

DECOMMISSIONING PLAN
FOR THE
LYNCHBURG RESEARCH CENTER'S
BUILDING C

NRC SNM LICENSE 778

BABCOCK & WILCOX
Lynchburg Research Center
Lynchburg, Virginia

March 1983

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1.0 INTRODUCTION

1.1 OBJECTIVE OF DECOMMISSIONING

Building C at Babcock & Wilcox's Lynchburg Research Center has been used for nuclear fuels R&D since 1962. During the late 1960's and early 1970's, plutonium fuels R&D was conducted in this building. The objective of this decommissioning project is to decontaminate the building so as to enable re-release of the facility for unrestricted use and from licensed control.

1.2 OBJECTIVE OF DOCUMENT

The Babcock & Wilcox Company (B&W) holds NRC License SNM-778 to conduct operations at its Lynchburg Research Center (LRC) near Lynchburg, Virginia. B&W, in ceasing licensed activities in Building C, is complying with a valid license requirement to provide for decontamination of this building to protect the environment and the general public from exposure to levels of radioactivity in excess of those permissible. The objective of this document is to provide a plan for the decontamination of Building C in a responsible and safe manner.

1.3 ORGANIZATION OF DOCUMENT

This Decommissioning Plan contains nine sections. Following this introductory section, a history of Building C is provided to describe operations conducted relating to nuclear fuels R&D. The third section describes the current degree and type of residual radioactive contamination. The fourth and fifth sections describe the planned decontamination work and the criteria applicable to comprehensive radiation surveys following decontamination. The sixth section discusses procedures that will be used to assure that all operations will be conducted safely throughout the decontamination and to assure that the building will be left in a safe condition. Information is provided about environmental considerations in Section 7. Section 8 contains information describing the administrative requirements for this project. The final summary section highlights the important points of the decontamination and decommissioning of Building C.

