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Tech Specs  
50-320*

A6.0 ADMINISTRATIVE CONTROLS

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mtg 7/11/79*

A6.1 RESPONSIBILITY

A6.1.1 The Unit Superintendent shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

A6.2 ORGANIZATION

OFFSITE

A6.2.1 The offsite organization of the Corporate Technical Support Staff for unit management and technical support shall be as shown on Figure A6.2-1.

UNIT STAFF

A6.2.2 The unit organization shall be as shown on Figure A6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table A6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- d. A Site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Site Fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

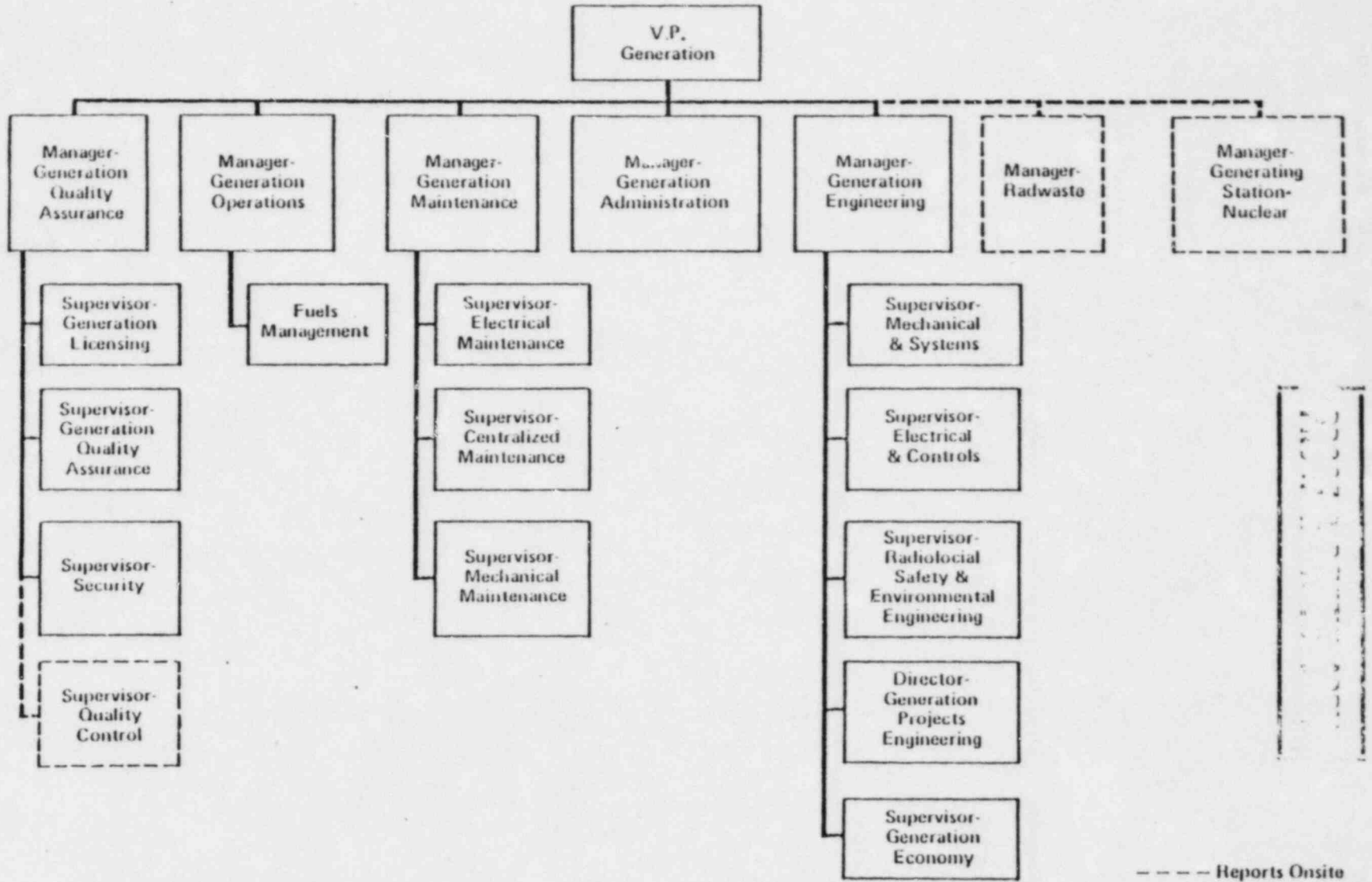
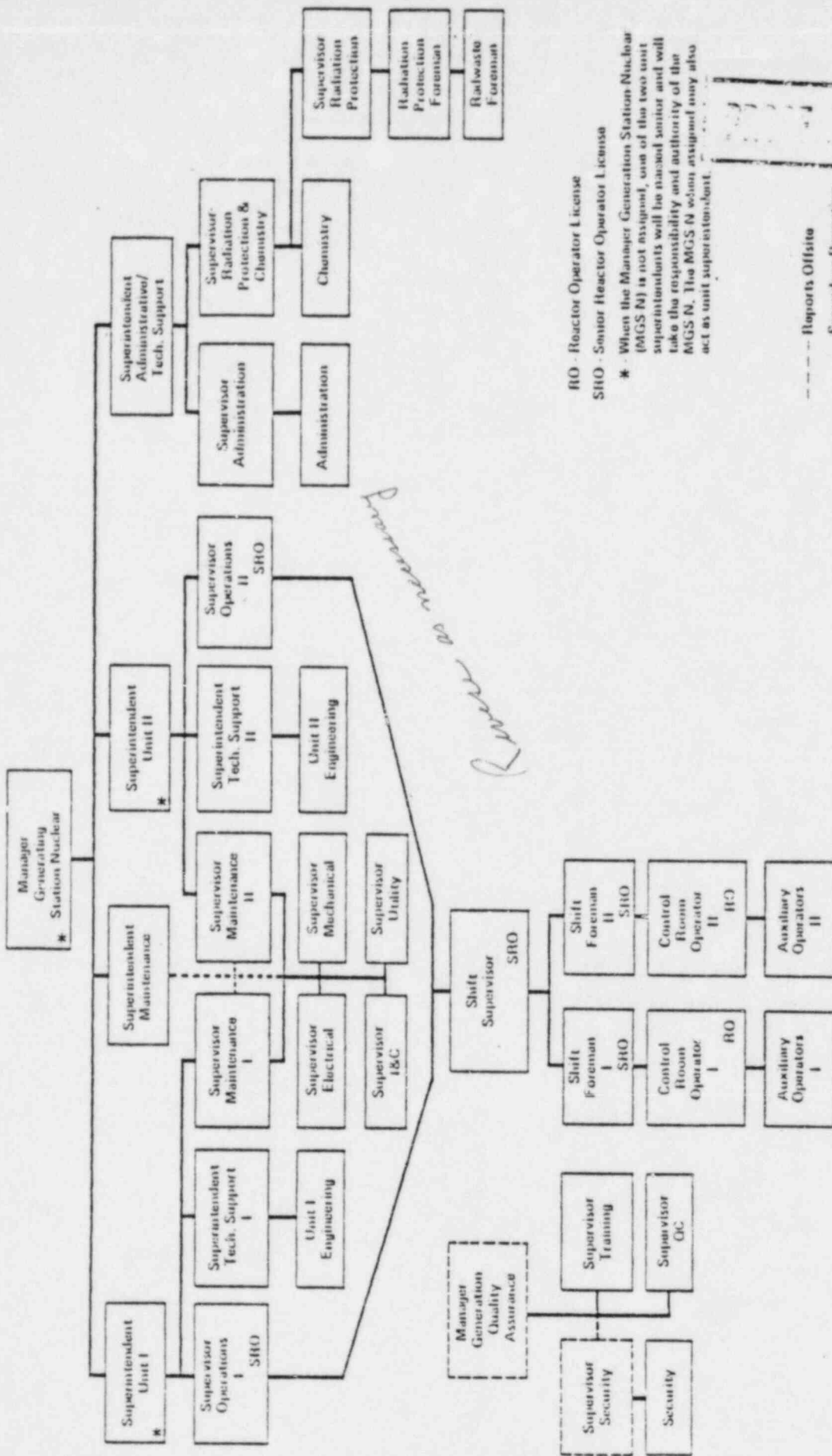


