NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) LICENSE NUMBER (2) (3)0 B Η 2 0 0 0 -00 LICENSEE CODE LICENSE CON'T REPORT 3 8 0 4 2 6 8 1 (7) 0 4 1 2 8 0 1 3 0 (6)0 510 0 0 SOURCE REPORT DATE DOCKET NUMBER EVENT DATE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) Unit 2 was shut down on 03/25/83 for Refueling 9. Eddy current examinat 0 2 tion of the steam generator tubes was conducted from 04/07/83 to 0 3 04/12/83. On 04/12/83 eddy current data verified that 16 tubes in the 0 4 "A" steam generator and 2 tubes in the "B" steam generator had indica-0 5 tions greater than the repair limit of 40%. A secondary-to-primary 0 6 leak check was also performed. This event is similar to others, and 0 7 is reportable per Technical Specification 15.6.9.2.A.3. 0 8 SYSTEM CAUSE CAUSE COMP VALVE COMPONENT CODE CODE SUBCODE SUBCODE CB E (12) D (13) T E X C H (14) F (15) 0 9 (11) Z (16) H 18 OCCURRENCE REVISION SEQUENTIAL REPORT REPORT NO. EVENT YEAR CODE TYPE LER RO NO. REPORT 0 0 2 0 1 0 3 T 8 NUMBER 32 PRIME COMP. COMPONENT NPRD-4 ACTION FUTURE EFFECT ON PLANT METHOD ATTACHMENT SUBMITTED (22) HOURS FORM SUB. 0 0 0 2 0 (26) B (18) Z (19 Z (20) Z (21) Y (23) 0 Y N (25) 11 (24) W CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Fourteen indications in the "A" SG were caused by IGA within the tube-10 111 | sheet. One indication in the "A" SG and all "B" SG indications are rem-[ nants of phosphate wastage at the top of the tubesheet. One indication] occurred at the "A" SG first support plate. All tubes with degradation [ 1 3 greater than 40% will be repaired. Sludge lancing was also performed. 1 4 METHOD OF FACILITY OTHER STATUS (30) DISCOVERY DESCRIPTION (32) % POWER H (28) 0 0 0 (29) 5 B (31) Eddy current examination 80 CONTENT ACTIVITY AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) RELEASED\_OF RELEASE Z (33) Z (34) N/A N/A 6 10 11 PERSONNEL EXPOSURES 80 DESCRIPTION (39) NUMBER TYPE 0 0 0 (37) Z (38) N/A PERSONNEL INJURIES 80 DESCRIPTION (41) NUMBER N/A 0 0 1 0 (40) 1 8 12 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z (42) N/A 1 9 8305030280 830426 PUBLICITY NRC USE ONLY DESCRIPTION 45 PDR ADOCK 05000301 ISSUED Z (44) N/A 9 91 7-912 210 . . . . . . . . . . . . PDR 80 68 414/277-2811 C. W. Fay NAME OF PREPARER. PHONE:

## ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-002/01T-0

Wisconsin Electric Power Company Point Beach Nuclear Plant Unit 2 Docket No. 50-301

On March 25, 1983, Unit 2 was shut down for its ninth refueling outage. Eddy current testing of the steam generators commenced on April 7, 1983. The eddy current inspection program consisted of the following:

- 1. Full length inspection of 33 tubes in the "B" steam generator which contained previous cold-leg degradation.
- Inspection over the U-bend of 56 tubes in the "A" steam generator, and 57 tubes in the "B" steam generator with prior degradation in the hot leg.
- 3. Full length inspection of 126 tubes in the "A" steam generator and inspection over the U-bend of 128 tubes in the "B" steam generator to meet the requirements of the Technical Specification and Regulatory Guide 1.83 recommendations.
- 4. Inspection to the first support from the hot leg of essentially all readily remotely accessible tubes.

There were four tubes per steam generator that were not inspected due to fixture interference. These tubes are not located in a wastage or IGA problem area, and the overall eddy current results did not indicate the necessity to inspect the tubes.

On April 12, 1983, verification of all steam generator eddy current data for tubes with indications exceeding the plugging limit was completed. Sixteen (16) tubes in the "A" steam generator were verified to have degradation greater than 40%, which is the repair limit of Technical Specification 15.6.2.A.5. These tubes are as follows:

## "A" STEAM GENERATOR

Tube	Defect	Location	Cause
R19C5	47%	#1 TSP-H/L	Cracking
R1C90	83%	4.5-8.0" ATE-H/L	Crevice Corrosion
R16C30	100%	5.5-16" ATE-H/L	Crevice Corrosion
R20C37	100%	6-15" ATE-H/L	Crevice Corrosion
R17C42	100%	3-16" ATE-H/L	Crevice Corrosion
R16C43	91%	5.5-18.5" ATE-H/L	Crevice Corrosion
R18C43	68%	16" ATE-H/L	Crevice Corrosion
R14C48	89%	12-15" ATE-H/L	Crevice Corrosion
*R17C30	81%	12.5"ATE-H/L	Crevice Corrosion

Tube	Defect	Location	Cause
*R17C34	86%	7.5" ATE-H/L	Crevice Corrosion
*R18C33	84%	6" ATE-H/L	Crevice Corrosion
*R17C39	41%	TTS-H/L	Thinning
	89%	14" ATE-H/L	Crevice Corrosion
*R19C37	888	5" ATE-H/L	Crevice Corrosion
R18C24	89%	10-12" ATE-H/L	Crevice Corrosion
R5C21	43%	TTS-H/L	Thinning
*R26C27	81%	5.5-17.5" ATE-H/L	Crevice Corrosion

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\* These tubes also have undefinable indications and are included in the below list.

