



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 16, 1994

Docket Nos. 50-259, 50-260  
and 50-296

LICENSEE: Tennessee Valley Authority (TVA)  
FACILITY: Browns Ferry Nuclear Plant, Units 1, 2, and 3  
SUBJECT: SUMMARY OF THE MAY 25, 1994 MEETING WITH THE TENNESSEE VALLEY  
AUTHORITY REGARDING A PROPOSED AMENDMENT TO THE BROWNS FERRY  
NUCLEAR PLANT, UNITS 1, 2, AND 3 TECHNICAL SPECIFICATIONS

A meeting was held on May 25, 1994 in Rockville, Maryland, between representatives of the NRC staff and the Tennessee Valley Authority (TVA). This meeting was held to review and discuss TVA's March 30, 1994 request to amend the Technical Specifications (TS) for the Browns Ferry Nuclear Plant (BFN) Units 1, 2, and 3. Meeting attendees are listed in Enclosure 1. A copy of the handout provided by TVA is Enclosure 2.

TVA's March 30, 1994 request to amend the BFN TS has six components:

- A. BFN Unit 3 analog transmitter/trip system (ATTS),
- B. BFN Units 1 and 3 reactor vessel water level safety limit and Level 1 low reactor vessel water level setpoint,
- C. BFN Unit 2 instrument identifiers,
- D. BFN Unit 2 ATTS calibration frequencies and functional test descriptions,
- E. BFN Units 1, 2, and 3 suppression chamber-reactor building vacuum breaker calibration frequency, and
- F. editorial changes, correcting capitalization of terms.

TVA stated that when the proposed amendment is approved, the BFN Unit 1, 2, and 3 TS will be the same for these items.

Item A is similar to a previously-issued BFN Unit 2 TS. The staff commented that TVA should take care to ensure all components are within an appropriate maintenance program.

Item B is also similar to a previously-issued BFN Unit 2 TS. The staff noted that during the BFN Unit 2 review, there had been a concern about the rigor in the reference to "vessel zero" for the limits and setpoints. TVA was cautioned to ensure that a valid reference is available. TVA indicated that it would make these references available to the staff if need be.

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TVA's requested changes are currently under review by the staff, with issuance of the amendment scheduled for April 1995.

Original signed by

Joseph F. Williams, Project Manager  
Project Directorate II-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Attendance List
2. TVA Handout

cc w/enclosures:

See next page

OFF	PDII-4/LA	PDII-4/PM	PDII-4/PM	PDII-4/D
NAME	BClayton	JWilliams	DTrimble	FHebdon
DATE	6/3/94	6/15/94	6/16/94	6/16/94

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15-B-18

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8-H-3

ACRS(10)

L. Plisco

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R11

ATTENDEES

TVA/NRC MEETING

MAY 24, 1994

<u>NAME</u>	<u>ORGANIZATION</u>
Joe Williams	NRR/PD II-4
Dave Trimble	NRR/PD II-4
Fred Paulitz	NRR/HICB
Pedro Salas	TVA/BFN Licensing Manager
Steve Kane	TVA Licensing
Barry Hargis	TVA Browns Ferry
Lynn Chandler	TVA/Browns Ferry Engineering

# **Proposed Technical Specification No. 318**

**Analog Transmitter/Trip System,  
Level 1 Reactor Water Level Setpoints,  
and Various Calibration Frequencies**



**TVA/NRC Meeting  
May 25, 1994**

## PROPOSED TECHNICAL SPECIFICATION NO. 318

### AGENDA

- |      |   |          |
|------|---|----------|
| I.   | OPENING REMARKS                                       | P. SALAS |
| II.  | SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION | S. KANE  |
| III. | LAYOUT OF PROPOSED TECHNICAL SPECIFICATION            | S. KANE  |
| IV.  | SAFETY ANALYSIS OF PROPOSED CHANGES                   | S. KANE  |

## I. OPENING REMARKS

- TVA RECOGNIZES THAT PROCESSING TECHNICAL SPECIFICATION 318 WILL BE ADMINISTRATIVELY BURDENSOME
- TVA DOES NOT CONSIDER CONTENTS OF TECHNICAL SPECIFICATION 318 TO BE CONTROVERSIAL
- PURPOSE OF MEETING IS TO FAMILIARIZE NRC WITH CONTENTS OF TECHNICAL SPECIFICATION 318



