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Topical Report Review Status

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ABSTRACT

This report provides industry with procedures for submitting topical reports, guidance on how the U.S. Nuclear Regulatory Commission (NRC) will process and respond to topical report submissions, and an accounting of all topical reports currently being reviewed by the NRC staff.

This report will be published semiannually, and each sponsoring organization with one or more topical reports under review will receive copies.

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TOPICAL REPORT REVIEW STATUS

NUCLEAR REGULATORY COMMISSION LICENSING TOPICAL REPORT PROGRAM

Under the Nuclear Regulatory Commission (NRC) licensing topical report program, industry organizations may, on their own volition or at the request of the NRC staff, submit reports on specific safety-related subjects to the NRC and have them reviewed independently of any construction permit or operating license review. The purpose of this program is to minimize time and effort required of both industry and the NRC on subjects repeated in numerous licensing actions. It provides for a single review and approval with subsequent referencing, rather than repetitive reviews of the same subject. The reports already under review are given in the appendix.

LICENSING TOPICAL REPORT PROGRAM MANAGEMENT

A topical report program manager (PM) has been assigned to manage and coordinate the overall program. Each review branch with responsibility for the detailed technical review of a specific report will assign review specialists who will report to their own management on the detailed technical review matters.

PROCEDURE FOR SUBMITTING LICENSING TOPICAL REPORT

Submittal of Report for Review

A report (and requisite number of copies, as discussed below) submitted under the NRC licensing topical report program must be accompanied by a letter of transmittal. The first paragraph of this letter must include a statement that the applicant is submitting a licensing topical report under the NRC licensing topical report program for review and acceptance for referencing in licensing actions. This paragraph must specify the report identifier, title, and issue date. The transmittal letter and report should be addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; Document Control Desk; ATTN: Chief, Planning, Program and Management Support Branch. If a report is not sent to this address, there is no assurance that it will be entered into the agency's workload information and scheduling program database for tracking.

The transmittal letter should also include any pertinent information about the timeframe of the review and first referencing action. All correspondence with the NRC concerning licensing topical reports or special reports should include the report identifier specified by the submitting organization in the upper right-hand corner of the transmittal letter.

Submittals After Report Has Been Accepted for Review

Licensing topical reports accepted for review must be submitted in hard copy in the following quantities. Additional copies will be provided to the NRC staff upon request.

<u>Report Classification</u>	<u>Hard Copy</u>
Nonproprietary reports	23
Proprietary reports	
Proprietary version	23
Nonproprietary version	12

These quantity requirements pertain to all review submittals including initial submittals, revisions, supplements, responses to requests for additional information, and NRC-approved versions.

FORMAT OF LICENSING TOPICAL REPORT

Report Identifier

Each licensing topical report should have a unique alphanumeric identifier for filing and reference purposes that is assigned to it by the submitting organization. The report identifier should be typed in the upper right-hand corner of the first page or cover page of all documents relating to the report.

Each organization submitting licensing topical reports should consult with the NRC topical report PM to ensure that the identifier it selects does not conflict with that of another organization and to receive an NRC technical assistance control (TAC) number. The identifier for all proprietary reports should have "-P" or "(P)" at the end. Each nonproprietary version of a proprietary report should have the same identifier as the corresponding proprietary report except that "-NP" or "(NP)" should be included at the end. NRC-approved versions of topical reports should have "-A" after the identifier ("-P-A" for approved proprietary reports).

The initial submittal and all responses to NRC comments or requests for additional information regarding a specific licensing topical report should be identified by both the NRC TAC number and the submitting organization's identifier for that report followed by "Response to Comment," "Response to Request for Additional Information by the NRC Staff" or, if appropriate, "Response by Appendix or Supplement to the Original Report." The use of an addendum is discouraged. It is possible that only a revision to the original report can constitute a response. This is acceptable when appropriate. However, each revision will entail a new review with the old review closed out.