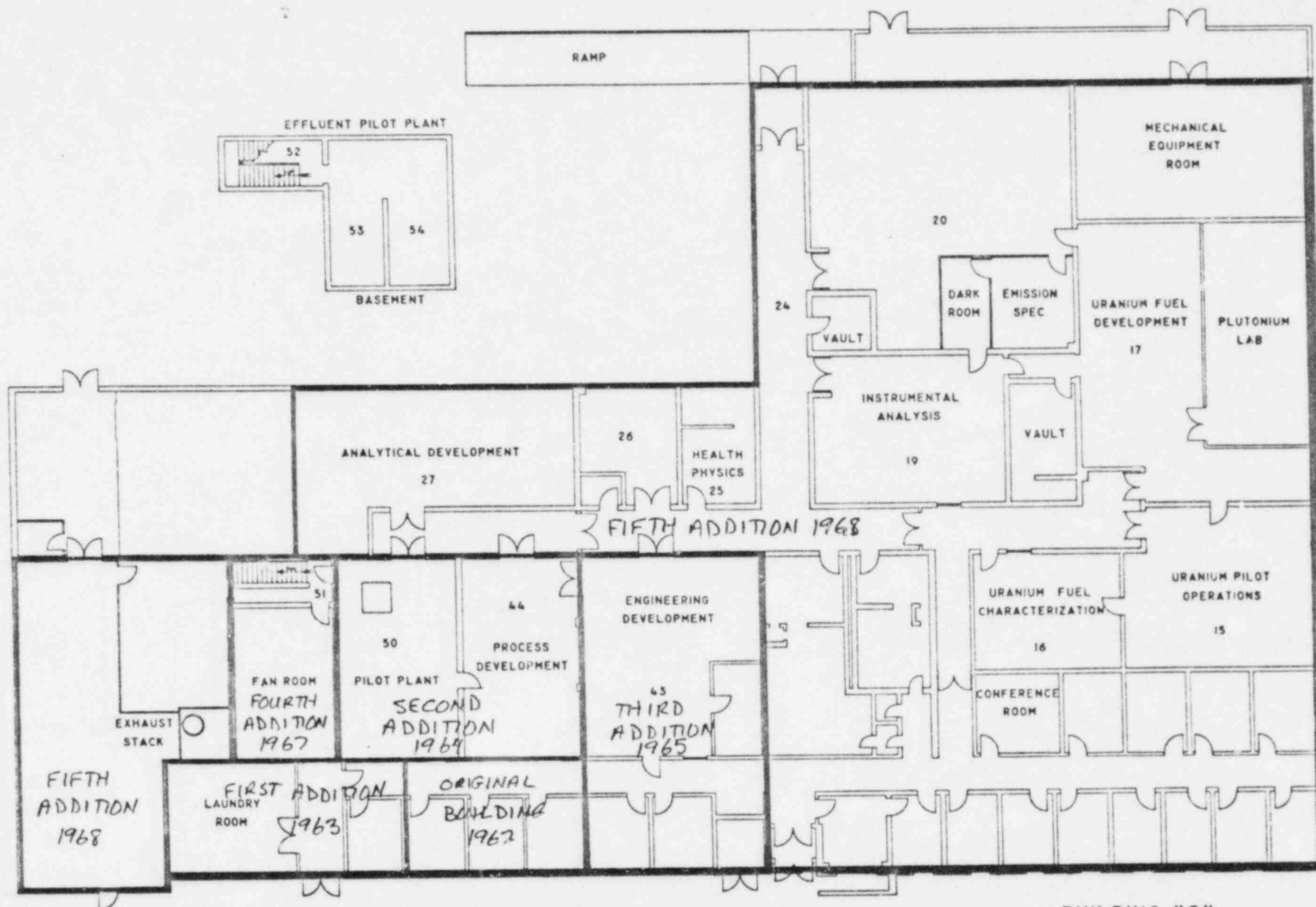
2.0 HISTORY OF BUILDING C

The existing structure known as Building C is the result of several additions to a small laboratory completed in 1962 (see Fig. 2-1). Bench-scale experiments were performed in this first laboratory with thorium-bearing fuels, converting thorium nitrate to thorium oxide. An expansion in 1964 added pilot facilities for preparation of thorium- ^{233}U nuclear fuel by a sol-gel method. R&D with plutonium-bearing fuels was initiated in new laboratories added in 1966. A major expansion was completed in 1968 to form Building C as it currently stands. Plutonium-fuel R&D activities included powder preparation via coprecipitation, sol-gel, and physical blending methods with fabrication of pellet, shard, and spherical particle fuel rods. Uranium fuel fabrication projects were performed in existing laboratories that were vacated at the conclusion of the FFTF program in 1971.

Thorium fuel R&D was conducted in Laboratories 43, 44, 50, 53, and 54 and existing offices that used to be part of Laboratories 43, 44, and 50. These projects were conducted during 1962-66. No further work has been performed on thorium fuels.

Plutonium fuel R&D was conducted in Laboratories 15, 16, 17, 19, 20, 43, 44, 50, 53, and 54 and existing offices that used to be part of Laboratories 43, 44, and 50. (Laboratory 25 has been used as Health Physics Office; Laboratory 26, Technician Office; and Laboratory 27, glovebox construction prior to conversion to an analytical chemistry laboratory for non-plutonium materials.) Plutonium-fuel projects were primarily conducted during 1966-71. (Two small projects were conducted for DOE during 1979-80 in Lab 17.) Plutonium-contaminated equipment and gloveboxes were removed from Laboratories 15, 16, 43, 44, 50, 53, and 54 and the associated "offices" in 1971. These areas were decontaminated to remove smearable activity and then painted and new floor tile installed. A second phase of glovebox removal was performed in 1979-80 in Laboratories 17, 19, and 20. Ten gloveboxes were retained in a newly formed Laboratory 17A. No painting was done at this time (except for the new walls of 17A), but new floor tile was installed in Laboratories 17 and 19.

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BUILDING "C"
SCALE 1"=25'
REVISED MAR 1980

FIGURE 2-1. BUILDING C CONSTRUCTION SEQUENCE

Uranium fuel R&D was conducted in Laboratories 15, 16, 17, 19, 27, 43, 44, 50, 53, and 54. Projects were completed in 1980 in Laboratories 43, 44, 50, 53, and 54; 1982 in Laboratories 19 and 27; and March 1983 in Laboratories 15, 16, and 17.

3.0 CURRENT STATUS OF BUILDING C

The types of nuclear fuel R&D activities conducted in Building C were described in Section 2.0. No work is being performed on either plutonium fuel or thorium fuel fabrication. The last planned uranium fuel R&D project has just been completed. Decontamination has been started in the older part of Building C (see Fig. 4-1). This portion is believed to be the most contaminated based on age and having been used for plutonium, thorium, and uranium R&D. The portion of the building to be decontaminated during Phase II has been used for both plutonium and uranium fuels R&D. (Phase I includes the decontamination of Labs 27, 43, 44, 50, 51, 52, 53, and 54 and existing offices that used to be parts of Labs 43, 44, and 50; Phase II, Labs 15, 16, and 17; and Phase III, Labs 19 and 20.) The last portion of the building to be decontaminated was used primarily for plutonium fuels R&D.

The only residual radioactive contamination in Building C will be alpha contamination from the projects conducted with plutonium, uranium, and thorium. The maximum amount of surface contamination is expected to be about 20,000 dpm/100 cm² based upon data collected during the years projects were being conducted. No significant beta-gamma contamination should be present in Building C since work with beta-gamma emitters was not conducted in this facility. Maximum residual radioactivity in the soil under Building C will be less than 10 nCiPu per gram of soil based upon soil cores taken late in 1982.