## **II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION**

- **PROPOSED TECHNICAL SPECIFICATION INCLUDES SIX PARTS**
  - **Part A: Unit 3 Analog Transmitter/Trip System (ATTS)**
  - **Part B: Units 1 and 3 Reactor Vessel Water Level Safety Limit and the Level 1 Low Reactor Vessel Water Level Setpoint**
  - **Part C: Unit 2 Instrument Identifiers**
  - **Part D: Unit 2 ATTS Calibration Frequencies and Functional Test Descriptions**
  - **Part E: Units 1, 2, and 3 Suppression Chamber-Reactor Building Vacuum Breakers Calibration Frequency**
  - **Part F: Corrects Capitalization of Terms**

## **II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **PART A: UNIT 3 ANALOG TRANSMITTER/TRIP SYSTEM (ATTS)**
  - **Reactor Protection System (RPS) and Emergency Core Cooling System (ECCS) Mechanical and Differential Pressure Switches are being Replaced with an ATTS**
  - **Instrument Identifiers, Functional Test Descriptions, Group Designators, Minimum Functional Test Frequency Notes, Minimum Calibration Frequencies, and an Indicator Range are being Changed to Reflect the New Equipment**
  - **Except for Some Calibration Frequency Changes (See Part D), Unit 3 Changes make Technical Specifications Consistent with Changes Previously Approved for Unit 2**

## **II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **PART A: UNIT 3 ANALOG TRANSMITTER/TRIP SYSTEM (ATTS) (Continued)**
  - **Reduces Functional Test and Calibration Frequencies**
  - **Decreases Duration and Complexity of Required Testing and Calibration**
  - **Reduces Testing and Maintenance Related Scrams**

## II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)

- PART B: UNITS 1 AND 3 REACTOR VESSEL WATER LEVEL SAFETY LIMIT AND THE LEVEL 1 LOW REACTOR VESSEL WATER LEVEL SETPOINT
  - Safety Limit from  $\geq 378$  Inches to  $\geq 372.5$  Inches to Reflect the Analytical Limit Provided by General Electric
  - Level 1 Setpoint from  $\geq 378$  Inches to  $\geq 398$  Inches to Provide a More Conservative Limit
  - Units 1 and 3 Changes make Technical Specifications Consistent with Changes Previously Approved for Unit 2

## **II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **PART C: UNIT 2 INSTRUMENT IDENTIFIERS**
  - **RPS and ECCS Instrument Identifiers Added/Corrected**
  - **Enhances Useability of Technical Specifications**

## II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)

- PART D: UNIT 2 ATTS CALIBRATION FREQUENCIES AND FUNCTIONAL TEST DESCRIPTIONS
  - Reactor High Water Level, RCIC and HPCI Turbine Steam Line High Flow, and Drywell Pressure Instrumentation Calibration Frequencies Revised to Reflect Current Calculations (18 Months)
  - Calibration Frequencies will be the Same as Proposed for the Same Unit 3 Equipment in Part A
  - Functional Test Descriptions are being Revised to Reflect Current Test Methods

## **II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **PART E: UNITS 1, 2, AND 3 SUPPRESSION CHAMBER-REACTOR BUILDING VACUUM BREAKERS CALIBRATION FREQUENCY**
  - **Calibration Frequencies Revised to Reflect Current Calculations  
(18 Months)**

## **II. SCOPE OF/REASONS FOR PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **PART F: CORRECTS CAPITALIZATION OF TERMS**
  - **Corrects Capitalization on the Affected Pages in order to Conform with the Current Definitions Section**



### **III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION**

- **COVER LETTER**
- **ENCLOSURE 1: DESCRIPTION AND EVALUATION OF THE PROPOSED CHANGE**
- **ENCLOSURE 2: MARKED TECHNICAL SPECIFICATION PAGES**
- **ENCLOSURE 3: REVISED TECHNICAL SPECIFICATION PAGES**
- **ENCLOSURE 4: SUMMARY OF COMMITMENT**

### **III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **COVER LETTER**
  - Requests Approval by April 20, 1995
  - Requests Technical Specification be made Effective Within 30 Days of NRC Approval

### **III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **ENCLOSURE 1: DESCRIPTION AND EVALUATION OF THE PROPOSED CHANGE**
  - Section I: Description of the Proposed Change (Page E1-2)
  - Section II: Reason for the Proposed Change (Page E1-57)
  - Section III: Safety Analysis (Page E1-60)
  - Section IV: No Significant Hazards Consideration Determination (Page E1-89)
  - Section V: Environmental Impact Consideration (Page E1-97)
  - Section VI: References (Page E1-98)