Abstract

The report must include an abstract no longer than one page that summarizes the contents of the report and the conclusions reached.

Introduction

The report should have an introductory section that gives the purpose of the report and clearly defines its scope and applicability. Any limitations or restrictions on the use of the report or its results or conclusions as determined by either the staff or the sponsoring organization should also be listed in this section.

Body of Report

The body of the report may be organized at the discretion of the sponsoring organization to suit its needs and the subject matter of the report. It is recommended that long tabulations of data such as test results, computer program descriptions, detailed technical analyses, or derivations be included as appendices.

References

The report should include a list of all pertinent references.

CRITERIA FOR ACCEPTING NEW LICENSING TOPICAL REPORTS FOR REVIEW

The topical report PM will screen reports submitted for review under the licensing topical program in accordance with the following criteria. No new report submitted by any organization will be accepted as a licensing topical report unless the PM, in conjunction with the cognizant technical review organization, determines that it qualifies as such. In making such a determination, all four of the following criteria should be met:

- (1) The report deals with a specific "safety-related" subject regarding a nuclear power plant that requires a safety assessment by the NRC staff, such as component design, analytical models or techniques, or performance testing of components and/or systems that can be evaluated independently of any specific license application.
- (2) The report is, or is expected to be, referenced in a number of license or standardized reference design approval applications.
- (3) The report contains complete and detailed information on the specific subject presented. Conceptual or incomplete preliminary information will not be reviewed.
- (4) NRC approval of the report will result in increased efficiency of the review process for applications that reference the report.

Licensing topical reports addressing a new design or procedure not currently addressed in any license or standardized reference design approval application will require special consideration. In support of such a report, the sponsoring organization must submit documentation from at least two potential applicants of their intent to reference the report before NRC commits resources to perform the review. It is not the intent of the licensing topical report program to provide engineering evaluations of feasibility, economics, or desirability of material submitted that could directly or indirectly be used in sales promotion efforts.

Exceptions to these acceptance criteria may be allowed. These exceptions must be based on a staff decision that the evaluation of a proposed licensing topical report is in the public interest. The sponsoring organization should provide the staff with information that justifies such exceptions. This may include such items as cost savings to the industry if implemented, or a consideration of whether the evaluation would contribute to the formation of a broader base for

resolving a present or developing safety-related subject generally evidenced by experience from operating nuclear power plants. As an example, a proposed licensing topical report on a new pump design might be accepted for review, if experience at operating nuclear power plants indicated that currently installed pumps might be expected to infringe on the margins of safety to an unacceptable degree, and the new pump design might reasonably be expected to remedy the problem.

These exceptions will include a determination that NRC staff resources expended in the review of the licensing topical report will provide a foreseeable benefit equal to or greater than the impact from redirection and reduction of resources committed to other ongoing license actions, licensing topical report reviews, or generic technical issues.

When a sponsoring organization is planning a report that it believes can qualify as a licensing topical report, it should contact the topical report PM well in advance of the planned submittal to determine if the proposed report will satisfy the definition of a licensing topical report and, if so, to clarify the requirements for preparing and submitting the report. A meeting might be appropriate to discuss these issues. The topical report PM, in consultation with the staff technical specialists and management, will make a timely determination on the qualifications of the proposed report. If the report qualifies as a licensing topical report, the sponsoring organization may then formally submit it for review.

The NRC staff may also find that addressing a safety-related matter in a licensing topical report is desirable. In such a case, NRC management will contact the sponsoring organization and discuss the preparation of such a report. If the organization agrees to prepare it, they will formally submit it for review.

Other types of reports that do not meet the criteria for licensing topical reports may be submitted as special reports.

