UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

DOCKET NO. 50-255

PALISADES PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 71
License No. DPR-20

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Consumers Power Company (the licensee) dated July 6, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-20 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B (Environmental Protection Plan), as revised through Amendment No. 71 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 1, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 71

PROVISIONAL OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Revise Appendix A Technical Specifications by removing the following pages and by inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines indicating the area of change.

PAGES
4-28
4-29

4.5 CONTAINMENT TESTS (Contd)

(2) If repairs are not completed and conformance to the acceptance criterion of 4.5.2.b is not demonstrated within 48 hours, the reactor shall be shut down and depressurized until repairs are effected and the local leakage meets this acceptance criterion.

d. Test Frequency

(1) Individual penetrations and containment isolation valves shall be leak rate tested at a frequency of at least every six months prior to the first postoperational integrated leak rate test and at a frequency of at least every refueling thereafter, not exceeding a 2-year interval, except as specified in (a) and (b) below:

(a) The containment equipment hatch and the fuel transfer tube shall be tested at each refueling shutdown or after each time used, if that be sooner.

(b) The personnel air lock seals shall be tested at six-month intervals, except when the air locks are not opened during the interval. In that case, the test is to be performed after each opening, except that no test interval is to exceed twelve months.

(2) Each three months the isolation valves must be stroked to the position required to fulfill their safety function unless it is established that such operation is not practical during plant operation. The latter valves shall be full-stroked during each cold shutdown.

4.5.3 Recirculation Heat Removal Systems

a. Test

(1) The portion of the shutdown cooling system that is outside the containment shall be tested either by use in normal operation or hydrostatically tested at 255 psig at the interval specified in 6.15.

(2) Piping from valves CV-3029 and CV-3030 to the discharge of the safety injection pumps and containment spray pumps shall be hydrostatically tested at no less than 100 psig at the interval specified in 6.15.

4.5 CONTAINMENT TESTS (Contd)

(3) Visual inspection shall be made for excessive leakage from components of the system at the interval specified in 6.15. Any significant leakage shall be measured by collection and weighing or by another equivalent method.

b. Acceptance Criterion

The maximum allowable leakage from the recirculation heat removal systems' components (which include valve stems, flanges and pump seals) shall not exceed 0.2 gallon per minute under the normal hydrostatic head from the SIRW tank (approximately 44 psig).

c. Corrective Action

Repairs shall be made as required to maintain leakage within the acceptance criterion of 4.5.3.b.

4.5.4 Surveillance for Prestressing System

a. Tendon inspection shall be accomplished in accordance with the following schedule:

1. One year after initial structural integrity test.
2. Three years after initial structural integrity test.
3. Five years after initial structural integrity test.
4. At five-year intervals thereafter for the life of the plant.

b. Surveillance tendons for the one-year inspection shall be the nine designated surveillance tendons plus V-104 and V-200. In addition, 15 vertical tendons shall be tested for lift-off forces only.

c. For the three-year inspection, the surveillance tendons shall consist of the 11 tendons inspected during the one-year test plus an additional 10 vertical tendons to be tested for lift-off force only. The additional 10 tendons shall be selected from tendons other than those tendons tested for lift-off force during the one-year inspection.