Figure A6.2-1 Offsite Organization for Facility Management and Technical Support.



RO - Reactor Operator License

SRO - Senior Reactor Operator License

\* - When the Manager Generating Station Nuclear (MGS N) is not assigned, one of the two unit superintendents will be named senior and will take the responsibility and authority of the MGS N. The MGS N when assigned may also act as unit superintendent.

--- Reports Offsite

..... Secondary Reporting

Figure A6.2-2 Facility Organization - Three Mile Island - Unit 2

TABLE A6.2-1

MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	RECOVERY MODE
SOL	1
OL	1
Non-Licensed	2

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.





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ALTERNATES

A6.5.1.3 All alternate members shall be appointed in writing by the PORC Chairman or Vice Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities at any one time.

MEETING FREQUENCY

A6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the Chairman or his designated alternate.

QUORUM

A6.5.1.5 A quorum of the PORC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

- A6.5.1.6 The Plant Operations Review Committee shall be responsible for:
- a. Review of 1) all procedures required by Specification A6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Unit Superintendent to affect nuclear safety.
  - b. Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to Appendix "A" Technical Specifications except those concerned with radwaste activities.
  - d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
  - e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager Generating Station-Nuclear and the Generation Review Committee (GRC) Chairman.
  - f. Review of events requiring 24 hour notification to the Commission.
  - g. Review of unit operations to detect potential nuclear safety hazards.

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A6.5.1.6 RESPONSIBILITY (Continued)

- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Unit Superintendent and/or the Manager Generating Station-Nuclear.
- i. Review of the Security Plan and implementing procedures and shall submit recommended changes to the Unit Superintendent and the Manager Generating Station-Nuclear.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Unit Superintendent and the Manager Generating Station-Nuclear.
- k. Review of the Recovery Operations Plan and implementing procedures and shall submit recommended changes to the Unit Superintendent and the Manager Generating Station - Nuclear.

AUTHORITY

A6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend to the Unit Superintendent in writing, approval or disapproval of items considered under A6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under A6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Manager Generating Station-Nuclear and the GRC Chairman of disagreement between the PORC and the Unit Superintendent; however, the Unit Superintendent shall have responsibility for resolution of such disagreements pursuant to A6.1.1 above.

RECORDS

A6.5.1.8 The Plant Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Manager Generating Station-Nuclear and the GRC Chairman.

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A6.5.2 GENERATION REVIEW COMMITTEE (GRC)

FUNCTION

A6.5.2.1 The Generation Review Committee (GRC) shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear unit operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices

COMPOSITION

A6.5.2.2 The GRC shall be composed of at least five members including the Chairman, appointed in writing by the Vice President-Generation. GRC members and alternates may be appointed from within the corporate organization or from external sources.

QUALIFICATIONS

A6.5.2.3 Each GRC member and alternate shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years of technical experience, of which a minimum of three years shall be in one or more of the areas specified in A6.5.2.1.

