April 25, 1983

Mr. James G. Keppler, Regional Administrator Directorate of Inspection and Enforcement - Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

Subject: Byron Station Units 1 & 2

10 CFR 50.55(e) 30-Day Report NSSS Support Steel Installation NRC Docket Nos. 50-454/455

Reference (a): E. D. Swartz letter to J. G. Keppler

dated April 8, 1983

Dear Mr. Keppler:

On March 25, 1983, the Commonwealth Edison Company Project Construction Department notified Mr. Julian Hinds of your office of a potential deficiency reportable pursuant to 10 CFR 50.55 (e), concerning the installation of NSSS Support Steel by Hunter Corporation at our Byron Station. For your tracking purposes, this potential deficiency was assigned Number 83-03.

This letter fulfills the thirty (30) day reporting requirements of 10 CFR 50.55(e) regarding this matter and is considered a final report.

DESCRIPTION OF DEFICIENCY:

During a review of the as-built location of the Unit l & 2 Reactor Coolant Pump and Steam Generator Support Columns, it was determined that the columns were not installed within the tolerances specified on the Sargent and Lundy Design Drawing S-1105. The tolerances specified were referenced to dimensions from the building structure, while the actual installation was performed with respect to the installed equipment location. These deficiencies have been documented on CECo NCR Nos. F-750 and F-803.

ANALYSIS OF SAFETY IMPLICATIONS:

A review was performed by Sargent and Lundy to evaluate the impact of CECo NCR Nos. F-750 and F-803 on plant design and safety. This review provided revised tolerances for the installation of NSSS Support Steel which will ensure proper operation of these components.

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CORRECTIVE ACTION TO BE TAKEN:

The following summarizes the corrective action taken to date and to be performed in the future:

Unit #1

- The Steam Generator Columns have been verified to be acceptable as installed using the revised tolerances. The Reactor Coolant Pump Columns have been reworked, as required, to conform to the revised installation tolerances.
- 2. The Pressurizer Columns and Base, and the Lateral Supports on the Steam Generators and Reactor Coolant Pumps will be verified with the new tolerances as part of the clearance checks performed during Hot Functional Testing.

Unit #2

The Steam Generator, Reactor Coolant Pump, and Pressurizer Column and Lateral Support Installations will be verified using the new tolerances.

The amount of rework required to correct and verify that the NSSS Support Steel is installed within the revised tolerances was deemed significant enough to make this a reportable construction deficiency. The completion of this work will be tracked on CECo NCR Nos. F-750 and F-803. Any required rework and verification will be completed prior to Fuel Load for Unit #1 and by February 4, 1984 for Unit #2.

Reference (a) provided the 10 CFR 50.55(e) 30-day response Nos. 83-03 concerning a similar potential deficiency at our Braidwood Station. In that case, the re-work was not considered to be extensive and we ultimately determined that the nonconformance was not reportable pursuant to 10 CFR 50.55(e). As stated above, we are cognizant of the requisite rework necessary to correct the Byron Station nonconformance. However, because of our experience gained at Byron Station in their rework efforts, we believe that the costs to be incurred for the requisite Braidwood Station corrective rework will not be extensive. In our judgement, the similar nonconformance at our Braidwood Station continues to not be reportable pursuant to 10 CFR 50.55(e).

Please address any questions concerning this matter to this office.

Very truly yours,

E. Douglas Swartz

Nuclear Licensing Administrator

cc: Region III Inspector - Byron

Director of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, DC 20555