HANDLING OF PROPRIETARY REPORTS

When an organization submits a licensing topical report for NRC staff review pursuant to this NUREG series report and believes that portions of the submittal contain confidential business (proprietary) information, it must submit at least 12 copies of a nonproprietary version of the proprietary report at the same time it submits the proprietary report. The proprietary report must show which portions of the document are proprietary by the use of italics, margin lines, underscoring, or by bracketing the proprietary material. A nonproprietary report should show what information has been deleted from the proprietary version because the submitting organization considers it to be proprietary. For example, if brackets are used in the proprietary version to show what portion of the information is considered proprietary, the nonproprietary version should have a blank portion between the brackets to show that information was removed. Upon submittal of a properly marked licensing topical report (see 10 CFR 2.790) considered and justified as proprietary, the NRC staff will make a determination of whether or not it is proprietary and notify the submitting organization accordingly.

CORRESPONDENCE FROM SPONSORING ORGANIZATION PERTAINING TO REPORT

All correspondence regarding licensing topical reports should be addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; Document Control Desk; ATTN: Chief, Planning, Program and Management Support Branch, and should contain both the appropriate report identifier and NRC TAC number in the upper right-hand corner.

STAFF REQUESTS FOR ADDITIONAL INFORMATION DURING REVIEW CYCLE

The NRC staff can make requests for additional information on a licensing topical report under review either during a meeting or by letter. The Chief of the technical review branch will transmit to the sponsoring organization written requests for additional information on licensing topical reports. The sponsoring organization will submit its responses to these requests and to staff comments during the review as information supplementary to the original report. The technical review branch will schedule meetings with the sponsoring organization to discuss the contents of a licensing topical report.

TRANSMITTAL OF STAFF EVALUATION OF REPORT

When the staff determines that a licensing topical report is acceptable for referencing, it will give the extent of and the conditions for acceptance, if any, in a letter transmitting the results of the evaluation. For a proprietary report, the transmittal letter will state that both proprietary and nonproprietary versions must be referenced in future license applications. The staff evaluation and letter of transmittal will be incorporated in the accepted version of the report when submitted.

PUBLICATION OF APPROVED VERSION OF REPORT

After the staff accepts a licensing topical report for referencing, the sponsoring organization should publish an approved version. A copy of the staff's transmittal letter and its evaluation report should be inserted immediately after the title page of the approved version.

PLACEMENT OF REPORT AND ASSOCIATED DOCUMENTS IN PUBLIC DOCUMENT ROOM

All nonproprietary correspondence regarding the review of licensing topical reports, all nonproprietary licensing topical reports, and all nonproprietary versions of proprietary reports will be placed in the NRC Public Document Room (PDR) located at 2120 L Street, N.W., Washington DC; telephone (202) 634-3273. The sponsoring organization and the NRC staff should exercise care to ensure that any information identified as proprietary by the sponsoring organization is not used in any of the above correspondence and in licensing topical report evaluations. Licensing topical reports are not placed in local PDRs; however, nonproprietary versions are available on request.

REVISIONS TO REPORT

After a licensing topical report has been found acceptable for referencing, the sponsoring organization occasionally might need to add supplementary information

to keep it up to date. This is different from the situation where the report has become obsolete and must be withdrawn. A proposed revision to an approved report is submitted to and reviewed by the NRC staff in accordance with the same requirements and procedures as those that apply to a new licensing topical report. The revision will have the same identifier as the base report with the suffix "Revision 1 or 2," etc. However, the NRC topical report PM will issue a new NRC TAC number. When a revision to an accepted report is submitted for review, the "-A" designation will no longer be valid until the revision itself has been reviewed and accepted.

When the staff determines that the revised or supplementary material is acceptable for referencing, the sponsoring organization will submit the required number of complete new reports incorporating the revised or supplementary information. The revised report must contain a note on the cover page stating that it supersedes and replaces all earlier versions of the same numbered report. The superseded reports will not be returned to the sponsoring organization.