ALTERNATES

A6.5.2.4 All alternate members shall be appointed in writing by the GRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in GRC activities at any one time.

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ADMINISTRATIVE CONTROLS

CONSULTANTS

A6.5.2.5 Consultants shall be utilized as determined by the GRC Chairman to provide expert advice to the GRC.

MEETING FREQUENCY

A6.5.2.6 The GRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

A6.5.2.7 A quorum of GRC shall consist of the Chairman or his designated alternate and at least 4 GRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

A6.5.2.8.1 The GRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- c. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- d. Events requiring 24 hour written notification to the Commission.
- e. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- f. Reports and meetings minutes of the PORC.

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A6.5.2.8.2 At least one cognizant member of the GRC shall review:

- a. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- b. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed changes to Technical Specifications or this Operating License.

AUDITS

A6.5.2.9 Audits of unit activities shall be performed under the cognizance of the GRC. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The unit performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of all activities required by the Operating Quality Assurance Plan to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 24 months.
- f. The Security Plan and implementing procedures at least once per 24 months.
- g. The Radiation Protection Plan and implementing procedures at least once per 6 months.
- h. The Fire Protection Program Plan and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit performed at least once per 12 months, utilizing either qualified offsite licensee personnel or an outside fire protection firm.



PROTECTIVE SERVICES

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AUDITS (Continued)

- j. An inspection and audit of the fire protection and loss prevention program performed by an outside qualified fire consultant at least once per 36 months.
- k. Any other area of unit operation considered appropriate by the GRC or the Manager-Generation Quality Assurance.

AUTHORITY

A6.5.2.10 The GRC shall report to and advise the Vice President-Generation on those areas of responsibility specified in Sections A6.5.2.8 and A6.5.2.9.

RECORDS

A6.5.2.11 Records of GRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each GRC meeting shall be prepared, approved and forwarded to the Vice President-Generation and Manager-Generating Station-Nuclear within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Vice President-Generation and Manager-Generating Station-Nuclear within 14 days following approval of the review.



ADMINISTRATIVE CONTROLS

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A6.6 REPORTABLE OCCURRENCE ACTION

A6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification A6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PORC and a report submitted to the GRC Chairman and the Manager Generating Station-Nuclear.
- c. Each monthly and quarterly report required pursuant to Specification A6.9.1.10 shall be reviewed by the PORC and a copy shall be submitted to the GRC Chairman, and the Manager Generating Station-Nuclear.

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### A 6.7 SAFETY LIMIT VIOLATION

A 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The Safety Limit violation shall be reported to the Commission, the Manager Generating Station - Nuclear and to the GRC Chairman within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Operations Review Committee. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the GRC Chairman and the Manager Generating Station - Nuclear within 14 days of the violation.

### A 6.8 PROCEDURES

A 6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Recovery Operations Plan implementation.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.

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A6.8 PROCEDURES (Continued)

- g. RECOVERY MODE implementation. (Specifically RECOVERY MODE procedures which involve a reduction in the margin of safety, including those which:)
1. Directly relate to core cooling.
  2. Could increase the likelihood or magnitude of radiological releases.
  3. Could increase the likelihood of failures in systems important to safety.

A6.8.2 Each procedure of A6.8.1 above, and changes thereto, shall be reviewed by the Plant Operations Review Committee and approved by the Unit Superintendent prior to implementation and reviewed periodically as set forth in administrative procedures. Each procedure of A6.8.1 b., f. and g. above, and changes thereto, shall be submitted to the NRC prior to implementation; these procedures, and changes thereto, shall be subject to approval by the NRC prior to implementation. The Unit Superintendent shall have responsibility for determining which procedures are in the A6.8.1 b., f. and g. category.

A6.8.3.1 Temporary changes to procedures of A6.8.1 a., c., d. and e. above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the unit management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the Plant Operations Review Committee and approved by the Unit Superintendent within 14 days of implementation.

A6.8.3.2 Temporary changes to procedures of A6.8.1 b., f. and g. may be made provided the provisions of A6.8.3.1 are satisfied and the change is submitted to the NRC for review within 72 hours.

## ADMINISTRATIVE CONTROLS

### A 6.9 REPORTING REQUIREMENTS

#### ROUTINE REPORTS AND REPORTABLE OCCURRENCES

A 6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

#### ANNUAL REPORTS<sup>1/</sup>

A 6.9.1.4 Annual reports covering the activities of the unit as described below during the previous calendar year shall be submitted prior to March 1 of each year.

A 6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,<sup>2/</sup> e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The following information on aircraft movements at the Harrisburg International Airport:
  1. The total number of aircraft movements (takeoffs and landings) at the Harrisburg International Airport for the previous twelve-month period.
  2. The total number of movements of aircraft larger than 200,000 pounds, based on a current percentage estimate provided by the airport manager.

<sup>1/</sup> A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

<sup>2/</sup> This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

## ADMINISTRATIVE CONTROLS

### REPORTABLE OCCURRENCES

A6.9.1.7 The REPORTABLE OCCURRENCES of Specifications A6.9.1.8 and A6.9.1.9, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

### PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

A6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within 30 days. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Deleted.
- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
- c. Abnormal degradation discovered in reactor coolant pressure boundary, or primary containment.
- d. An unplanned reactivity insertion of more than 0.5% delta K/K or occurrence of any unplanned criticality.
- e. Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- f. Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.
- g. Conditions arising from natural or man-made events that, as a direct result of the event require operation of safety systems, or other protective measures required by technical specifications.
- i. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during unit life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.



