U. S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: (4) 0 (3) 0 (5)0 0 0 0 4 LICENSEE CODE CON'T REPORT 219 (7)1 BL 05 11 8 2 011 8 0 1 L (6) SOURCE REPORT DATE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) While in Mode 6 and performing OP-4613, "Reactor Permissive Circuit Cal-0 2 libration and Functional Test", bistables 420 and 423 were found out of T] 0 3 lechnical Specification limits. During subsequent review it was found tha 0 4 It a Technical Specification misinterpretation had been written into the 0 5 procedure in 1977. The requirements are found in T.S.4.3.1.2 and Table 3] 0 6 .3-1. There is no impact on the safety analysis and no adverse effect on 0 the public hea th or safety. 0 80 COMP VALVE SYSTEM CAUSE CAUSE CODE SUBCODE COMPONENT CODE SUBCODE IAI A (12) C (13) X (15 7 (16) N S R U (14 11 9 0 REVISION OCCURRENCE REPORT SEQUENTIAL REPORT NO. CODE TYPE NO. EVENT YEAR LER/RO 011 (17)REPORT 0 3 T NUMBER COMPONENT PRIME COMP. SHUTDOWN METHOD ATTACHMENT NPRD-/ ACTION FUTURE EFFECT ON PLANT (22) HOURS FORM SUB SUFPLIER MANUFACTURER 3 3 N (24) Y (23) N (25) 00 N 5 (26) 18) 01 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) cause of this event is attributed to a misinterprotation of thi The 1 0 root 1] Is Technical Specification. When discovered steps were immediately taken Ito correct the condition, Personnel were reminded of the importance of TI echnical Specification compliance and that extreme care be taken in theil Ir interpretation. No further corrective actions are necessary. 4 80 METHOD OF DISCOVERY FACILITY (30)DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER H (28) 0 010 B (31) Surveillance Test 5 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY 35 RELEASED_OF RELEASE Z 33 Z 34 N/A 6 N/A 41 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 0 (37) (38) N/A 0 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 (40) NA 0 0 80 8211180326 821105 PDR ADOCK 05000029 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION TYPE Z (42) S FDR N/A 80 PUBLICITY NRC USE ONLY DESCRIPTION (45) ISSUED, N (44) 111 N/A 60 80.5 68 NAME OF PREPARER Norman N. St. Laurent PHONE (413) 625-6140

LER 82-035/01T-1 Yankee Atomic Electric Company Yankee-Rowe 50-29

Event Description And Probable Consequences

While in Mode 6 and performing procedure OP-4613, "Reactor Permissive Circuit Calibration and Functional Test", the startup rate bistable setting was found at 17.63 MWe which is out of both procedural and Technical Specification limits as stated in the procedure.

In reviewing the procedural requirements and the associated Technical Specification requirements, it was discovered that the procedure was written to calibrate the bistables contrary to Technical Specification Table 3.3.1.

The At Power Trip Bistable was calibrated to actuate at 15.12 to 16.56 MWe instead of prior to 15 MWe increasing and the startup rate bistable was calibrated to actuate at 12.96 to 14.4 MWe instead of prior to 15 MWe decreasing. This procedural inadequacy has existed since 6/16/77.

It has been determined that the effect of these small changes in the permissive circuit deenergizing/energizing has no impact on the results of the Yankee reference safety analysis. Therefore, there could be no plant operation beyand the assumptions used in the reference safety analysis. There were no adverse consequences to the public health or safety.

Cause Description And Corrective Actions

The cause of this event is attributed to a misinterpretation of the Technical Specification. In turn, this misinterpretation was translated into the procedure to functionally test and calibrate the circuit.

When the error was discovered, steps were immediately implemented to correct it. A minor circuit modification will be implemented to facilitate calibration and the procedure will be revised to ensure that its properly tested and calibrated to satisfy Technical Specification requirements.

Personnel were reminded of the importance of Technical Specifications and that extreme care should be used in their interpretation. No further corrective actions are necessary.