4.0 DESCRIPTION OF PLANNED WORK

4.1 DEFINITIONS

4.1.1 Decommissioning

Decommissioning means an action taken by the U.S. Nuclear Regulatory Commission (NRC) that results in the facility or a portion thereof being released for unrestricted use and termination of control of License SNM-778 for the facility or portion thereof.

4.1.2 Decontamination

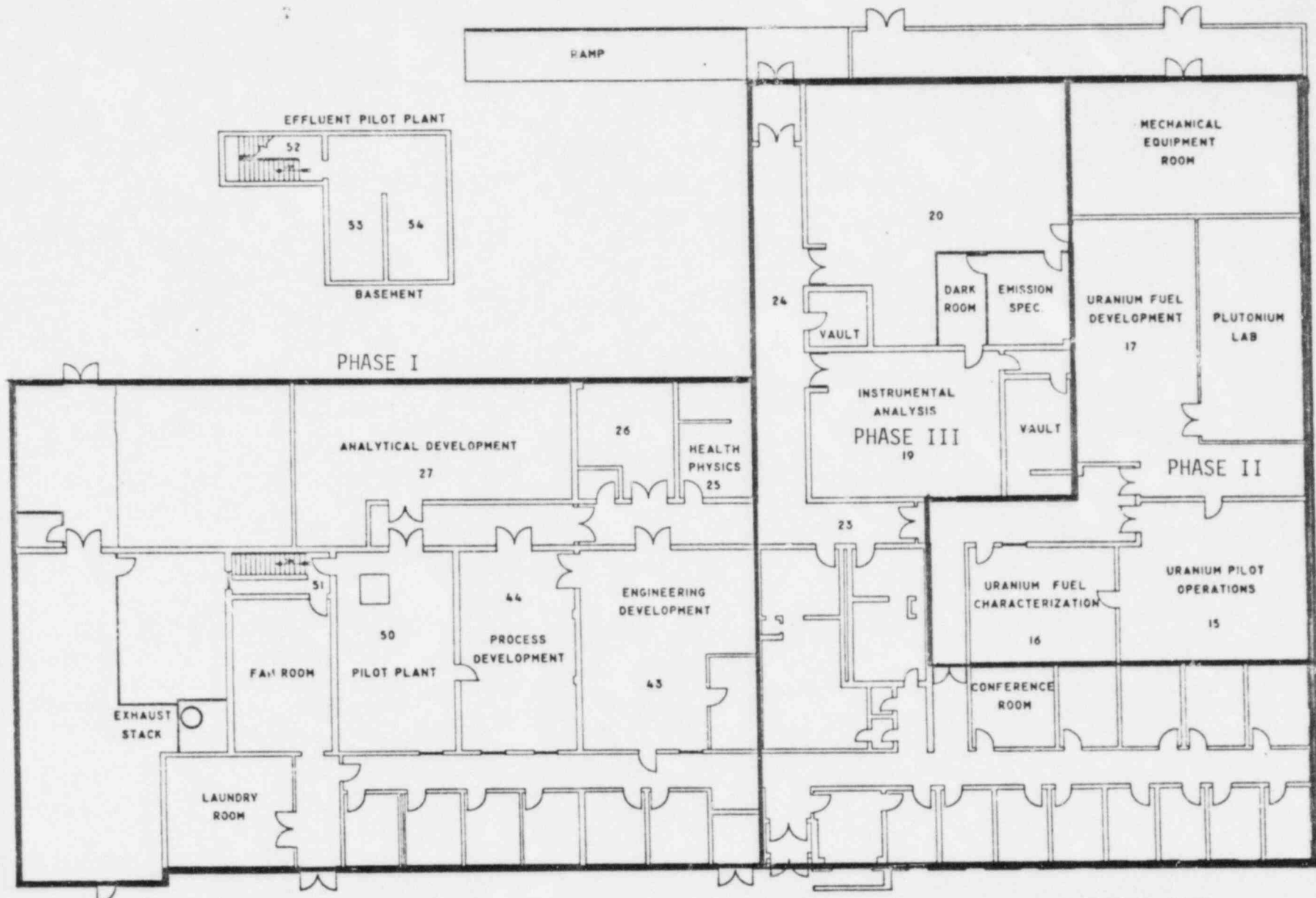
Decontamination means the removal or reduction of radioactive contamination from buildings, walls, floors, and equipment and of radioactively contaminated soil from around and beneath buildings.

4.2 OBJECTIVE

The objective of the Building C decommissioning project is to decontaminate the building to levels that will enable the NRC to release it for unrestricted use and terminate licensed control of it. The project is divided into three phases with a different section of the building being decontaminated and decommissioned in each phase. (See Fig. 4-1.) Each phase will take about one year with a NRC verification inspection and release as the final milestones in each phase.