## ADMINISTRATIVE CONTROLS

### THIRTY DAY WRITTEN REPORTS

A6.9.1.9 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation.
- c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in engineered safety feature systems.
- d. Abnormal degradation of systems other than those specified in A6.9.1.8.c above designed to contain radioactive material resulting from the fission process.

### REPORTING REQUIREMENTS FOR INCIDENT WHICH OCCURRED ON MARCH 28, 1979

A6.9.1.10 For the first three months following the incident which occurred on March 28, 1979, a written update/status report of the incident and related events which occurred through June 30, 1979, shall be submitted to the Director of the Regional Office no later than the 15th of each month following the calendar month covered by the report (i.e., May 15, June 15, and July 15, 1979). These monthly reports shall include available information concerning the cause of the incident, probable consequences of the incident, planned (short term and preliminary long term) corrective action and a description of continuing activities related to the incident. Following these first three monthly reports, quarterly reports shall be submitted. The first of these quarterly reports shall be submitted no later than October 15, 1979, and it shall include information compiled during the months of July, August and September 1979. Subsequent quarterly reports shall be submitted no later than the 15th of the month following the quarterly report period. These quarterly reports shall continue until a final report is issued and these reports shall include the similar type of information described above to be included in the monthly reports as well as those interim analysis results and interim evaluation concerning the incident which have become available. The final report shall include a summary of Technical Specification violations which occurred during and after the transient. The final report shall also include a summary

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ADMINISTRATIVE CONTROLS

of the cause(s) of the incident, a sequence of events which occurred during the transient, and corrective actions (both short term and long term) which were taken as a result of the March 28, 1979 incident. These monthly and quarterly reports shall be in lieu of the reports required by A6.9.1.8 and A6.9.1.9 and A6.9.2 which would have been required for the interval of March 28, 1979 through June 30, 1979.

SPECIAL REPORTS

A6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report.

A6.10 RECORD RETENTION

A6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of unit operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to the procedures required by Specification A6.8.1.

f. Reports required by A6.9.1.10.

A6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Safety Analysis Report.



## ADMINISTRATIVE CONTROLS

- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
  
- c. Records of transient or operational cycles for those unit components designed for a limited number of transients or cycle.
- d. Records of reactor tests and experiments.
- e. Records of training and qualification for current members of the unit staff.
- f. Records of in-service inspections performed pursuant to these Technical Specifications.
- g. Records of Quality Assurance activities required by the Operating Quality Assurance Plan.
- h. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- i. Records of meetings of the PORC, and the GRC.
- j. Records of the incident which occurred on March 28, 1979.

### A6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and the NRC approved Radiation Protection Plan; these procedures shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### A6.12 HIGH RADIATION AREA

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area shall be controlled as specified in the Radiation Protection Plan.

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RECOVERY OPERATIONS PLAN

TMI-2

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## 4 SURVEILLANCE REQUIREMENTS

### 4.0 APPLICABILITY

#### SURVEILLANCE REQUIREMENTS

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4.0.1 Surveillance Requirements shall be applicable during the RECOVERY MODE or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement. The Surveillance Requirements shall be performed to demonstrate compliance with the OPERABILITY requirements of the Limiting Conditions for Operations and are contained in the RECOVERY OPERATIONS PLAN.

4.0.2 Each Surveillance Requirement shall be performed within the specified time interval with:

- a. A maximum allowable extension not to exceed 25% of the surveillance interval, and
- b. A total maximum combined interval time for any 3 consecutive tests not to exceed 3.25 times the specified surveillance interval.

4.0.3 Performance of a Surveillance Requirement within the specified time interval shall constitute compliance with OPERABILITY requirements for a Limiting Condition for Operation and associated ACTION statements unless otherwise required by the specification.

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SURVEILLANCE REQUIREMENTS

4.1 WATER INJECTION COOLING AND REACTIVITY CONTROL SYSTEMS

4.1.1 BORATION CONTROL

BORON INJECTION

4.1.1.1 Two systems capable of injecting borated cooling water into the Reactor Coolant System shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that the accessible (per occupational exposure considerations) heat traced portion of the flow path from the concentrated boric acid storage system is energized.
- b. At least once per 31 days by verifying that each accessible (per occupational exposure considerations) valve (manual, power operated or automatic) in each flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. At least once per 31 days by verifying, that on recirculation flow, Tech. Spec. makeup pump develops a discharge pressure of greater than or equal to 1125 psig and that each pump operates for at least 15 minutes (per occupational exposure considerations).
- d. At least once per 31 days by verifying, that on recirculation flow, Tech. Spec. decay heat removal pump develops a discharge pressure of greater than or equal to 151 psig and that each pump operates for at least 15 minutes (per occupational exposure considerations).
- e. At least once per 31 days by:
  1. Starting (unless already operating) the boric acid pump from the control room.
  2. Verifying the boric acid pump develops a discharge pressure of greater than or equal to 65 psig (per occupational exposure considerations).
  3. Verifying boric acid pump operation for at least 15 minutes.
  4. Verifying that the boric acid pump is aligned to receive electrical power from an OPERABLE bus.

