

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

June 14, 1994
LIC-94-0106

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: May 1994 Monthly Operating Report (MOR)

Enclosed is the May 1994 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/mah

Enclosures

- c: LeBoeuf, Lamb, Greene & MacRae
- L. J. Callan, NRC Regional Administrator, Region IV
- S. D. Bloom, NRC Project Manager
- R. P. Mullikin, NRC Senior Resident Inspector
- R. T. Pearce, Combustion Engineering
- R. J. Simon, Westinghouse
- Office of Management & Program Analysis (2)
- INPO Records Center

9406200341 940531
PDR ADOCK 05000285
R PDR

210079

IE24
1/1

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

MAY 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of May, the station operated at a nominal 100% power level. The spent fuel pool reracking project continued.

The condensate cooler was cleaned and returned to service.

Technical Specification (TS) 2.19 on fire protection has been removed from the TSs. Fire protection requirements are now contained within Standing Orders G-102, "Fire Protection Program Plan" and G-103, "Fire Protection Operability Criteria and Surveillance Requirements."

New setpoints were put into the Power Operated Relief Valve (PORV)/Low Temperature Overpressure Protection (LTOP) circuitry to allow plant operation through 20 Effective Full Power Years (EFPY) of operation.

On May 16, Reactor Protection System (RPS) Channel A Thermal Margin/Low Pressure (TM/LP) and Axial Power Distribution (APD) trip units failed to the tripped condition. A 48 hour Limiting Condition for Operation (LCO) was entered per Technical Specification 2.15(1) and trip units for Channel A high power, TM/LP, and APD were bypassed. The problem was traced to a loose connection in the APD calculator and was repaired. The affected trip units were declared operable and the LCO was exited the same day.

On May 26, a Swagelock outlet fitting from the secondary system hydrazine tote was found to be not fully engaged and leaking. Access to the area was controlled and the spill was cleaned up by Hazardous Material personnel. A notification was made to the State of Nebraska with a follow-up 4-hour notification made to the NRC due to the spill of hazardous material.

The following NRC inspections were completed during this reporting period:

<u>IER No.</u>	<u>Description</u>
94-12	Monthly Resident Inspection
94-15	Engineering/Safety Assessment & Quality Verification/SALP Cycle Closeout Team Inspection

The following Licensee Event Reports were submitted during this reporting period:

LER No. Description

- | | |
|--------|--|
| 94-004 | Inoperability of Halon Gas Fire Suppression System Due to Inoperable Fire Damper |
| 94-005 | Failure to Appropriately Address Out-of-Tolerance Test Results for Snubbers. |

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of May, no PORV or primary safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

RCS leak rate was steady throughout the month of May. The leak rate was a nominal 0.10 gpm, with no degrading trends noted. The only changes observed were due to normal plant transients.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No. Description

NONE

5. SIGNIFICANT SAFETY RELATED MAINTENANCE

- Replaced closing springs with those of a more reliable design on Breakers 1A4-12/Raw Water Pump AC-100, 1A4-1/Diesel Generator #2 Feed to Bus 1A4, and 1A4-15/Feeder for Lighting Transformer TIC-4A.
- Installed a new size orifice plate to reduce system vibration on Component Cooling Water Heat Exchanger AC-1B.
- Installed the rebuilt Raw Water Pump AC-10D and replaced the lugs on the Raw Water Pump Motor AC-10D-M.
- Replaced the failed multiplier/divider module on Nuclear Instrumentation and Reactor Protective System Channel A (AI-31A).
- Installed the rebuilt Backwash Control Valve HCV-2805B for Raw Water Strainer AC-12B.
- Replaced a failed solenoid on Component Cooling Water Outlet Valve HCV-403D for Containment Cooling Coil VA-8B.
- Installed and filled a Furmanite containment box on the leaking Swagelock fitting upstream of Main Steam Warmup Line Isolation Valve MS-367 for Auxiliary Feedwater Pump FW-10.

LIC-94-0106
Enclosure
Page 3

- Replaced the CR-120A relays with those of a more reliable design in Sequencer 52-1 circuits 3, 4, 5, and 6 and Sequencer 52-2 circuits 3, 4, 5, and 6.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
 UNIT FORT CALHOUN STATION
 DATE JUNE 06, 1994
 COMPLETED BY M. A. HOWMAN
 TELEPHONE 402-533-6939

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
 2. Reporting Period: MAY 1994

NOTES

3. Licensed Thermal Power (MWt): 1500
 4. Nameplate Rating (Gross MWe): 502
 5. Design Elec. Rating (Net MWe): 478
 6. Max. Dep. Capacity (Gross MWe): 502
 7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
 N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
 N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
	-----	-----	-----
11. Hours in Reporting Period.....	744.0	3623.0	181297.0
12. Number of Hours Reactor was Critical	744.0	3589.2	141280.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	3574.1	139637.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113428.5	5238740.2	184524539.8
17. Gross Elec. Energy Generated (MWH)..	373638.0	1772530.0	60860554.2
18. Net Elec. Energy Generated (MWH)....	356578.1	1691909.6	58064972.5
19. Unit Service Factor.....	100.0	98.7	77.0
20. Unit Availability Factor.....	100.0	98.7	77.0
21. Unit Capacity Factor (using MDC Net)	100.3	97.7	69.4
22. Unit Capacity Factor (using DER Net)	100.3	97.7	67.7
23. Unit Forced Outage Rate.....	.0	1.3	4.1

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
 NONE

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE JUNE 06, 1994
COMPLETED BY M. A. HOWMAN
TELEPHONE 402-533-6939

MONTH MAY 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	485	17	478
2	485	18	477
3	485	19	476
4	484	20	477
5	483	21	477
6	483	22	477
7	484	23	476
8	485	24	473
9	484	25	473
10	483	26	474
11	482	27	476
12	481	28	476
13	480	29	476
14	481	30	475
15	480	31	472
16	479		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun St.
 DATE June 6, 1994
 COMPLETED BY M. A. Howman
 TELEPHONE (402) 533-6939

REPORT MONTH May 1994

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None									During May 1994, the plant operated at a nominal 100% power.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending May 31, 1994

1. Scheduled date for next refueling shutdown. March 11, 1995
2. Scheduled date for restart following refueling. April 29, 1995
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? No
 - a. If answer is yes, what, in general, will these be? N/A
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. No
 - c. If no such review has taken place, when is it scheduled? Prior to April 1995
4. Scheduled date(s) for submitting proposed licensing action and support information. No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. **
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 570 Assemblies
 - c) spent fuel pool storage capacity 729 Assemblies
 - d) planned spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1995 Outage*

* Capability of full core offload of 133 assemblies lost. Reracking began in March and is scheduled for completion in August 1994.

** OPPD is utilizing the CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16.

Prepared by Keri Hallett Date 6-6-94