All decontamination activities will be conducted in accordance with the plan. Procedures that implement this plan shall be in writing and approved by the Facility Supervisor and the Safety Review Committee. This plan was developed to assure that the facility will be decontaminated in a safe and orderly manner that will not be inimical to the common defense and security or to the health and safety of the public. A Quality Assurance Plan has been developed for this project that assures that surveys will be performed properly, that only properly calibrated instruments will be used, that standards used in connection with this project are traceable, and that data will be adequately documented and retained.

4-2



BUILDING "C"
SCALE: 1"=25'
REVISED MAR 1980

FIGURE 4-1. BUILDING C DECONTAMINATION PHASES

The QA Plan is available at the LRC for review. Services such as drains and ventilation may not be provided in all areas of Building C as described in the license during decontamination of the building. These services will be maintained, however, to prevent release of contamination to uncontrolled areas and to provide personnel safety.

4.3 TASKS

4.3.1 Preparation of Area for Decontamination

Loose equipment and material will be surveyed for contamination. The material and equipment will be released for unrestricted use if the surface contamination limits specified in Section 5.1 are met. If the survey results exceed the above specified limits, the equipment or material will be decontaminated to meet the limits or disposed of as contaminated waste.

Fixed equipment will be surveyed for contamination. The equipment will be released for unrestricted use if the surface contamination limits specified in Section 5.1 are met. If the surveyed equipment exceeds these limits, it will be decontaminated to meet the limits or the contaminated portions will be removed and disposed of as contaminated waste.

4.3.2 Pipe and Duct Decontamination

Service lines, electrical conduit and HVAC ducts that will interfere with a comprehensive survey of laboratory surfaces will be removed. Paint will be removed from the pipe and ducts. Dismantled, clean pipe will be surveyed for contamination and released as clean scrap if the limits specified in Section 5.1 are met. Dismantled, clean pipe which exceeds these limits will be disposed of as contaminated waste. Cleaned ducts will be surveyed for contamination and reinstalled if the limits specified in Section 5.1 are met. Ducts that exceed these limits will be cleaned to meet the limits or disposed of as contaminated waste.

4.3.3 Room Decontamination

Rooms within Building C will be decontaminated to meet the limits specified in Section 5.1. The paint will be removed from walls and ceilings down to the

block surface or the original coat of paint that was applied before the room was used for R&D utilizing licensed material. Walls that were erected after the cessation of R&D utilizing licensed material will not be cleaned but will be surveyed. Areas where licensed material has not been handled will be neither cleaned nor surveyed. The floor tile and tile cement will be removed to expose the bare concrete. Paint chips, floor tile and other waste will be stored in sealed containers.

4.3.4 Drain Line Removal

Drain lines in Building C were installed beneath the floor. Floor tile and tile cement will be removed. The concrete floor will be surveyed and decontaminated to meet the Section 5.1 criteria. The floor will then be removed above each drain line, surveyed and discarded if it meets the Section 5.1 criteria. If this debris exceeds the Section 5.1 limits, it will be disposed of as contaminated waste. Sub-surface soil will be removed to expose the drain line. The soil will be temporarily stored in sealed containers. Each drain line will be removed, surveyed, cut into sections, and disposed of as contaminated waste if it exceeds Section 5.1 limits. Precautions will be taken during drain line removal to prevent spillage of any material from the drain line onto the soil. Loose material shall be removed from the ditches and stored in sealed containers. All soil excavated from beneath Building C will be temporarily stored in sealed containers. Excavated soil found to exceed the Section 5.2 criteria will be disposed of as contaminated waste.

4.3.5 Surveys

The interior walls, ceiling, and floor in each room will be marked off into grids. Each grid section will comprise less than one square meter. A direct alpha survey will be conducted over 100% of the gridded area, including ledges and crevices, and a smear of a 100 cm² area will be taken in each grid for alpha measurement. Five floor grid sections will be randomly selected in each room for beta-gamma measurement. A direct beta and direct gamma survey will be made in each of the five selected grids. A smear will be taken over 100 cm² in each of five grids for the beta-gamma measurement. Areas found to exceed the Section 5.1 criteria will be cleaned to meet the criteria. No coatings will be used to cover contamination.

Soil samples will be taken along the entire length of ditches from which drain lines were removed. These samples will be analyzed for plutonium, uranium, and thorium. These samples will be disposed of after analysis.

4.3.6 Facility Restoration

Complete facility restoration will not begin until the NRC has completed its confirmatory survey and released the area, but partial restoration may be initiated if it will not hinder the NRC's verification inspection. Following successful NRC verification, the decontaminated portion of Building C will be isolated from the remaining portion in order to prevent recontamination.