## SURVEILLANCE REQUIREMENTS

### BORON CONCENTRATION (Concentration)

- f. At least once per 7 days by:
1. Verifying the boron concentration in the concentrated boric acid storage system is between 7875 and 13,125 ppm.
  2. Verifying the boron concentration in the BWST is between 3000 and 4500 ppm.
  3. Verifying the contained borated water volume of the concentrated boric acid storage system is in accordance with Figure 3.1-1.
  4. Verifying the contained borated water volume of the BWST is at least 445,620-gallons (53'6").
  5. Verifying the concentrated boric acid storage system solution temperature is at least 105°F.
- g. At least once per 24 hours by verifying the BWST temperature is at least 50°F when the outside air temperature is less than 50°F.
- h. At least once per 12 hours by verifying that the standby reactor coolant system pressure control system:
1. Surge tank water volume is between 400 to 800 gallons and the tank is pressurized to the operating RCS pressure but not higher than 600 psig.
  2. Isolation valves on the discharge side of the water filled tank nearest the reactor coolant system are open/single valve isolated.
  3. Nitrogen supply bottles are pressurized to between 1800 and 2200 psig.
- i. At least once per 7 days by verifying that the standby reactor coolant system pressure control system water filled tanks, the surge tank, and the degassed water supply tank contain borated water with:
1. A boron concentration of between 3000 and 4500 ppm.
- j. At least once per 31 days by verifying that the standby reactor coolant system pressure control system isolation valve on the discharge side of the water filled tank nearest the reactor coolant system closes automatically on a tank low level test signal.

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SURVEILLANCE REQUIREMENTS

BORON CONCENTRATION

4.1.1.2 The reactor coolant shall be determined to be within its limits by:

- a. By determining the boron concentration of the primary coolant to be between 3000 and 4500 ppm by:
  1. A mass balance calculation at least once per 24 hours.
  2. A chemical analysis at least once per 7 days.
- b. By verifying the primary coolant temperature to be greater than 50°F at least once per 12 hours.

4.1.3 CONTROL ASSEMBLIES

ROD POSITIONS

4.1.3.1 The control rod drive breakers shall be verified open at least once per 7 days.

## SURVEILLANCE REQUIREMENTS

### 4.3 INSTRUMENTATION

#### 4.3.1 NEUTRON MONITORING INSTRUMENTATION

4.3.1.1 Each neutron monitoring instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations during RECOVERY MODE and at the frequency shown in Table 4.3-1.

#### 4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

4.3.2.1 Each ESFAS instrumentation channel shall be demonstrated OPERABLE (per occupational exposure considerations) by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations during RECOVERY MODE and at the frequencies shown in Table 4.3-2.

#### 4.3.3 MONITORING INSTRUMENTATION

##### RADIATION MONITORING INSTRUMENTATION

4.3.3.1 Each fuel storage pool area radiation monitoring instrumentation channel shall be demonstrated OPERABLE (per occupational exposure considerations) by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3-3 and by verifying alarm/trip setpoints of less than or equal to  $3.6 \times 10^5$  cpm and  $9.41 \times 10^6$  cpm for the gaseous activity monitor and particulate activity monitor, respectively.

##### SEISMIC INSTRUMENTATION

4.3.3.3.1 Each of the above accessible (per occupational exposure considerations) seismic monitoring instruments shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3-4.

4.3.3.3.2 Each of the above accessible (per occupational exposure considerations) seismic monitoring instruments actuated during a seismic event shall be restored to OPERABLE status within 24 hours and a CHANNEL CALIBRATION performed within 5 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. A Special Report shall be prepared and submitted to the Commission pursuant to Specification 5.9.2 within 10 days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.



THREE MILE ISLAND - UNIT 2

TABLE 4.3-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. Intermediate Range, Neutron Flux and Rate	S	R(6)	M
2. Source Range, Neutron Flux and Rate	S	R(6)	M

NOTATION

(6) - Neutron detectors and all channel components located inside containment may be excluded from CHANNEL CALIBRATION.

TABLE 4.3-2

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. LOSS OF POWER			
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)			
1. Emergency Bus #2-1E and 2-2E	S	R	M
2. Emergency Bus #2-3E and 2-4E	S	NA	NA

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TABLE 4.3-4

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENTS AND SENSOR LOCATIONS</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. Triaxial Time-History Accelerographs			
a. Reactor Building Ring Girder (454'-8")	M*	R	SA
b. Reactor Building Mat (281'-6")	M*	R	SA
2. Triaxial Peak Accelerographs			
a. Reactor Service Structure	R	NA	NA
b. "B" Core Flood Tank Piping	R	NA	NA
c. 2-1E Switchgear	R	NA	NA
3. Triaxial Seismic Switches			
a. Reactor Building Base	M**	R	NA
4. Triaxial Response-Spectrum Recorders			
a. Reactor Building Mat (281'-6")	M	R	NA

\*Except seismic trigger

\*\*With reactor control room indication

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SURVEILLANCE REQUIREMENTS

METEOROLOGICAL INSTRUMENTATION

4.3.3.4 Each of the meteorological monitoring instrumentation channels shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-5.

REMOTE SHUTDOWN INSTRUMENTATION

4.3.3.5 Each remote shutdown monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-6.

POST-ACCIDENT INSTRUMENTATION

4.3.3.6 Each post-accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-10.

CHLORINE DETECTION SYSTEMS

4.3.3.7 Each chlorine detection system shall be demonstrated OPERABLE by performance of a CHANNEL CHECK at least once per 12 hours, and a CHANNEL FUNCTIONAL TEST at least once per 31 days. At least once per 18 months, the following inspections and maintenance shall be performed:

- a. Check constant head bottle level and refill as necessary,
- b. Clean the sensing cells,
- c. Check flow meter operation and clean or replace filters and air lines as necessary, and
- d. Check air pump for proper operation.
- e. Verify that the detector responds to HCL.

