



June 20, 1994
LD-94-042

Docket No. 52-002

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Revised System 80+™ Certified Design Material

References: 1) Letter LD-93-178, C. Brinkman (ABB-CE) to NRC, dated December 31, 1993
2) Letter, T. Boyce (NRC) to C. Brinkman (ABB-CE), dated March 24, 1994
3) Letter LD-94-032, C. Brinkman (ABB-CE) to NRC, dated April 29, 1994

Dear Sirs:

Enclosed are thirty-seven (37) copies of System 80+ Certified Design Material (CDM) and the affidavit, as required by 10 CFR 50.4(b) and 10 CFR 50.30(b). The enclosed CDM replaces ABB-CE's December 31, 1993, CDM submittal (Reference 1) in its entirety except for the divider tabs and binders. The content of the enclosed CDM is as described below.

ABB-CE met with the NRC staff on April 19 and 20, 1994, to discuss responses to NRC comments (Reference 2) on the System 80+ CDM and related CESSAR-DC material. Reference 3 was subsequently submitted to document the responses to those comments. The enclosed CDM includes the Reference 3 revisions to the CDM. Revisions to CESSAR-DC will be provided under separate cover in Amendment W.

The enclosed CDM includes the following revisions that did not arise from the NRC comments of Reference 2:

- 1) The date was changed to 6/17/94 on all pages;
- 2) The figure legend was modified by deleting features not shown on CDM figures (e.g., high and low alarm indications). A symbol for a finned cooling coil was added and editorial changes were made;
- 3) Designation of plant north was removed from the figures of CDM Section 2.1.1 (Nuclear Island Structures);

ABB Combustion Engineering Nuclear Power

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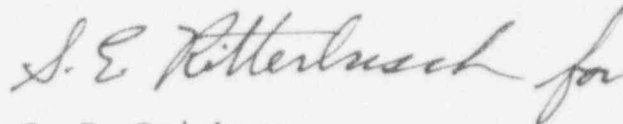
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- 4) The in-containment refueling water storage tank and the holdup volume tank are part of the nuclear island structures rather than components of the in-containment water storage system. Similarly, the emergency feedwater storage tanks are part of the nuclear island structures, not part of the emergency feedwater system. CDM Section 2.4.7 (In-Containment Water Storage System) and Section 2.8.8 (Emergency Feedwater System) have been revised to show that the subject tanks are part of the nuclear island structures;
- 5) For consistency with the CDM text, isolation devices were added in the signal path between DIAS-P and Channel A signal conditioning equipment on Figure 2.5.1-2 and between DIAS-P and signal conditioning equipment on Figure 2.5.2-2;
- 6) Sections 2.6.1 and 2.6.2 were modified to explicitly identify, in the CDM text, the existence of controls listed in the minimum inventory of Table 2.12.1-1 for operating the AC electrical power distribution system and the emergency diesel generator system. ITAAC 26.b) of Table 2.6.1-1 now requires verifying the operability of AC electrical power distribution system controls by testing;
- 7) ITAAC 8 of Table 2.7.17-1 has been modified to state that the TSC can be maintained at a positive pressure with respect to surrounding areas except for the MCR;
- 8) Per agreement to address closure of containment isolation valves on CIAS only in Section 2.4.5 (Containment Isolation System), the CIAS has been deleted from Figures 2.7.21-1 and 2.7.21-2;
- 9) Section 2.7.22 (Containment Cooling and Ventilation System) text has been modified to allow the radiation instrument shown on Figure 2.7.22-1 to be located outside the containment as is done in the current design;
- 10) To achieve consistency with CESSAR-DC, entries were added to Table 2.12.1-1 for instrumentation for primary coolant radiation, RCS pressure, reactor cavity level, EPW storage tank level, containment radiation, and SG pressure;
- 11) Responses to radiation protection questions not included in Reference 2 and responses to ACRS questions affecting the CDM have been incorporated.

Please feel free to query us as the staff evaluates this submittal. You may contact me, Mr. George Hess at (203) 285-5218, or Mr. John Rec at (203) 285-2861.

Very truly yours,

COMBUSTION ENGINEERING, INC.



C. B. Brinkman
Director
Nuclear Systems Licensing

gdh/lw

cc: T. Boyce (NRC)
M. Franovich (NRC)
P. Lang (DOE)
J. Trotter (EPRI)
T. Wambach (NRC)

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of:)
)
Combustion Engineering, Inc.)
)
Standard Plant Design)

APPLICATION FOR REVIEW OF
"COMBUSTION ENGINEERING STANDARD
SAFETY ANALYSIS REPORT -
DESIGN CERTIFICATION"

Regis A. Matzie, being duly sworn, states that he is the Vice President, ABB Combustion Engineering Nuclear Systems Engineering, of Combustion Engineering, Inc.; that he is authorized on the part of said corporation to sign and file with the Nuclear Regulatory Commission this document; and that all statements made and matters set forth therein are true and correct to the best of his knowledge, information, and belief.

COMBUSTION ENGINEERING, INC.

By: Regis A. Matzie
Regis A. Matzie
Vice President
ABB Combustion Engineering
Nuclear Systems Engineering

Subscribed and sworn to
before me this 20th day
of June, 1994.

Laurie J. White
Notary Public

My Commission Expires: 8/31/99