WITHDRAWAL OF REPORT

After a certain amount of time has elapsed, a licensing topical report may become obsolete, as determined by either the NRC staff or the sponsoring organization, because it does not adequately address the current state of the technology, or it is no longer applicable because NRC criteria or regulations have changed, or for other reasons. In this event, the sponsoring organization may, on its own volition, request that the report be withdrawn or the NRC staff may advise the sponsoring organization to do so. On receipt of a letter from a sponsoring organization requesting withdrawal of a licensing topical report, the staff will categorize it as "withdrawn." Withdrawn reports are not acceptable as references in license applications. The report may be replaced by an updated or new licensing topical report that has a new identifier and new NRC TAC number to avoid confusion with the withdrawn report.

APPENDIX

LICENSING TOPICAL REPORTS UNDER REVIEW BY THE NRC STAFF
LIST OF VENDORS

BABCOCK & WILCOX

TaskID -----	Title -----
M04645	BAW-10060-NP - "REACTOR INTERNALS DESIGN/ANALYSIS FOR NORMAL UPSET & FAULTED CONDITIONS"
M75040	"BAW-2086, "B&W J-R TEST PROCEDURE FOR WEDGE OPENING LOADED (WOL) FRACTURE TOUGHNESS SPECIMENS""
M79766	BWROG - THE NEW ATWS/STABILITY CALCULATIONS
M81851	BAW-10084P. REV. 3, PROGRAM TO DETERMINE IN-REACTOR PERFORMANCE OF B&W FUELS. JULY 1991
M82189	BAW - 10183P FUEL ROD GAS PRESSURE CRITERION, JULY 1991
M83823	BWROG - BWR BELTLINE MATERIAL UPPER SHELF ENERGY ESTIMATION METHODS (GE-NE-523-18-1191)
M84643	BAW-10164P - REV.2 - ADVANCED COMPUTER PROGRAM FOR LIGHT WATER REACTOR LOCA AND NON-LOCA TRANSIENT ANALYSIS AUG.1992
M84800	BAW-10164P, REV. 3, RELAP/MOD 2, B&W REV. 3
M84985	BAW-10168 P, REV.2, RSG LOCA, BWNT LOSS-OF-COOLANT ACCIDENT EVALUATION MODEL FOR RECIRCULATING STEAM GENERATOR PLANTS
M85120	BAW-10186P, EXTENDED BURNUP EVALUATION, NOVEMBER 1993
M86196	BAW-2178 P, LOW UPPER-SHELF TOUGH FRAC MECHANICS ANAL OF REAC VES OF B&WOG REAC VES WORK GRP FOR LEVEL C&D SVC.
M87102	BAW-10191-P, STAR SYSTEM COMPONENTS FOR REACTOR PROTECTION SYSTEM DIGITAL UPGRADES.
M87732	BAW-2192-P - LOW UPPER SHELF TOUGHNESS FRACTURE ANALYSIS OF REA. VESSEL OF B&WOG RVWG FOR LEVEL A&B CONDITION
M87805	BAW-10189P, CHF TESTING AND ANALYSIS OF THE MARK-BW FUEL ASSEMBLE DESIGN, 9/93.
M88150	BAW-10168 NP - REV.3 - OCT. 1993 - RSG LOCA - BWNT LOSS-OF-COOLANT ACCIDENT EVALUATION MODEL RECIRCULATING STEM PLANTS
M88739	BAW-2181 NP, CRAFT 2 - RELAPS/MOD 2 - COMPARISON FOR SMALL BREAK LOCA ACCIDENT ANALYSIS.

