

APPENDIX B

U. S. NUCLEAR REGULATORY COMMISSION
REGION IV

Report: 50-267/82-24

License: DPR-34

Docket: 50-267

Category: 5

Licensee: Public Service Company of Colorado
P. O. Box 840
Denver, Colorado 80201

Facility Name: Fort St. Vrain Nuclear Generating Station

Inspection at: Fort St. Vrain Site, Platteville, Colorado

Inspection Conducted: October 1-31, 1982

Inspectors: M. W. Dickerson 11-4-82
M. W. Dickerson, Senior Resident Reactor Inspector Date

G. L. Plumlee III 11-4-82
G. L. Plumlee III, Resident Reactor Inspector Date

Approved: T. F. Westerman 11-9-82
T. F. Westerman, Chief, Reactor Project Section A Date

Inspection Summary

Inspection Conducted October 1-31, 1982 (Report: 50-267/82-24)

Areas Inspected: Routine, announced inspection of Operational Safety Verification; Surveillance; Maintenance; Follow Up of Previous Inspection Findings; and Review of Periodic and Special Reports. The inspection involved 135 inspector-hours onsite by two NRC inspectors.

Results: Within the five areas inspected, two violations were identified. (Failure to follow procedures, paragraphs 2 and 3)

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DETAILS

1. Persons Contacted

M. Block, Superintendent of Operations
T. Borst, Radiation Protection Manager
R. Burchfield, Results Engineering Supervisor
W. Craine, Superintendent of Maintenance
D. Evans, Shift Supervisor
M. Ferris, Supervisor QA Technical Support
W. Franek, Nuclear Site Engineering Manager
W. Franklin, Shift Supervisor
C. Fuller, Technical Services Engineering Supervisor
J. Gahm, QA Manager
J. Hak, Shift Supervisor
E. Hill, Operations Manager
J. Jackson, QA/QC Supervisor
J. Liebelt, Senior Maintenance Supervisor
M. McBride, Technical/Administrative Services Manager
F. Novachek, Technical Advisor
H. O'Hagan, Shift Supervisor
T. Orlin, Superintendent QA Services
G. Reigel, Shift Supervisor
L. Singleton, Superintendent Operations QA
J. Van Dyke, Shift Supervisor
D. Warembourg, Manager Nuclear Production
R. Webb, Maintenance Supervisor
W. Woodard, Health Physicist

The NRC inspectors also contacted other plant personnel including reactor operators, maintenance men, electricians, technicians, and administrative personnel.

2. Operational Safety Verification

The NRC inspector reviewed licensee activities to ascertain that the facility is being operated safely and in conformance with regulatory requirements, and the licensee's management control system is effectively discharging its responsibilities for continued safe operation. The review was conducted by direct observation of activities, tours of the facility, interviews and discussions with licensee personnel, independent verification of safety system status and limiting conditions for operations, and review of facility records.

Logs and records reviewed included:

- . Shift Supervisor Logs
- . Reactor Operator Logs
- . Equipment Operator Logs

- . Auxiliary Operator Logs
- . Technical Specification Compliance Logs
- . Operations Order Book
- . Operations Deviations Reports
- . Clearance Log
- . Temporary Configuration Reports
- . Plant Trouble Reports

During tours of accessible areas, particular attention was directed to the following:

- . Monitoring Instrumentation
- . Radiation Controls
- . Housekeeping
- . Fluid Leaks
- . Piping Vibrations
- . Hanger/Seismic Restraints
- . Clearance Tags
- . Fire Hazards
- . Control Room Manning
- . Annunciators

The operability of selected systems or portions of systems were verified by walkdown of the accessible portions. The NRC inspector verified the operability of, "System 82 - Instrument and Service Air System." The problems identified during the system walkdown are being corrected by the licensee.

Procedures were also reviewed and implementation observed for Gaseous Effluent Releases 705, 709, and Liquid Effluent Release 602. The releases appeared to have been made in a satisfactory manner.

On October 12, 1982, while performing a tour of the reactor building, the NRC inspector noted that the main clearance card for Clearance 4039 was on the supply power breaker for the control rod drive in Region 27 and the breaker was in the "ON" position. From a review of the clearance form for Clearance 4039, the NRC inspector determined that the breaker was not positioned in the "OFF" position as required by Clearance 4039. The clearance had been issued to the refueling supervisor on October 8, 1982, and the breaker position had apparently been changed as required for maintenance on the control rod drive in Region 27. However, the standard clearance point form as written did not allow repositioning of the breaker.