4.3.7 Waste Disposal

Radioactive waste is currently stored in accordance with License SNM-778. A substantial amount of contaminated waste is expected to be generated from this project. Due to the limited amount of indoor waste storage area, it may be necessary to store a limited number of drums, each containing less than 100 nCiPu/gram of waste, out of doors awaiting off-site shipment. All contaminated waste will be shipped to a licensed disposal facility in accordance with applicable requirements in 10 CFR 71, 49 CFR, and the disposal facility license conditions.

Uncontaminated waste generated from this project will be released for local disposal.

4.4 RECORDS AND REPORTS

Monthly reports will be prepared that describe project activities during the previous month including schedule and budget information. These reports will be assembled in a project file along with pertinent correspondence and information.

Data sheets from building surveys and route sheets from sample analyses, required by the Quality Assurance Plan, will be assembled in notebooks. These notebooks will be preserved on microfilm. All project information and records will be available for review by the NRC. Records will be preserved for 40 years

after the completion date of the project. Following completion of each of the three phases mentioned in Section 4.2, a survey report will be submitted to the NRC to describe the scope of the survey, survey procedures used, and survey results. Each survey report will be accompanied with a request for the NRC to perform a verification survey and to release the areas covered for unrestricted use.

5.0 APPLICABLE CRITERIA

5.1 SURFACE CONTAMINATION

Table 5-1 specifies the radioactivity limits that will be used to determine when facilities and equipment surfaces can be released for unrestricted use. These limits were obtained from Regulatory Guide 1.86, "Termination of Operating Licenses for Nuclear Reactors" Revision 0, June 1974 and are contained in Annex C to SNM License 778.

5.2 SOIL CONTAMINATION

Table 5-2 specifies the residual soil contamination limits that will be used to determine when Building C can be released for unrestricted use. These limits were obtained from "Standard Review Plan for Termination of Special Nuclear Material Licenses."

Table 5-1. Acceptable Surface Contamination Levels

Nuclide ^a	Average ^{b c}	Maximum ^{b d}	Removable ^{b e}
U-nat, U-235, U-238, and associated decay products	5000 dpm α /100 cm ²	15,000 dpm α /100 cm ²	1000 dpm α /100 cm ²
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100 dpm/100 cm ²	300 dpm/100 cm ²	20 dpm/100 cm ²
Th-nat, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1000 dpm/100 cm ²	3000 dpm/200 cm ²	200 dpm/100 cm ²
Beta-gamma emitters (nuclides with decay modes other than alpha emission or spontaneous fission) except Sr-90 and others noted above.	5000 dpm β - γ /100 cm ²	15,000 dpm β - γ /100 cm ²	1000 dpm β - γ /100 cm ²

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^aWhere surface contamination by both alpha- and beta-gamma-emitting nuclides exists, the limits established for alpha- and beta-gamma-emitting nuclides should apply independently.

^bAs used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute observed by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

^cMeasurements of average contaminant should not be averaged over more than 1 square meter. For objects of less surface area, the average should be derived for each such object.

^dThe maximum contamination level applies to an area of not more than 100 cm².

^eThe amount of removable radioactive material per 100 cm² of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and assessing the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of less surface area is determined, the pertinent levels should be reduced proportionally and the entire surface should be wiped.

SOURCE: Regulatory Guide 1.86, "Termination of Operating Licenses for Nuclear Reactors," Revision 0, June 1974.

Table 5-2. Acceptable Soil Contamination Levels

<u>Material</u>	<u>Soil Concentration Level, pCi/gm of soil</u>
Enriched Uranium, Soluble or Insoluble	30
Plutonium Compounds	25
Thorium Compounds	10

6.0 SAFETY

Nuclear criticality safety is not a consideration of this plan because all accountable amounts of SNM will be removed from the areas prior to decontamination.

Neither shielding nor remote handling methods are planned because the R&D performed in Building C was with plutonium, thorium, and uranium fuels.

All decontamination work will be performed in accordance with health physics procedures contained in "Lynchburg Research Center Building C Area Operating Procedures" and in accordance with "Lynchburg Research Center Industrial Safety Manual." The supervisor of Health and Safety will oversee surveillance for the work. All operations will be supervised and conducted according to approved plans and procedures to assure that no exposure in excess of 10 CFR 20 limits will occur. To assure that all operations are conducted safely through the decommissioning process and to assure that the facility will be left in a safe condition, all plans and decontamination work shall be reviewed and approved by the Safety Review Committee.

7.0 ENVIRONMENTAL CONSIDERATIONS

7.1 ENVIRONMENTAL IMPACT

Surfaces in Building C will be decontaminated to levels consistent with Table 5-1 and underlying soil will be decontaminated to levels consistent with Table 5-2. The contaminated waste material produced during the decontamination of Building C will be sent to a licensed burial ground. If any decontamination liquids are generated, they will be handled in accordance with 10 CFR Part 20 requirements and procedures presently in existence at the LRC. Therefore, decontamination of Building C will cause no significant environmental impact.