BABCOCK & WILCOX

TaskID

Title

M88899

BAW-10187P, APPENDIX F

COMBUSTION ENGINEERING

TaskID -----	Title -----
M04685	CENPD-212-NP - "INSTRUMENT QUALIFICATION ENVIRONMENTAL QUALIFICATION OF C-E INSTRUMENTATION EQUIPMENT, PARTS ONE.
M47020	CENPD-172A-NP - "TR" PLANT PROTECTION SYSTEM WITH INTEGRATED AUTOMATIC TEST SYSTEM (SSPPS) INTERCONNECT DES
M72033	"CEN-381-P LOW TEMP OVERPRESSURIZATION TRANSIENT P-T LIMIT FOR DETERMINATION LOW TEMP OP PROTE
M75965	CEN RPA 89-112 ABB ATOM CONTROL ROD DROP ACCIDENT ANALYSIS METHODOLOGY FOR BWRS
M75966	CEN-00-000 RPA-89-053 P ABB ATOM HIGH WORTH CONTROL RODS FOR BWRS ROD DROP ACCIDENT ANALYSIS
M80895	ABB REPORT - RPA 89-053 "ABB HIGH WORTH RODS, ROD DROP ACCIDENT ANALYSIS"
M81374	CEN-403 - ESFAS SUBGROUP RELAY TEST INTERVAL EXTENSION
M82272	CEN-405-P, REV.1-P APPLI OF REACTOR VESS SURVEILL DATA FOR EMBRITT MGMT
M82718	CENPD--282P TECH MANUAL FOR CENTS CODE VOLUME 3
M85911	CENPD-282 (P), VOL. 4, TECHNICAL MANUAL FOR THE CENTS CODE
M86126	CENPD-283-P BOILING WATER REACTOR EMERGENCY CORE COOLING EVALUATION MODEL:CODE SENSITIVITY FOR SVEA-96 FUEL.
M86193	CEN-420-P, VOL. 1, PART 1, SMALL BREAK LOCA REALISTIC EVALUATION MODEL.
M87733	CEN-405-P REV. 2 - APPLICATION OF REACTOR VESSEL SURVEILLANCE DATA FOR EMBRITTLEMENT MANAGEMENT.
M87910	CENPD-289-P, JUSTIFYING USE OF INSERT REPLACEMENT RODS IN ABB CENF FUEL ASSEMBLIES.
M88025	CENPD-284 P, CONTROL ROD DROP ACCIDENT ANALYSIS METHODOLOGY FOR BWR'S: SUMMARY AND QUALIFICATION
M88029	CEN-420 P - VOL. 1 - SMALL BREAK LOCA REALISTIC EVALUATION

COMBUSTION ENGINEERING

TaskID

Title

M88447

CEN-420, P VOL. II, WITH SUPPLS. 1 AND 2 AND VOL. 3 SMALL
BREAK LOCA REALISTIC EVALUATION MODEL

DUKE POWER

TaskID	Title
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M85181	DPC-NF-2005-P THERMAL/HYDRAULIC STATISTICAL CORE DESIGN METHODOLOGY
M88082	DUKE-2007 P - FUEL RECONSTITUTION ANALYSIS METHODOLOGY

UTILITY GROUP FOR REG. APPLICATION

TaskID -----	Title -----
M80205	EPRI REPORT NP-6628 PROCEDURE FOR SEISMIC EVALUATION AND DESIGN OF SMALL BORE PIPING (NCIG-14)
M88026	EPRI-TR-102470 P"HIGH FREQUENCY SEISMIC EFFECTS"
M88898	EPRI MOV TOPICAL REPORT REVIEW