On October 13, 1982, at 2:20 p.m. MDT, the NRC inspector performed a walkdown of Clearance 4039. The NRC inspector determined that contrary to the Standard Clearance Point Form 4039, Valve V-11277 was open and not tagged as required, and Valve V-11127 was not tagged as required. The NRC inspector determined that Valve V-11227 had been closed and tagged, instead of V-11277. An equipment operator verified the NRC inspector's findings and repositioned and tagged the valves in accordance with the standard clearance points form.

The licensee was informed of the failure to follow the standard clearance point form requirements and that the failure to comply with procedures which are Technical Specification requirements is considered a violation.
(50-267/8224-01)

The NRC inspector had no further questions in this area.

3. Surveillance (Monthly)

The NRC inspector reviewed all aspects of surveillance testing involving safety-related systems. The review included observation and review relative to Technical Specification requirements. The surveillance tests reviewed and observed were:

- SR 5.8.1abc-M Radioactive Gaseous Effluent System Test (Releases 705 and 709)
- SR 5.8.2bc-M Radioactive Liquid Effluent System Instrumentation Functional Test (Release 602)
- SR 5.6.1b-SA Loss of Outside Power and Turbine Trip
- SR 5.4.1.1.5.d-R/5.4.1.4.3d-R Wide Range Power Channel Calibration
- SR 5.4.1.1.9.c-R Primary Coolant Pressure Scram Calibration
- SR 5.3.6-M Instrument Air System Functional Test
- SR 5.4.1.1.15.b-M High Reactor Building Temperature (Pipe Cavity) Scram Test

On October 22, 1982, the NRC inspector observed performance of SR 5.3.6-M. The NRC inspector determined that the operator (tender) performing the surveillance was not checking off the surveillance steps upon completion or signing/dating the page upon completion as required by Administrative Procedure P-4. Upon questioning the operator, the NRC inspector was informed that the reactor operator, who was in radio contact with the tender, had the original surveillance form and that the tender had a copy to aid in performing the surveillance. The NRC inspector felt that only the original surveillance should be used. The person with the original should be directing performance of the surveillance. The NRC inspector recommended that the tender check each step upon completion to aid the tender during performance of the surveillance. Upon completion of the tender's performance review, the NRC inspector went to the control room and reviewed the original copy of SR 5.3.6-M controlled by the reactor operator. At this time, performance of the surveillance steps was complete. From a review of the original, the NRC inspector determined that the appropriate blanks provided in the surveillance had not been completed upon step completion as required by Procedure P-4. The original copy of the surveillance at the time of surveillance steps completion had none of its appropriate blank spaces for step and page completion filled in.

The above was discussed with the licensee and the licensee was informed that the failure to follow procedures which are Technical Specification requirements is considered a violation. (50-267/8224-02)

The NRC inspector had no further questions in this area.

4. Maintenance (Monthly)

The NRC inspector reviewed records and observed work in progress to ascertain that the following maintenance activities were being conducted in accordance with approved procedures, Technical Specifications, and appropriate Codes and Standards. The following maintenance activities were reviewed and observed:

CN 1404/CWP 82-48 Steam/Water Drain High Temperature Alarms and Steam/Water Drain Valve Control Reference to Steam Turbine Cavity Pressure Modifications

PTR 10-97 HV-2301 Repair in accordance with MP 23-2, "Removal and Installation of HV 2301 and HV 2302," and MP-4, "Maintenance/Repair of Velan Valves"

- PTR 10-109 Repair Leaking Sulzer Valves I - J - K - L - M on Steam Generator Module B-2-5. Repairs made in accordance with Steam Generator Maintenance Manual GA-10377
- PTR 8-52 Reserve Shutdown Region 27 Maintenance in accordance with MP 11-4, "In Core Removal of Reserve Shutdown Material"
- PTR 10-340 Remove Reserve Shutdown Material from Control Rod Drive 27 located in Equipment Storage Well Number 1
- PTR 10-339 Prepare Control Rod Drive 2 for use in Region 27 in accordance with MP 12-7, "Procedure to Move CRD Assembly from Equipment Storage Well to Work Stand and Back," and MP 12-6, "Maintenance and Repair of CRD and Orificing Assemblies"