TABLE 4.3-5

METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. WIND SPEED		
a. Nominal Elev. 100 ft.	D	SA
2. WIND DIRECTION		
a. Nominal Elev. 100 ft.	D	SA
3. AIR TEMPERATURE - DELTA T		
a. Nominal Elev. 33 ft.	D	SA
b. Nominal Elev. 150 ft.	D	SA

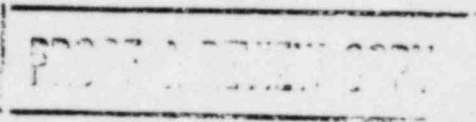


TABLE 4.3-6

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>DISPLAY METERS INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. RC Inlet Temp.	M	R*
2. Make Up Tank Level	M	R*
<u>DISPLAY INDICATIONS</u>		
3. Control Rod Drive Trip Breakers	M	N/A
<u>DISPLAY PATCH POINTS INSTRUMENT</u>		
4. NI Intermediate Range Level Log N	M	R*
5. NI Source Range Level	M	R*
6. RC Flow	M	R*
7. Decay Heat Removal Flow	M	R*
8. Steam Generator Pressure A/B	M	R*

\*Nuclear detectors and all channel components located inside containment and components inaccessible due to occupational exposure considerations may be excluded from CHANNEL CALIBRATION.



TABLE 4.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Building Pressure	S	R*
2. Core Flood Tank Level	S	R*
3. Reactor Coolant Outlet Temperature	S	R*
<hr/>		
4. Borated Water Storage Tank Level	S	R*
5. High Pressure Injection Flow	M**	R*
6. Low Pressure Injection Flow	M**	R*
7. Reactor Building Spray Pump Flow	M**	R*
8. Steam Generator Pressure	S	R*
9. Incore Thermocouples	S	R*
10. Reactor Coolant System Pressure	S	R*
11. Reactor Building Water Level	S	SA***

\*Neutron detectors and all channel components located inside containment and components inaccessible due to occupational exposure considerations may be excluded from CHANNEL CALIBRATION.

\*\*When applicable systems are in operation.

\*\*\*Unless an appropriate engineering evaluation deems the instrument to be in calibration.

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SURVEILLANCE REQUIREMENTS

4.3.3.8.1 Each of the required accessible (per occupational exposure considerations) fire detection instruments shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST.

4.3.3.8.2 The NFPA Code 71 supervised circuits supervision associated with the detector alarms of each of the required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months (per occupational exposure considerations).

4.3.3.8.3 The nonsupervised circuits between the local panels in Surveillance Requirements 4.3.3.8.2 and the control room shall be demonstrated OPERABLE at least once per 31 days.

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SURVEILLANCE REQUIREMENTS

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4.4 REACTOR COOLANT SYSTEM

REACTOR COOLANT LOOPS

4.4.1 Not applicable.

SAFETY VALVES

4.4.3 Not applicable.

4.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

4.4.9.1.1 The Reactor Coolant System pressure and temperature shall be determined to be within the limits at least once per 12 hours.

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SURVEILLANCE REQUIREMENTS

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4.6 CONTAINMENT SYSTEMS

4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

- a. At least once per 31 days by verifying that:
  - 1. All accessible (per occupational exposure considerations) penetrations not required to be open per approved procedures during RECOVERY MODE are closed by valves, blind flanges, or deactivated automatic valves secured in their positions.
  - 2. The equipment hatch is closed and sealed.
- b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

CONTAINMENT AIR LOCKS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a.\* After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying less than or equal to 0.01 L seal leakage when the volume between the door seals is stabilized<sup>a</sup> to a pressure of 10 psig.
- b. At least once per 6 months by conducting an overall air lock leakage test at  $P_0$ , 56.2 psig, and by verifying that the overall air lock leakage rate is within its limit.

INTERNAL PRESSURE

4.6.1.4 The primary containment internal pressure shall be determined to within the limits at least once per 12 hours.

\*Exemption to Appendix "J" of 10 CFR 50.

SURVEILLANCE REQUIREMENTS

AIR TEMPERATURE

4.6.1.5 The primary containment average air temperature shall be the arithmetical average of the temperatures at the following locations and shall be determined at least once per 24 hours:

Location

- a. RB nominal Elev. 350' (2 temperature indications)
- b. RB nominal Elev. 330' (2 temperature indications)
- c. RB nominal Elev. 305' (2 temperature indications)

4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

4.6.4.1 The gas partitioner shall be demonstrated OPERABLE at least once per 31 days by performing a CHANNEL CALIBRATION using sample gases containing:

- a. One volume percent hydrogen, balance air.
- b. Four volume percent hydrogen, balance air.

## SURVEILLANCE REQUIREMENTS

### 4.7 PLANT SYSTEMS

#### 4.7.1 FEEDWATER SYSTEM

4.7.1.1 At least two condensate pumps shall be demonstrated OPERABLE at least once per 31 days by starting (unless already operating) each pump and verifying each pump operates for at least 15 minutes.

4.7.1.2 Each emergency feedwater system shall be demonstrated OPERABLE:

a. At least once per 31 days on a STAGGERED TEST BASIS by:

1. Verifying that each motor driven pump starts and operates on recirc. for at least 15 minutes.
2. Verifying that each accessible (per occupational exposure considerations) valve (manual, power operated or automatic) in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position.

b. At least once per 92 days by cycling each testable valve through at least one complete cycle of full travel.

4.7.1.3 The "B" steam generator closed loop cooling system shall be demonstrated OPERABLE at least once per 31 days by starting (unless already operating) the pump and verifying a flow rate of at least 2000 gpm.

#### 4.7.2 SECONDARY SERVICES CLOSED COOLING WATER SYSTEM

4.7.2.1 At least two secondary services closed cooling water system pumps shall be demonstrated OPERABLE at least once per 31 days by verifying that each pump starts and operates (unless already operating) for at least 15 minutes.

