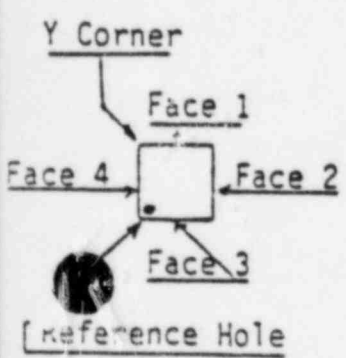
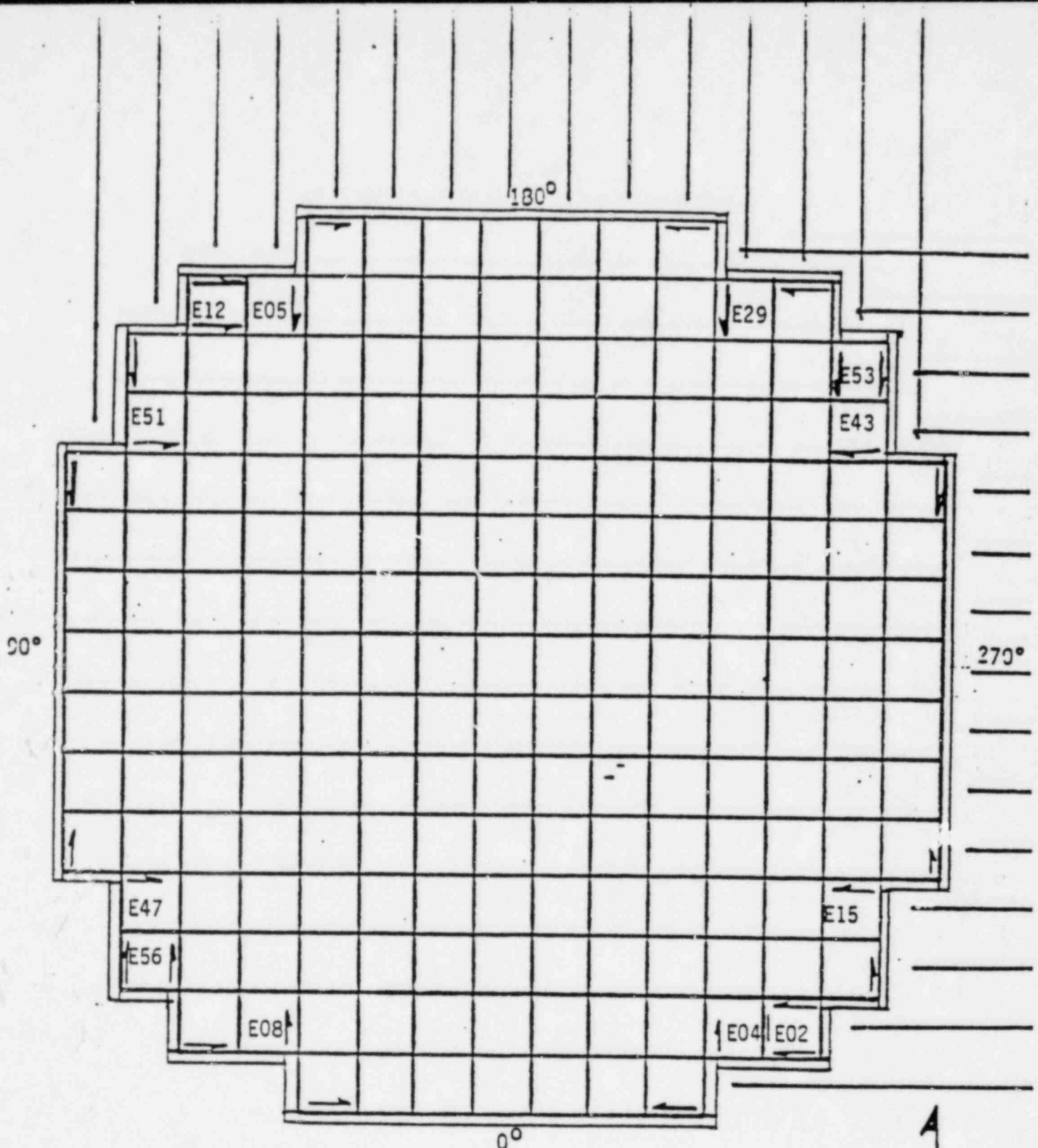
3.0 Channel Spacing

A probe was used for examining the spacing of fuel rods at the grid contact area. The purpose of the examination was to determine possible side way movement of fuel rods due to fretting. This examination was undertaken as a means of assessing the condition of the unfailed fuel. At the request of PGE, 8 damaged assemblies (F02, F03, F04, F05, F07, F18, F56) were also probed. The examination of 30 assemblies resulted in detection of no previously unsuspected failed assemblies.



Arrows indicate direction water would flow through a leaking baffle joint

Figure 1 Locations of the Eight Damaged Baffle Joint Assemblies in Trojan at EOC-4



Arrows indicate direction water would flow through a leaking baffle joint

Figure 2 Locations of the Twelve "E" Assemblies Adjacent to Baffle Joints in EOC-3