On October 7, 1982, during the review of work being performed in accordance with PTR 10-97, the NRC inspector determined that the incorrect vendor's drawing was being used for HV 2301. Due to the difference in material specifications between this drawing and the correct drawing, the NRC inspector also noted that the weld data sheet referenced the wrong weld procedure for one of the welds to be made on the valve bonnet. The weld data sheet had been prepared by a QA/QC inspector designee as authorized by the Fort St. Vrain Welding Manual. The QA/QC inspector stated that the weld procedure to be used had been determined by using the vendor's diagram located at the work site and from the assumption that the two base metals were the same, where, in fact, they were dissimilar metals. The weld had not occurred at the time the NRC inspector determined the problem.

The licensee was informed of the above problems. The incorrect vendor's drawing was immediately replaced with the correct drawing and a new weld data sheet was prepared referencing the required bimetallic weld procedure. The licensee stated that the QA/QC inspector would be reinstructed in the correct procedure for weld data sheet preparation.

The NRC inspector had no further questions in this area.

5. Reserve Shutdown System

As a follow up to the previously reported reserve shutdown material problem in Region 27 (NRC Report 50-267/82-23), the following information is provided.

During a forced outage which began on September 30, 1982, the licensee made the decision to remove the Reserve Shutdown (RSD) material from Region 27. The RSD material was removed from Region 27 on October 18, 1982,

in accordance with the licensee's Procedure MP 11-4, "In Core Removal of Reserve Shutdown Material." A repaired control rod drive was installed in Region 27 and made operational on October 24, 1982.

The NRC inspector had no further questions in this area.

6. Report Reviews

The NRC inspector reviewed the following reports for content, reporting requirements, and adequacy:

Monthly Operating Information Report, September 1982

Monthly Operations Report, September 1982

Twenty-fourth Startup Report, June 23, 1982, through September 22, 1982

No violations or deviations were identified.

7. Exit Interview

Exit interviews were conducted at the end of various segments of this inspection with Mr. D. Warembourg, Manager, Nuclear Production, and/or other members of the Public Service Company staff. At the interviews, the inspectors discussed the findings indicated in the previous paragraphs. The licensee acknowledged these findings.

INSPECTOR'S REPORT
Office of Inspection and Enforcement

DICKERSON, M.W.

REVIEWER
T.F. WESTERMAN

INSPECTORS
G.L. PLUMLEE, III

LICENSEE/VENDOR	TRANSACTION TYPE	DOCKET NO. (8 digits) OR LICENSE NO. (BY PRODUCT) (13 digits)	REPORT		NEXT INSP. DATE	
			NO.	SEQ.	MO.	YR.
PUBLIC SERVICE Co. of Colo. FORT ST. VRAIN	I - INSERT M - MODIFY D - DELETE R - REPLACE	05000267	8224	A		

PERIOD OF INVESTIGATION/INSPECTION							INSPECTION PERFORMED BY							ORGANIZATION CODE OF REGION/HQ CONDUCTING ACTIVITY (See 15MC 0530 Manpower Reporting - Weekly Manpower Reporting for code)		
FROM			TO				1 - REGIONAL OFFICE STAFF			OTHER				REGION	DIVISION	BRANCH
MO.	DAY	YR.	MO.	DAY	YR.	<input checked="" type="checkbox"/>										
10	01	82	10	03	82									4	C	A
20		25	26		31									33	34	35

REGIONAL ACTION (Check one box only)		TYPE OF ACTIVITY CONDUCTED (Check one box only)														
<input checked="" type="checkbox"/> 1 - NRC FORM 591	<input checked="" type="checkbox"/> 2 - REGIONAL OFFICE LETTER	<input checked="" type="checkbox"/> 02 - SAFETY	<input type="checkbox"/> 03 - INCIDENT	<input type="checkbox"/> 04 - ENFORCEMENT	<input type="checkbox"/> 05 - MGMT. AUDIT	<input type="checkbox"/> 06 - MGMT. VISIT	<input type="checkbox"/> 07 - SPECIAL	<input type="checkbox"/> 08 - VENDOR	<input type="checkbox"/> 09 - MAT ACCT.	<input type="checkbox"/> 10 - PLANT SEC.	<input type="checkbox"/> 11 - INVENT. VER.	<input type="checkbox"/> 12 - SHIPMENT/EXPORT	<input type="checkbox"/> 13 - IMPORT	<input type="checkbox"/> 14 - INQUIRY	<input type="checkbox"/> 15 - INVESTIGATION	