7.2 ALTERNATIVES TO DECONTAMINATION OF BUILDING C

B&W is discontinuing nuclear fuel fabrication activities at the LRC. These activities were being performed in Building C. Three alternatives were considered for disposition of Building C:

- (1) Custodial care
- (2) Renovation
- (3) Decontamination and decommissioning

The first step in preparing Building C for custodial care would be to remove useful equipment for installation elsewhere on the site. Building C would then be cleaned and mopped with remaining equipment placed in stand-by. Building services (ventilation, off-gas, criticality monitors, etc.) would be maintained. The building would not be entered except for health physics surveys and maintenance. Decontamination would be deferred to some future date. This alternative is unattractive because of the risk involved in leaving a contaminated facility unattended. Since contaminated building surfaces and contaminated sub-floor soil would remain, contamination could be released by acts of nature (tornado, earthquake, etc.) or by vandalism or sabotage. Also, deferral of decontamination includes the risk of unavailability of personnel skilled in decontaminating our type of facility. Therefore, custodial care is an unattractive alternative.

8.0 ADMINISTRATIVE REQUIREMENTS

8.1 ORGANIZATION

The line organization shown in Figure 8-1 will be responsible for decontaminating Building C. Personnel experienced in handling radioactive materials will be responsible for the decontamination and dismantling operation. The minimum requirements for key personnel in the line organizational chart are given in the following paragraphs.

Manager, Plutonium Decontamination – The Plutonium Decontamination Project Manager is appointed by the Director of the LRC based upon demonstrated technical experience with nuclear fuels and project management experience and reports to the Director of the LRC. The Project Manager shall be an Authorized User of radioactive materials and shall be responsible for the safe decontamination and decommissioning of Building C according to the limits prescribed by the license, the requirements of this decontamination plan, and an established schedule and budget. The Project Manager has overall responsibility for the successful decontamination and decommissioning of Building C.

Facilities Manager – The Facilities Manager is appointed in accordance with Company policy.

Supervisor, Plant Engineering – The Supervisor, Plant Engineering shall have a B.S. degree in mechanical or civil engineering or equivalent experience and a minimum of 20 years related applicable experience. The Plant Engineering Supervisor shall be responsible for determining and applying expedient and efficient procedures for the decontamination of Building C including timely supply of personnel and equipment.

Supervisor, Health and Safety – The Supervisor, Health and Safety shall have a B.S. degree in physics or engineering, 2 years post-graduate work or equivalent training in nuclear health physics, and 6-8 years of health physics experience. The Health and Safety Supervisor directs the Building C cleanup technicians and advises the Project Manager on matters concerning radiological safety.

The first step in preparing Building C for renovation would be to remove all laboratory equipment. Contaminated equipment would be cleaned for unrestricted use or shipped off-site for disposal as contaminated waste. Building C would then be cleaned and mopped. This would be followed by painting and installation of new floor tile to assure any residual contamination was fixed in place. Laboratories and offices would then be reoccupied. Decontamination would be deferred to some future date. This alternative is also unattractive because of the same risks as custodial care; i.e., potential uncontrolled release of contamination from fixed contamination on building surfaces and sub-floor soil and unavailability of personnel skilled in decontaminating our type of facility.

The first step in preparing Building C for decontamination and decommissioning would be to remove all laboratory equipment from Building C. Contaminated equipment would be cleaned for unrestricted use or shipped off-site for disposal as contaminated waste. Building C would then be decontaminated including underlying soil. This would be followed by refurbishment and reoccupation. This is the only reasonable alternative.

7.3 COSTS AND BENEFITS

The estimated cost of decontaminating and decommissioning Building C could reach \$3,000,000 depending upon disposal costs for contaminated waste. Deferral of decontamination would lead to escalated costs because of inflation and use of less skilled people than currently available. No benefits would be derived by deferring decontamination. However, decontamination and decommissioning would eliminate the risk involved in retaining a contaminated facility.

7.4 CONCLUSIONS

The LRC concludes that there will be no significant environmental impact associated with decontamination and decommissioning of Building C, and that no environmental impact statement is required for this activity.

Foreman, Building C Cleanup – The Foreman, Building C Cleanup is appointed by the Facilities Manager based upon demonstrated experience in maintenance, repair, dismantling, and decontamination. The Foreman shall be responsible for performance of the work crew in accomplishing the daily tasks.

Facility Supervisor – The Facility Supervisor is appointed by the Director of the LRC and is responsible to the Director for the safe conduct of all operations at the LRC and for ensuring that these operations are conducted in accordance with all license conditions. The Facility Supervisor must have a minimum of 3 years related experience and either hold a degree in his related work or have 5 years of experience in the use and handling of licensed material.

