



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 213 TO FACILITY OPERATING LICENSE NO. DPR-59
POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated December 29, 1993, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise Appendix B of the TSs, the Radiological Effluent Technical Specifications (RETS). Specifically, the changes would revise Surveillance Requirement (SR) 3.1.a. and Table 3.10-2, "Minimum Test and Calibration Frequency for Radiation Monitoring Systems," to provide surveillance requirements for data recorders associated with the gaseous effluent monitoring system. The proposed amendment would also make an editorial change to Appendix B Limiting Condition for Operation (LCO) 3.1.a. to improve consistency and clarity.

2.0 EVALUATION

Appendix B SR 3.1.a. and Table 3.10-2 currently require the performance of instrument checks, tests, and calibrations to assure the operability of the main stack, refuel floor, reactor building, turbine building and radwaste building exhaust gaseous effluent radiation monitors. The TSs do not now require surveillance of the associated data recorders. However, LCO 3.1.a. specifically states that "... pathways shall be monitored and recorded ..." which requires the recorders to be operable for the gaseous effluent monitoring system to be considered operable.

The licensee has proposed that SRs be established for these data recorders by adding the recorders to SR 3.1.a. and Table 3.10-2. Specifically, the words "and recorders" would be added between the words "monitors" and "listed" in the first sentence of SR 3.1.a. The gaseous effluent monitor channel identifiers in Table 3.10-2 would be modified to read as follows:

Main Stack Exhaust Monitors and Recorders
Refuel Area Exhaust Monitors and Recorders
Reactor Building Area Exhaust Monitors, Recorders, and Isolation
Turbine Building Exhaust Monitors and Recorders
Radwaste Building Exhaust Monitors and Recorders

The licensee also proposed editorial changes to Appendix B LCO 3.1.a. Specifically, in LCO 3.1.a. the word "exhaust" would be added after "Main stack" and the word "vent" would be replaced with "exhaust" for the other four release pathways. This change would make the release pathway designations in LCO 3.1.a. consistent with the instrument channel identifiers in Table 3.10-2.

The NRC staff has concluded that the proposed changes to Appendix B SR 3.1.a. and Table 3.10-2 will establish consistency between LCO 3.1.a. and SR 3.1.a. and provide greater confidence in system operability. The proposed changes are, therefore, acceptable. The staff has also concluded that the proposed editorial changes to Appendix B LCO 3.1.a. will improve consistency within the RETS and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 4946). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
J. Menning

Date: June 16, 1994

June 16, 1994

Mr. William A. Josiger, Acting Executive
Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Josiger:

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M88462)

The Commission has issued the enclosed Amendment No. 213 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 29, 1993.

The amendment revises Appendix B of the TSs, the Radiological Effluent TSs. Specifically, the amendment revises Appendix B Surveillance Requirement 3.1.a. and Table 3.10-2 to provide surveillance requirements for data recorders associated with the gaseous effluent monitoring system. The amendment also makes an editorial change to Appendix B Limiting Condition for Operation 3.1.a. to improve consistency and clarity.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
ORIGINAL SIGNED BY:
John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

- Enclosures:
1. Amendment No. 213 to DPR-59
2. Safety Evaluation
cc w/enclosures:
See next page

PDI-1:EA <i>for</i>	PDI-1:PM <i>in</i>	NRR/PRP3 <i>PRP3</i>	NRR/HICB <i>HW</i>	OGC <i>OPW</i>	PDI-1:D <i>MB</i>
CVogam	JMenning:avl	LCunningham	JWermiel	<i>OPW</i>	MBoyle
5/20/94	5/20/94	5/23/94	5/24/94	5/24/94	6/16/94

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