GENERAL ELECTRIC

TaskID -----	Title -----
M57536	NEDO-30832-NP - "TR: "ELIMITATION OF LIMIT ON BWR SUPPRES SION POOL TEMPERATURE FOR SRV DISCHARGE WITH QUEN"
M64221	NEDC-31336-P "GENERIC ELECTRIC INSTRUMENT SET POINT METHODO LOGY METHODOLOGY"
M77253	NEDE 2 - 24011-P-A, AMENDMENT 23 "GENERAL ELECTRIC STANDARD APPLICATION FOR REACTOR FUEL" (GESTAR-11)
M80309	NEDC-31951P - IMPLEMENTATION OF IMPROVED WATER CHEMISTRY AND TECHNICAL BASIS FOR REVIDED PIPING INSPECTION SCHEDULES
M81275	NEDO-31695 BWR SUPPERSSION POOL TEMPERATURE TECHNICAL SPECIFICATION LIMITS
M82663	NEDC 31984P, GENERIC EVAL. OF GE BWR POWER UPRATE, SUPPL 1, DTD 10/91
M82823	NEDE-32177-P GENERAL ELECTRIC CO. TRACG COMPUTER CODE QUALIFICATION
M83089	BWR'S GRP (GE RPT NEDC 32013P) SYSTEM ANALYSES FOR ELMINATION OF SELECTED RESPONSE TIMETESTING RREQUIREMENTS
M83533	GE COMPANY REVIEW OF QA TOPICAL REPORT NEDO-11209-04A, "GE NUCLEAR ENERGY QAPD," FROM ACCEPTANCE OF REV 9
M84984	NEDC-31984 P - SUPPLEMENT 2, GENERIC EVALUATIONS OF GENERAL ELECTRIC BOILING WATER REACTOR POWER UPRATE, OCTOBER 1992
M85776	NEDC-32160 P, BWR OWNERS' GROUP LICENSING TOPICAL REPORT CALIBRATION INTERVAL EXTENSION.
M85913	NEDE-32176 (P), TRACG, MODEL DESCRIPTION, 2/93.
M85915	NEDE-32178 (P), APPLICATION OF TRACG MODEL TO SBWR LICENSING SAFETY ANALYSES, 2/93.
M87103	NEDE-32177-P, TRACG QUALIFICATION, REVISION 1, JUNE 1993.
M87911	NEDC-31858-P, REV. 2, BWROG RPT FOR INCREASING M3IV LEAKAGE RATE LIMITS & ELIMINATION OF LEAKAGE CONTROL SYSTEMS.

SIEMENS

TaskID -----	Title -----
M73094	"ANF-929 SUP 2 EVALUATION OF RANGE OF APPLICABILITY OF HEAT TRANSFER COEFFICIENTS PROPOSED FOR 9X9
M80400	ANF-90-145(P) VOLS. 1 & 2 - RODEX3 FUEL ROD THERMAL-MECHANICAL RESPONSE EVALUATION MODEL, ADVANCED NUCLEAR FUEL CORP.
M81069	ANF-90-082(P) REV. 1 APPLICATION OF ANF DESIGN METHODOLOGY FOR FUEL ASSEMBLY RECONSTITUTION ADVANCE NUCLEAR FC, MAY '91
M81070	ANF-89-90(P) REV 1 GENERIC MECHANICAL DESIGN CRITERIA FOR BWR FUEL DESIGNS ADVANCED NUCLEAR FUEL CORP. APRIL 1991
M83302	XN-NF-82-49(P), REV.1, REVISED EXEM/PWR SMALL BREAK LOCA MODEL, SIEMENS NUCLEAR POWER CORP. SUPPLEMENT 1
M84245	EMF-92-116 P - GENERIC MECHANICAL DESIGN CRITERIA FOR PWR FUEL DESIGNS.
M84718	EMF-92-139(P), VOL. 3, REALISTIC LOCA ECCS EVALUATION MODEL ASSESSMENT FOR PWR LARGE BREAK LOCA ANALYSIS.
M84811	EMF-92-139(P), VOL. 2, S-RELAP 5 REALISTIC LOCA ECCS EVALUATION MODEL FOR PWR LARGE BREAK LOCA ANALYSIS.
M85182	EMF-92-139(P), VOLUME 1, "REALISTIC LOCA ECCS EVALUATION MODEL METHODOLOGY FOR PWR. LARGE BREAK LOCA ANALYSIS,"12/92
M86765	EMF-93-074 (P), GENERIC MECHANICAL LICENSING RPT FOR ADVANCED 17X17 FUEL DESIGN.
M87010	EMF-92-139(P), VOL 3 SUPPL 2, REAL LOCA ECCS EVAL MODEL ASSMT FOR PWR LARGE BREAK LOCA ANALYSIS ASSMT FOR CCTF TEST
M87011	EMF-92-139(P), VOL. 3, SUP 1, REAL LOCA ECCS EVAL MODEL ASSMT FOR PWR LARGE BREAK LOCA ANALYSIS FLECHT SEASET 31504
M87012	EMF-92-139-P, V4, REALISTIC LOCA ECCS EVAL MODEL FOR PWR LARGE BREAK LOCA ANALYSIS-UNCERTAINTY ANALYSES, 7/93
M87013	EMF-92-139-P, V3, S7, REALISTIC LOCA ECCS EVAL MODEL ASSMT FOR PWR LARGE BREAK LOCA ANALYSIS, W 4 LOOP PWR SAM PROBLEM
M87014	EMF-92-139-P, V3, S6, REALISTIC LOCA ECCS EVAL MODEL ASSMT FOR PWR LARGE BREAK LOCA ANALYSIS-W 3-LOOP PWR SAM PROBLEM, 5/93
M87015	EMF-92-139-P, V3, S5, REALISTIC LOCA ECCS EVAL MODEL ASSMT FOR PWR LARGE BREAK LOCA ANALYSIS ASSMT UPTF TEST 6&7, 6/93