INSPECTION/INVESTIGATION FINDINGS (Check one box only)				TOTAL NUMBER OF VIOLATIONS AND DEVIATIONS				ENFORCEMENT CONFERENCE HELD				REPORT CONTAIN 2,790 INFORMATION				LETTER OF REPORT TRANSMITTAL DATE					
A	B	C	D	A	B	C	D	A	B	C	D	A	B	C	D	NRC FORM 591 OR REG. LETTER IS SUED			REPORT SENT TO HQ. FOR ACTION		
<input checked="" type="checkbox"/>				0	2											1	1	1	7	8	2

MODULE INFORMATION														MODULE INFORMATION																							
REC. ORD.		MODULE NUMBER INSP.					MODULE REQ. FOLLOWUP							REC. ORD.		MODULE NUMBER INSP.					MODULE REQ. FOLLOWUP																
TYPE	NUMBER	PHASE	MANUAL	CHAPTER	PROCEDURE NUMBER	LEVEL	SEQ.	PRIORITY	DIRECT INSP. EFFORT IN STAFF HOURS EXPENDED THIS INSPECTION	PERCENTAGE COMPLETED	TO DATE	STATUS	PHASE	MANUAL	CHAPTER	PROCEDURE NUMBER	LEVEL	TYPE	NUMBER	PHASE	MANUAL	CHAPTER	PROCEDURE NUMBER	LEVEL	SEQ.	PRIORITY	DIRECT INSP. EFFORT IN STAFF HOURS EXPENDED THIS INSPECTION	PERCENTAGE COMPLETED	TO DATE	STATUS	PHASE	MANUAL	CHAPTER	PROCEDURE NUMBER	LEVEL		
B	53	07	03	B	A	0	01						B	57	17	09	A	0	37	80	C																
B	56	17	26	A	0	15	100	C					B	59	07	13	A	0	01	100	C																
B	56	27	03	A	0	15	100	C					B	59	27	06	A	0	27																		
B	57	17	07	A	0	37	80	C					B				A																				

* CIRCLE SEQUENCE IF VIOLATION OR DEVIATION

**INSPECTOR'S REPORT
(Continuation)**

Office of Inspection and Enforcement

DOCKET NO. (8 digits) OR LICENSE
NO. (BY PRODUCT) (13 digits)

05000267

REPORT

NO. SEQ.

8224

A
B
C
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MODULE NUMBER

5161726

I.E.

VIOLATION SEVERITY OR DEVIATION

1 2 3 4 5 6

X

SITE RELATED

A C
B D

VIOLATION OR DEVIATION (Enter up to 2400 characters for each item. If the text exceeds this number, it will be necessary to paraphrase. Limit lines to 50 characters each.)

Contrary to the requirements of Administrative procedure P-4 The NRC inspector determined that Surveillance SR 5.3.6-M did not have its appropriate blanks filled in upon step completion & the bottom of each page had not been signed/dated.

INSPECTOR'S REPORT
(Continuation)
Office of Inspection and Enforcement

DOCKET NO. (8 digits) OR LICENSE NO. (BY PRODUCT) (13 digits)		REPORT NO.		MODULE NUMBER	I.E.		
05000267		8224		57117091			
				VIOLATION SEVERITY OR DEVIATION D		SITE RELATED	
				1 2 3 4 5 6		A C	
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				27		28	

VIOLATION OR DEVIATION (Enter up to 2400 characters for each item. If the text exceeds this number, it will be necessary to paraphrase. Limit lines to 50 characters each.)

1.
2. Contrary To the requirements of Administrative procedure
3. P-2, on October 12, 1982, while standard Clearance
4. point Form 4039 was still in effect, the NRC inspector
5. determined that the main power breaker for Region 27
6. Control rod drive was not in the "OFF" position as
7. required. ON October 13, 1982 it was determined that
8. Valve V-11277 was open & not tagged & valve V11127
9. was not tagged also contrary to Form 4039 requirements
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