In addition to the defects listed above, undefinable indications were found in 14 additional tubes within the tubesheet. All the tubes containing undefinable indications are as follows:

Tube	Location	Tube	Location
R14C45	13-19.5" ATE-H/L	R16C31	9-15" ATE-H/L
R17C44	8-13" ATE-H/L	R17C30	6-14" ATE-H/L
R18C44	4.5-14" ATE-H/L	R20C36	13-18" ATE-H/L
R12C48	8-8.5" ATE-H/L	R17C34	7-16" ATE-H/L
R19C47	7-16" ATE-H/L	R18C33	6-15" ATE-H/L
R14C47	14-18" ATE-H/L	R17C39	10-17.5" ATE-H/L
R13C47	11.5-16" ATE-H/L	R17C38	6-18" ATE-H/L
R9C47	11.5-17.5" ATE-H/L	R19C37	4.5-18" ATE-H/L
R15C46	8.5-16" ATE-H/L	R17C37	3.5-7.5" ATE-H/L
R17C31	12.5-16" ATE-H/L	R26C27	5.5-17.5" ATE-H/L

ATS - Above Tubesheet ATE - Above Tube End TTS - Top of Tubesheet H/L - Hot Leg C/L - Cold Leg

Two tubes in the "B" steam generator were verified to have degradation greater than 40%. These tubes are as follows:

Tube	Defect	Location	Cause
R9C48	43%	1" ATS-C/L	Thinning
R17C28	51%	TTS-H/L	Thinning

The above tubes for both steam generators will be repaired by sleeving or plugging in accordance with Technical Specification 15.4.2.6.

The following table summarizes the number of indications found during the 1983 inspection. Also, for comparison purposes, the number of indications found in each category during the 1982 inspection is provided.

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Indication	"A" Steam	Generator	"B" Steam	Generator
	1982	1983	1982	1983
Hot Leg Thinning	414	469	149	232
Cold Leg Thinning	0	2	32	33
Indications at Support Plates	5	5	0	0
Crevice Corrosion	8	27	0	0
Distorted Tubesheet Signals	101	132	49	121

It should be noted that the eddy current program performed in 1982 was not the same as the 1983 program. The extent of the eddy current inspection programs for 1982 and 1983 is summarized below:

	Tubes Inspected			
	"A" Stea	am Generator	"B" Stear	m Generator
	1982	1983	1982	1983
To First Support	1304	2982	1311	2997
Over U-Bend	565	0	250	185
Full Length	0	182	31	33

Some restrictions at the first tube support plate were encountered during the inspection. The majority of the restrictions in the periphery were present during previous inspections. The "A" steam generator contained substantially more restrictions than the "B" steam generator; thus, it was decided to gauge the restrictions in the peripheral tubes of "A". All of these restrictions passed a .650 diameter probe. Some restrictions at the first support were also encountered in the central region of the "A" steam generator. These restrictions were not considered to be an indication of a problem since a larger diameter probe than usual was used in this area. The large diameter probe was used to provide gauging information for sleeving.

An 800 psid secondary-to-primary leakage check was performed on both steam generators on April 9, 1983. The 800 psid secondary-to-primary leakage check was performed visually with the aid of remote video equipment. The specific conditions identified during the leakage checks are noted below.

	"A" :	Steam Ger	nerator				
		Hot Lee	1				
R27C82	(explosive	plug)	Wet				
R31C52	(explosive	plug)	Wet				
R32C15	(explosive	plug)	Wet				
R17C42	Tube		10-12	drops	per	minute	
R16C30	Tube		10-15	drops	per	minute	
R20C37	Tube		Wet				

## "B" Steam Generator Hot Leg

R26C84 (explosive plug)	Wet
R28C72 Tube	Wet
R7C83 Tube	Wet
R30C81 Tube	Wet

It is intended that all the explosive plugs noted above will be repaired this outage. In addition, R40C25, an explosive plug in the "A" steam generator hot leg, will also be repaired since it was noted during previous outages as a potential leaker.

The wet unplugged tubes in the "A" steam generator each showed 100% eddy current indications within the tubesheet. The wet unplugged tubes in the "B" steam generator were inspected over the U-bend and no detectable degradation was found. These tubes will be checked again during the leak check performed after sleeving.

All tubes having degradation greater than 40% will be repaired by sleeving or plugging prior to startup in accordance with Technical Specification 15.4.2.6.

Based on the results of the eddy current and comparisons with previous inspections, there has been no real progression of wastage at the top of the tubesheet or denting at the supports. However, corrosion within the tubesheet crevice is occurring.

The indications within the tubesheet area are believed to be the results of intergranular attack caused by caustic corrosion. The indications at the top of the tubesheet, or above, are believed to be remnants of phosphate wastage, as evidenced by the fact that they were noted during previous outages. The eddy current indications of wastage are not significantly greater than the plugging limit and, for the most part, the difference in comparison to previous outages is within the expected range of scatter for small volume indications which are masked by a tubesheet signal.

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In an attempt to reduce corrosion, the steam generators have been sludge-lanced. In addition, to eliminate the concern with crevice corrosion and wastage at the top of the tubesheet, a major sleeving effort is planned for this outage. The planned sleeving program will cover 590 of the 628 indications found this outage in the "A" steam generator hot leg, and 325 of the 353 indications found in the "B" steam generator hot leg.

This event is reportable in accordance with Technical Specification 15.6.9.2.A.3.

The Resident Inspector has been notified of this event.