SIEMENS

TaskID -----	Title -----
M87016	EMF-92-139-P V3,S4, REALISTIC LOCA ECCS EVAL MODEL ASMT FOR PWR LARGE BREAK LOCA ANALYSIS LOFT TEST L2-6, 6/93
M87017	EMF-92-139P V-3,S-3, REALISTIC LOCA ECCS EVAL MODEL ASMT FOR PWR LARGE BREAK LOCA ANALYSIS-ASMT LOFT L2-5, 6/93
M87174	EMF-074-SIEMENS POWER CORPORATION BWR STABILITY CODE, STAIF (EMF-CC-074(P), VOLUME 1)
M87906	EMF-074 (P), VOL. 2, STAIF: A COMPUTER PROGRAM FOR BWR STABILITY ANALYSIS-CODE QUALIFICATION RPT, 9/93
M87907	EMF-93-164(P), POWER DISTRIBUTION MEASUREMENT UNCERTAINTY FOR INPAX-W IN WESTINGHOUSE PLANTS, SEPT. 1993.
M89013	STAFF REVIEW OF REV. 27 TO SIEMENS' POWER CORPORATION NUCLEAR DIV. (SPC) QA TOPICAL EMF-1A, DATED 1/31/94.

STONE & WEBSTER

TaskID -----	Title -----
M04723	SWECO-7703-NP - "MISSILE-BARRIER INTERACTIONS"
M81528	STONE & WEBSTER QAPD(TOPICAL REPORT SWSQAP 1-74A) FROM COMPLETION OF REVIEW OF REVISION F THRU COMPLETION OF REVIEW OF