### **III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **ENCLOSURE 1: DESCRIPTION AND EVALUATION OF THE PROPOSED CHANGE  
(CONTINUED)**
  - **Section I (General Description) and Sections II, III and IV Address  
Each of the Six Parts Separately**
  - **Section I (Description of Individual Technical Specification Changes)  
Addresses the Specific Proposed Changes in the Order They Appear in  
the Technical Specifications**
  - **Cross Reference between Specific Proposed Changes and the Six Parts  
Included in the Reason for the Proposed Change Section**

### **III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **ENCLOSURE 2: MARKED TECHNICAL SPECIFICATION PAGES**
  - Separated by Unit in Numerical Order

### **III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION (CONTINUED)**

- **ENCLOSURE 3: REVISED TECHNICAL SPECIFICATION PAGES**
  - **Separated by Unit in Numerical Order**

### III. LAYOUT OF PROPOSED TECHNICAL SPECIFICATION (CONTINUED)

- ENCLOSURE 4: SUMMARY OF COMMITMENT
  - Unit 1 Specific Pressure Suppression Chamber-Reactor Building Vacuum Breakers Calibration Frequency Calculation will be Performed Prior to Unit 1 Restart

## IV. SAFETY ANALYSIS OF PROPOSED CHANGES

- PART A: UNIT 3 ANALOG TRANSMITTER/TRIP SYSTEM (ATTS)
  - Located in Enclosure 1, Section III, Pages E1-60 to E1-86
  - Enveloped by Approved GE Licensing Topical Report NEDO-21617-A
  - Includes Plant Specific Information Requested by NEDO-21617-A
  - Except for Some Calibration Frequency Changes (See Part D), Unit 3 Changes Previously Approved for Unit 2



## **IV. SAFETY ANALYSIS OF PROPOSED CHANGES**

**(CONTINUED)**

- **PART B: UNITS 1 AND 3 REACTOR VESSEL WATER LEVEL SAFETY LIMIT AND THE LEVEL 1 LOW REACTOR VESSEL WATER LEVEL SETPOINT**
  - Located in Enclosure 1, Section III, Pages E1-86 and E1-87
  - Safety Limit Revised to Reflect Analytical Limit Provided by General Electric
  - Level 1 Low Reactor Vessel Water Level Setpoint Revised to Provide a More Conservative Limit
  - Units 1 and 3 Changes Previously Approved for Unit 2

## **IV. SAFETY ANALYSIS OF PROPOSED CHANGES**

**(CONTINUED)**

- **PART C: UNIT 2 INSTRUMENT IDENTIFIERS**
  - Located in Enclosure 1, Section III, Page E1-88
  - Administrative Change
  - No Change in Equipment, Operation, or Function

## IV. SAFETY ANALYSIS OF PROPOSED CHANGES (CONTINUED)

- PART D: UNIT 2 ATTS CALIBRATION FREQUENCIES AND FUNCTIONAL TEST DESCRIPTIONS
  - Located in Enclosure 1, Section III, Page E1-88
  - Current Calibration Frequency Calculations Based on Regulatory Guide 1.105, Instrument Setpoints for Safety Related Systems
  - Regulatory Guide 1.105 Endorses Instrument Society of America (ISA) Standard ISA-S67.04 - 1982, Setpoints for Nuclear Safety Related Instrumentation Used in Nuclear Power Plants
  - Calculations also Conform to ISA-S67.04 - 1982
  - Functional Test Descriptions are being Revised to Reflect Current Test Methods

## IV. SAFETY ANALYSIS OF PROPOSED CHANGES (CONTINUED)

- PART E: UNITS 1, 2, AND 3 SUPPRESSION CHAMBER-REACTOR BUILDING VACUUM BREAKERS CALIBRATION FREQUENCY
  - Located in Enclosure 1, Section III, Pages E1-88 and E1-89
  - Current Calibration Frequency Calculations Based on Regulatory Guide 1.105, Instrument Setpoints for Safety Related Systems
  - Regulatory Guide 1.105 Endorses Instrument Society of America (ISA) Standard ISA-S67.04 - 1982, Setpoints for Nuclear Safety Related Instrumentation Used in Nuclear Power Plants
  - Calculations also Conform to ISA-S67.04 - 1982

## IV. SAFETY ANALYSIS OF PROPOSED CHANGES (CONTINUED)

- PART F: CORRECTS CAPITALIZATION OF TERMS
  - Located in Enclosure 1, Section III, Page E1-89
  - Administrative Change
  - Conforms with Current Definitions Section