Safety Review Committee – The SRC shall be comprised of at least five technical members having expertise in chemistry, radiological safety, industrial safety and nuclear engineering. The members shall be appointed by the Director, who shall also appoint the Chairman, alternate chairman and the members who serve on the Audit Subcommittee. The Director shall appoint a management representative to be a committee member who shall serve as the committee coordinator.

8.2 REVIEW AND AUDIT

- (a) A Safety Review Committee shall review and audit Building C decontamination and decommissioning operations and advise the Director of the LRC in matters relating to the health and safety of the public and the safety of the decontamination.
- (b) The Safety Review Committee shall have at least five members of whom no more than the minority shall be from the line organization shown in Figure 8-1. The Committee shall be made up of senior personnel who shall collectively provide experience in nuclear engineering, chemistry and radio-chemistry, radiological safety, and mechanical and electrical systems.
- (c) The Committee shall meet at least once every 4 months.
- (d) The quorum shall consist of not less than a majority of the full Committee and shall include the chairman or his designated alternate.

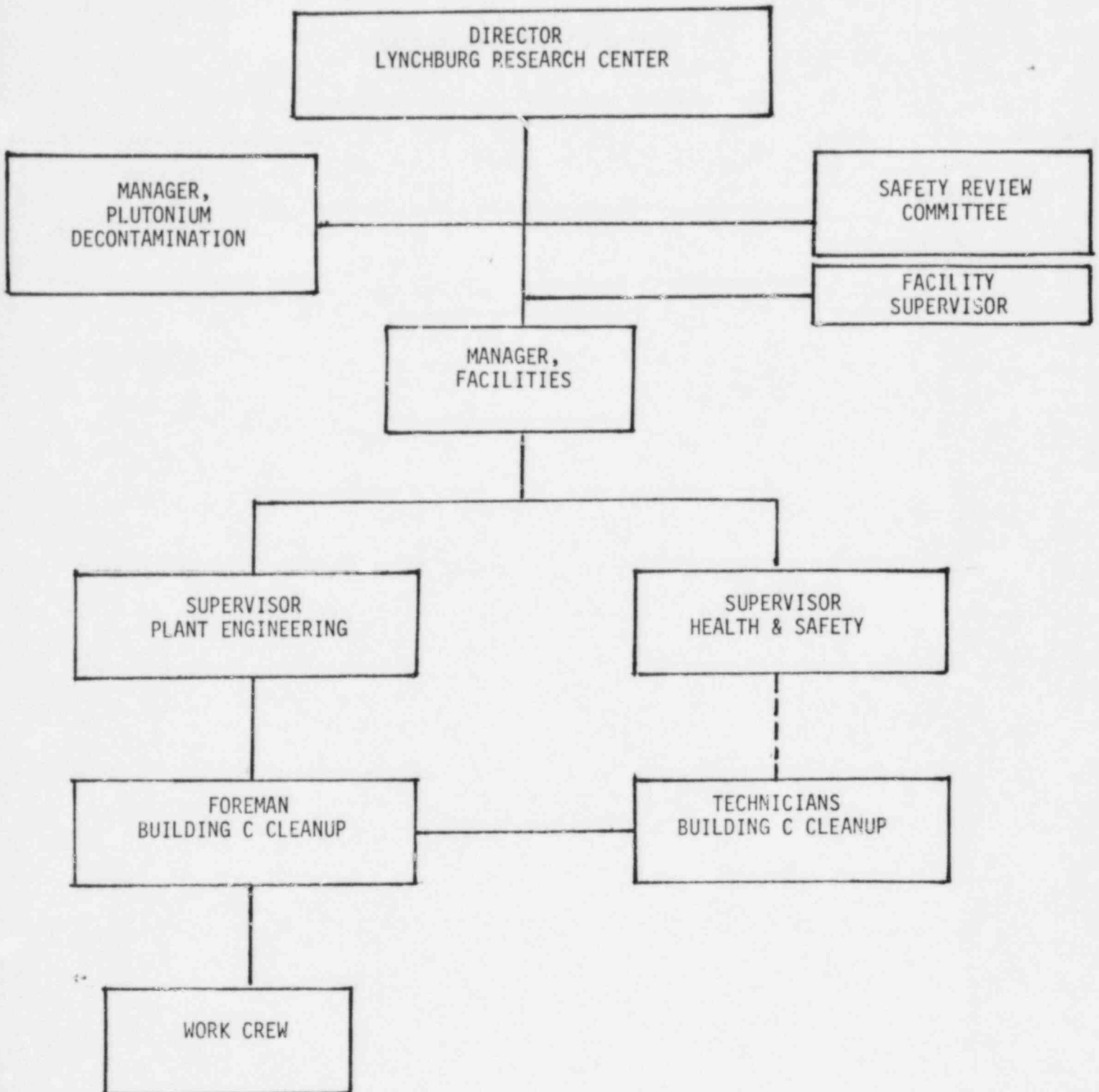


FIGURE 8-1. LINE ORGANIZATION CHART FOR DECOMMISSIONING BUILDING C

(e) Minutes of each Committee meeting shall be distributed to the Director of the LRC, all Safety Review Committee members, and such others as the chairman may designate.