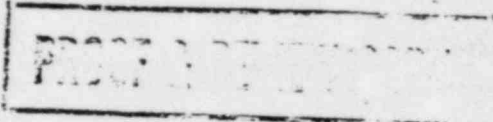
#### 4.7.3 CLOSED CYCLE COOLING WATER SYSTEM

##### NUCLEAR SERVICES CLOSED CYCLE COOLING SYSTEM

4.7.3.1 Tech. Spec. nuclear services closed cycle cooling water pump and valves shall be demonstrated OPERABLE:

a. At least once per 31 days by:

1. Verifying that each pump starts and operates (unless already operating) for at least 15 minutes.



SURVEILLANCE REQUIREMENTS

NUCLEAR SERVICES CLOSED CYCLE COOLING SYSTEM (Continued)

2. Verifying that each accessible (per occupational exposure considerations) valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed or otherwise secured in position, is in its correct position.
- b. At least once per 92 days by cycling each testable valve through at least one complete cycle of full travel.

DECAY HEAT CLOSED COOLING WATER SYSTEM

4.7.3.2 Tech. Spec. decay heat closed cooling water pump and valves should be demonstrated OPERABLE:

- a. At least once per 31 days by:
  1. Verifying that each pump starts and operates (unless already operating) for at least 15 minutes.
  2. Verifying that each accessible (per occupational exposure considerations) valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed or otherwise secured in position, is in its correct position.
- b. At least once per 92 days by cycling each testable valve through at least one complete cycle of full travel.

4.7.4 NUCLEAR SERVICE RIVER WATER SYSTEM

4.7.4.1 Each nuclear service river water loop shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
  1. Verifying that each pump starts and operates (unless already operating) for at least 15 minutes.
  2. Verifying that each accessible (per occupational exposure considerations) valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed or otherwise secured in position, is in its correct position.
- b. At least once per 92 days by cycling each testable valve through at least one complete cycle of full travel.



SURVEILLANCE REQUIREMENTS4.7.6 FLOOD PROTECTION

4.7.6.1.1 The water level at the Unit 1 intake structure shall be determined to be within the limit by:

- a. Measurement at least once per 24 hours when the water level is below elevation 301 feet Mean Sea Level USGS datum.
- b. Measurement at least once per 2 hours when the water level is equal to or above elevation 301 feet Mean Sea Level datum.

4.7.6.1.2 The dike surrounding the island and the dike erosion protection shall be:

- a. Inspected at least once per 6 months and within 31 days after every flood which exceeds 485,000 cfs at the Harrisburg gage. This inspection shall include:
  1. Dike slope and notation of any location where slope is steeper than 2:1.
  2. Depressions and notation of their size and location.
  3. Stone degradation and notation of any areas where splitting, spalling or weathering of stone appears excessive.
  4. Vegetation and notation of location, quantity, and type.
  5. Operability of inside and outside gates of the Southeast Drainage Culvert.

4.7.6.1.3 A Special Report shall be prepared and submitted to the Commission within 10 days if evidence of degradation is noted during an inspection. This report shall describe the extent and nature of the degradation and the plans and schedule for restoring the dike and erosion protection to a status equivalent to the original design provisions.

SURVEILLANCE REQUIREMENTS

4.7.7 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM

4.7.7.1 The control room emergency air cleanup system shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 100°F.
- b. At least once per 31 days by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 15 minutes.
- c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:
  - 1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c\* and C.5.d\* of Regulatory Guide 1.52, Revision 2, March 1978, and the system flow rate is 14,350 cfm  $\pm$  10%.
  - 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.
  - 3. Verifying a system flow rate of 14,350 cfm  $\pm$  10% during system operation when tested in accordance with ANSI N510-1975.
- d. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.

\*The prerequisites of Section 10.3 and 12.3 of ANSI-N510-1975 do not apply.

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SURVEILLANCE REQUIREMENTS

CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM (Continued)

- e. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filter and charcoal adsorber banks is less than 6 inches Water Guage while operating the system at a flow rate of 14,350 cfm  $\pm$  10%.
  2. Verifying that on a control room air inlet radiation test signal or chlorine detection test signal, the system automatically switches into a recirculation mode of operation with flow through the HEPA filters and charcoal adsorber banks.
  3. Verifying that the system maintains the control room at a positive pressure of greater than or equal to 1/10 inch W.G. relative to the outside atmosphere during system operation.
- f. After each complete or partial replacement of a HEPA filter bank by verifying that the HEPA filter banks remove greater than or equal to 99% of the DOP when they are tested in-place in accordance with ANSI N510-1975 while operating the system at a flow rate of 14,350 cfm  $\pm$  10%.
- g. After each complete or partial replacement of a charcoal adsorber bank by verifying that the charcoal adsorbers remove greater than or equal to 99% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1975 while operating the system at a flow rate of 14,350 cfm  $\pm$  10%.

SURVEILLANCE REQUIREMENTS4.7.10 FIRE SUPPRESSION SYSTEMSFIRE SUPPRESSION WATER SYSTEM

4.7.10.1.1 The FIRE SUPPRESSION WATER SYSTEM shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that at least 90,000 gallons of water are in the Altitude Tank, equivalent level in Unit 1 circulating water flume and/or equivalent level in the river.
- b. At least once per 31 days on a STAGGERED TEST BASIS by starting each pump and operating it for at least 20 minutes on recirculation flow.
- c. At least once per 31 days by verifying that each accessible (per occupational exposure considerations) valve (manual, power operated or automatic) in the flow path is in its correct position.
- d. At least once per 12 months by:
  1. Running the full flow of one fire pump through all main header loops as a flush, and
  2. Cycling each accessible (per occupational exposure considerations) testable valve in the flow path through at least one complete cycle of full travel.
- e. At least once per 18 months by performing a system functional test which includes simulated automatic actuation of the system (i.e., pumps start at set pressure  $\pm$  10 psig), and:
  1. Verifying that each pump develops at least 2500 gpm at a total dynamic head no more than 10% below that indicated on manufacturer's curves.
  2. Cycling each accessible (per occupational exposure considerations) valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel, and
  3. Verifying that each high pressure pump starts (sequentially) to maintain the fire suppression water system pressure greater than or equal to 70 psig.
- f. At least once per 3 years by performing a flow test of the system in accordance with Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition, published by the National Fire Protection Association.

