



**Entergy
Operations**

Entergy Operations, Inc.

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June 15, 1994

2CAN069403

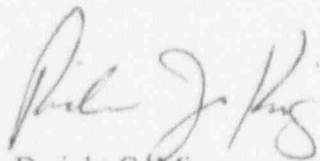
U. S. Nuclear Regulatory Commission
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Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 2 Monthly Operating Report (MOR) for May 1994 is attached. This report is submitted in accordance with ANO-2 Technical Specification 6.9.1.6.

Very truly yours,


for Dwight C. Mims
Director, Licensing

DCM/jrh
Attachment

9406210048 940531
PDR ADOCK 05000368
R PDR

Handwritten initials/signature

cc: Mr. Leonard J. Callan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
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OPERATING DATA REPORT

DOCKET NO: 50-368
 DATE: June 2, 1994
 COMPLETED BY: M. S. Whitt
 TELEPHONE: (501) 964-5560

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: May 1-31, 1994
3. Licensed Thermal Power (MWt): 2,815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

	<u>MONTH</u>	<u>YR-TO-DATE</u>	<u>CUMULATIVE</u>
11. Hours in Reporting Period	744.0	3,623.0	124,319.0
12. Number of Hours Reactor was Critical	744.0	2,602.6	95,424.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	744.0	2,570.1	93,496.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	2,059,877	6,973,084	247,953,067
17. Gross Electrical Energy Generated (MWH)	679,828	2,299,613	81,621,950
18. Net Electrical Energy Generated (MWH)	649,543	2,187,552	77,660,461
19. Unit Service Factor	100.0	70.9	75.2
20. Unit Availability Factor	100.0	70.9	75.2
21. Unit Capacity Factor (Using MDC Net)	101.8	70.4	72.8
22. Unit Capacity Factor (Using DEC Net)	95.7	66.2	68.5
23. Unit Forced Outage Rate	0.0	0.0	10.9
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End of Report Period. Estimated Date of Startup: _____
26. Units in Test Status (Prior to Commercial Operation): _____

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	_____	<u>12/05/78</u>
INITIAL ELECTRICITY	_____	<u>12/26/78</u>
COMMERCIAL OPERATION	_____	<u>03/26/80</u>

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368
 UNIT: Two
 DATE: June 2, 1994
 COMPLETED BY: M. S. Whitt
 TELEPHONE: (501) 964-5560

MONTH May 1994

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	846
2	855
3	882
4	884
5	887
6	866
7	697
8	767
9	865
10	891
11	889
12	888
13	890
14	887
15	889
16	889
17	891
18	891
19	890
20	875
21	888
22	887
23	886
24	886
25	883
26	886
27	889
28	889
29	887
30	884
31	883

AVGS: 873

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Complete to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

MAY 1994

UNIT TWO

The unit began the month of May in a power hold at 97%.

Power escalation was initiated at 1645 hours on the second, and 100% power was attained at 1824 hours that same day. Power reduction to 95% was commenced for moderator temperature coefficient (MTC) testing at 1001 hours on the sixth. Following completion of the MTC test, power escalation was commenced and 100% power was attained at 2300 hours that same day. A power reduction to 70% for condenser tube leak repairs was commenced at 0300 hours on the seventh. Following completion of the condenser repairs, power escalation was commenced at 1710 hours on the seventh. Power was held at 85% per the system dispatcher until the ninth when 100% power was attained at 0615 hours. A turbine control valve stroke test required a power reduction to 90% so the control valve testing could be completed. Following completion of the test, power was returned to 100% at 0028 hours on the twentieth-first.

The unit ran at 100% power for the remainder of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT FOR MAY 1994

DOCKET NO.	<u>50-368</u>
UNIT NAME	<u>ANO Unit 2</u>
DATE	<u>May 5, 1994</u>
COMPLETED BY	<u>M. S. Whitt</u>
TELEPHONE	<u>501-964-5560</u>

<u>NO.</u>	<u>DATE</u>	<u>TYPE</u> ¹	<u>DURATION</u> <u>(HOURS)</u>	<u>REASON</u> ²	<u>METHOD OF</u> <u>SHUTTING DOWN</u> <u>REACTOR</u> ³	<u>LICENSEE</u> <u>EVENT</u> <u>REPORT #</u>	<u>SYSTEM</u> <u>CODE</u> ⁴	<u>COMPONENT</u> <u>CODE</u> ⁵	<u>CAUSE & CORRECTIVE ACTION TO</u> <u>PREVENT RECURRENCE</u>
94-02	940507	S	0	H	5	N/A	SG	COND	Power reduction to locate and plug leaking condenser tube.

¹
F: Forced
S: Scheduled

²
Reason:
A - Equipment Failure (Explain)
B - Maintenance of Test
C - Refueling
D- Regulatory Restriction
E - Operator Training & License Examination
F - Administration
G - Operational Error
H - Other (Explain)

³
Method:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Continuation
5 - Load Reduction
9 - Other

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

⁵
Exhibit I - Same Source

DATE: May 1994

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. September 22, 1995
3. Scheduled date for restart following refueling. November 6, 1995
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Unknown at this time.
5. Scheduled date(s) for submitting proposed licensing action and supporting information.
Unknown at this time.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
None planned.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
a) 177 b) 637
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 988 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
DATE: 1997 (Loss of full core off-load capability)