(f) The Safety Review Committee shall:

- Review and approve proposed Area Operating Procedures and proposed changes to Area Operating Procedures.
- Audit decontamination operations and records for compliance with internal rules, procedures and regulations and with licensed provisions including the requirements of this decommissioning plan.

8.3 PROCEDURES

Written procedures, including applicable check lists, reviewed and approved by the Safety Review Committee shall be in effect and followed for the following operations:

- (a) Removing contaminated vacuum lines.
- (b) Removing paint from ceilings and walls.
- (c) Removing floor tile/covering and tile cement.
- (d) Building C Area Operating Procedures.

Substantive changes to the above procedures shall be made only with the approval of the Safety Review Committee. The temporary changes to the procedures that do not change their original intent may be made with the approval of the Plutonium Decontamination Project Manager. All such temporary changes to the procedures shall be documented and subsequently reviewed by the Safety Review Committee.

Written technical procedures, including applicable data sheets, reviewed and approved for Quality Assurance shall be in effect and followed for the following operations:

- (a) Survey of decontaminated area in Building C for fixed and smearable contamination.
- (b) Calibration of the fixed contamination counter.
- (c) Calibration of the smear counter.
- (d) Sampling soil under Building C.

- (e) Preparation and counting of samples for plutonium, uranium, and thorium content.
- (f) Calibration of the soil counter.

8.4 RECORD RETENTION

The following records and logs shall be prepared and retained at the LRC:

- (a) Tests, checks, and measurements documenting compliance with decontamination surveillance requirements.
- (b) Abnormal occurrences.
- (c) Records of radioactive shipments.
- (d) Minutes of Safety Review Committee Meetings.

9.0 SUMMARY

Building C at Babcock & Wilcox's Lynchburg Research Center has been used for nuclear fuels R&D since 1962. B&W is discontinuing nuclear fuel fabrication activities at the LRC and plans to decontaminate Building C so it can be released for unrestricted use. All contaminated equipment and materials will be decontaminated for unrestricted use or will be shipped off-site for disposal in a licensed burial site. Building surfaces will be decontaminated to meet acceptable NRC surface contamination levels and underlying contaminated soil will be excavated to meet acceptable NRC soil contamination levels. Contaminated waste from building and soil decontamination will be shipped off-site for disposal in a licensed burial site. There will be no significant environmental impact associated with decontamination of Building C.

DOCKET NO. _____ DOCKET NO. 70-824
 CONTROL NO. _____ CONTROL NO. 22166
 DATE OF DOC. _____ DATE OF DOC. 03/28/83
 DATE RCVD _____ DATE RCVD 04/18/83
 FCUF _____ ✓ PDR _____ ✓
 FCAF _____ LPDR _____
 WM _____ I&E REF. _____ ✓
 WMUR _____ SAFEGUARDS _____ ✓
 FCTC _____ OTHER _____

DESCRIPTION:

DESCRIPTION:

_____ The attached
 _____ decommissioning
 _____ plan for
 _____ Building C

INITIAL _____ 04/19/83 INITIAL cec

"LICENSE AMENDMENTS"

Docket No. 70-824

William G. Miller License Fee Management Branch, ADM

MATERIALS LICENSE AMENDMENT CLASSIFICATION

Applicant: B&W
 License No: SNM-778 Fee Category: _____
 Application Dated: 3/28/83 Received: 4/1/83
 Applicant's Classification: info only

The above application for amendment has been reviewed by NMSS in accordance with §170.31 of Part 170, and is classified as follows:

1. Safety and Environmental Amendments to Licenses in Fee Categories 1A through 1H, 2A, 2B, 2C, and 4A
 - (a) ___ Major safety and environmental
 - (b) ___ Minor safety and environmental
 - (c) ___ Safety and environmental (Categories 1D through 1G only)
 - (d) ___ Administrative

2. Justification for reclassification: _____

3. The application was filed (a) ___ pursuant to written NRC request and the amendment is being issued for the convenience of the Commission, or (b) Other (State reason): _____

RECEIVED
 APR 13 1983
 12

Info only

Signature W. G. Miller
 Division of Fuel Cycle & Material Safety
 Date 4/1/83

DOCKET NO. 70-824
CONTROL NO. 22166
DATE OF DOC. 03/28/83
DATE RCVD. 04/18/83
FCUF PDR
FCAF _____ LPDR _____
WM _____ I&E REF.
WMUR _____ SAFEGUARDS
FCTC _____ OTHER _____

DESCRIPTION:

The attached
decommissioning plan
for Building C

04/19/83 INITIAL cec