WESTINGHOUSE

TaskID -----	Title -----
M00789	WCAP-08163-NP - "REACTOR COOLANT PUMP INTEGRITY IN A LOCA"
M03000	WCAP-08687-P "ENVIRONMENTAL QUALIFICATION OF WESTINGHOUSE NSS CLASS IE EQUIPMENT"
M04246	WCAP-08510-NP "METHOD FOR FRACTURE MECHANICS ANALYSIS OF NUCLEAR REACTORS UNDER SEVERE THERMAL THRANSIENTS"
M04849	WCAP-09172-P - "AN N-16 TRANSIT TIME FLOW MEASUREMENT SYSTEM (TTFM) DESCRIPTION AND PERFORMANCE"
M04882	WCAP-09292-NP "DYNAMIC FRACTURE TOUGHNESS OF ASME SA508 CLASS 2A ASME SA533 GRADE A CLASS 2 BASE & HEAT AFFECTED.
M55319	WCAP-08354,SUPP.3-P "LONG TERM ICE CONDENSER CONTAINMENTCODE FOLLOWING STEAMLINERUPTURE WITH SUPER HEATED STEAM"
M61013	WCAP-10972-P - ABB ATOM-WESTINGHOUSE CORRELATION FOR PREDICT ING CPR MARGINS IN BOILING WATER REACTORS.
M64956	"(WCAP-10858-P-A) ADDENDUM 1 "AMSAC GENERIC DESIGN PACKAGE"
M66167	WCAP-11500-P "WESTINGHOUSE REFERENCE REPORT FOR BWR FUEL"
M77257	WCAP 12488, WESTINGHOUSE FUEL CRITERIA EVALUATION PROCESS
M79497	WCAP-12762 SECONDARY SIDE SAFETY INJECTION FUNCTION DELETION MODIFICATION
M81078	WCAP-12909-P "WESTINGHOUSE ECCS EVALUATION MODEL: REVISED LARGE BREAK LOCA POWER DISTRIBUTION METHODOLOGY
M81712	WCAP-12945-P, VOL.1, WESTINGHOUSE CODE QUALIFICATION DOCUMEN T FOR BEST ESTIMATE LOSS OF COOLANT ACCIDENT ANALYSIS
M82099	WCAP-10924-P-REV. 2-VOLUME 2-ADDENDUM 3 - LARGE BREAK LOCA BEST ESTIMATE METHODOLOGY - UPPER PLENUM INJECTION MODEL IMP
M82772	WCAP - 8183 REV. 19 - OPERATIONAL EXPER W WESTINGHOUSE CORES 12/31/90 N/P
M83235	WCAP-13360 WESTINGHOUSE DYNAMIC ROD WORTH MEASUREMENT TECHNIQUE, MAY 1992 P

WESTINGHOUSE

TaskID -----	Title -----
M83321	WCAP-12476, EVALUATION OF LOCA DURING MODE 3 AND MODE 4 OPERATION FOR WESTINGHOUSE NSSS.
M83964	WCAP-12945-P VOL. 1, REV. 1-WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOSS OF COOLANT ACCIDENT.
M84324	WCAP 13246 P/WCAP 13412 NP W-GOTHIC - A COMPUTER CODE FOR ANALYSES OF THERMAL HYDRAULIC TRANSIENTS FOR NUCLEAR PLANT.
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M86764	WCAP-13749 (P), SAFETY EVALUATION SUPPORT THE CONDITION EXEMP OF THE MOST NEGATIVE EOL MOD TEMP COEFFICIENT MEASURE
M87018	WCAP-12945-P, VOLS. 4&5, WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOSS OF COOLANT ACCIDENT ANALYSIS
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WESTINGHOUSE STEAM TURBINE-GENERATOR

TaskID

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M65737

WSTG-4-P "ANALYSIS OF THE PROBABILITY OF THE GENERATION OF
MISSILES FROM FULL INTEGRAL NUCLEAR LOW PRES.

LIST OF VENDORS

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General Electric Company

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This report provides industry with procedures for submitting topical reports, guidance on how the U.S. Nuclear Regulatory Commission (NRC) will process and respond to topical report submissions, and an accounting of all topical reports currently being reviewed by the NRC staff. This report will be published semiannually.

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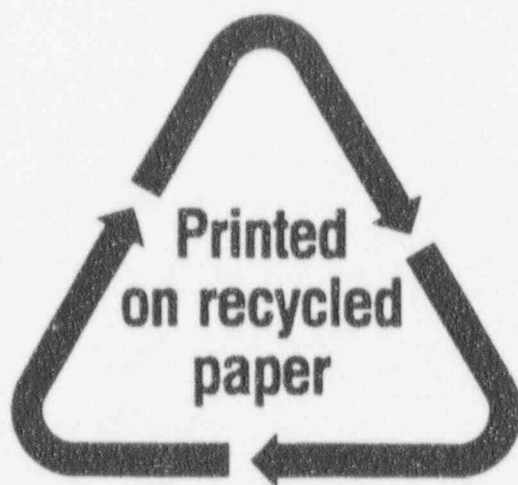
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