SURVEILLANCE REQUIREMENTS

FIRE SUPPRESSION WATER SYSTEM (Continued)

4.7.10.1.2 Each fire pump diesel engine shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying;
  - 1. Its associated fuel storage tank contains at least 250 gallons of fuel, and
  - 2. The diesel starts from ambient conditions and operates for at least 20 minutes.
- b. At least once per 92 days by verifying that a sample of diesel fuel from each fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM D975-74 when checked for viscosity, water and sediment.
- c. At least once per 18 months, by:
  - 1. Subjecting each diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service, and
  - 2. Verifying each diesel starts from ambient conditions on the auto-start signal and operates for greater than or equal to 20 minutes while loaded with the fire pump.

4.7.10.1.3 Each fire pump diesel starting battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  - 1. The electrolyte level of each battery is above the plates, and
  - 2. The overall battery voltage is greater than or equal to 24 volts.
- b. At least once per 92 days by verifying that the specific gravity is appropriate for continued service of the battery.
- c. At least once per 18 months by verifying that:
  - 1. The batteries, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration, and
  - 2. The battery-to-battery and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material.



SURVEILLANCE REQUIREMENTS

DELUGE AND/OR SPRINKLER SYSTEM

4.7.10.2 Each of the required deluge and/or sprinkler systems shall be demonstrated OPERABLE:

- a. At least once per 12 months by cycling each accessible (per occupational exposure considerations) testable valve in the flow path through at least one complete cycle of full travel.
- b. At least once per 18 months:
  - 1. By performing a system functional test (per occupational exposure considerations) which includes tripping detectors and verifying actuation of trip devices on associated deluge valves. Deluge/sprinkler valves shall be inspected internally to verify operability in all instances where header flooding during the test is not practical.
  - 2. Cycling each accessible (per occupational exposure considerations) valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel.
  - 3. By visual inspection of the accessible (per occupational exposure considerations) deluge headers to verify their integrity.
  - 4. By visual inspection of each accessible (per occupational exposure considerations) nozzle to verify no blockage.
- c. At least one per 3 years by performing an air or gas flow test through each deluge header and verifying each accessible (per occupational exposure considerations) deluge nozzle is unobstructed.

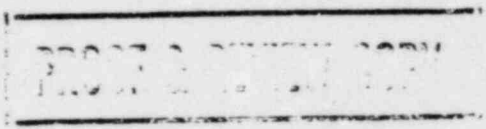
HALON SYSTEM

4.7.10.3.1 The Cable Room and Transformer Room Halon system shall be demonstrated OPERABLE at least once per 6 months by verifying each Halon storage tank weight and pressure.

4.7.10.3.2 The Air Intake Tunnel Halon System shall be demonstrated OPERABLE:

- a. At least once per 6 months by verifying pressure in each halon storage tank, and
- b. At least once per 18 months by verifying storage tank weight.





SURVEILLANCE REQUIREMENTS

FIRE HOSE STATIONS

4.7.10.4 Each accessible (per occupational exposure considerations) fire hose station listed in Table 3.7-4 shall be verified OPERABLE:

- a. At least once per 31 days by visual inspection of the station to assure all required equipment is at the station.
- b. At least once per 18 months by:
  - 1. Removing the hose for inspection and re-racking, and
  - 2. Replacement of all degraded gaskets in couplings.
- c. At least once per 3 years by:
  - 1. Partially opening each hose station valve to verify valve OPERABILITY and no flow blockage.
  - 2. Conducting a hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station.

PENETRATION FIRE BARRIERS

4.7.11 The required (accessible per occupational exposure considerations) penetration fire barriers shall be verified to be functional:

- a. At least once per 18 months by a visual inspection.
- b. Prior to returning a penetration fire barrier to functional status following repairs or maintenance by performance of a visual inspection of the affected penetration fire barrier(s).

## SURVEILLANCE REQUIREMENTS

### 4.8 ELECTRICAL POWER SYSTEMS

#### 4.8.1 A.C. SOURCES

4.8.1.1.1 Each of the required independent circuits between the offsite transmission network and the onsite Class 1E distribution system shall be determined OPERABLE at least once per 7 days by verifying correct breaker alignments and indicated power availability.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
  1. Verifying the fuel level in the day fuel tank.
  2. Verifying the fuel level in the fuel storage tank.
  3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the day tank.
  4. Verifying the Class 1E diesels start from ambient condition and accelerate to at least 900 rpm (ie 60 Hz on the generator) in less than or equal to 10 seconds.
  5. Verifying each Class 1E diesel generator is synchronized; loaded to greater than or equal to 3000 kw, and operated for greater than or equal to 60 minutes.
  6. Verifying each BOP diesel generator is synchronized; loaded to greater than or equal to 2400 kw, and operates for greater than or equal to 60 minutes.
  7. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
- b. At least once per 92 days by verifying that a sample of diesel fuel obtained in accordance with ASTM-D270-63, from the fuel storage tank is within the acceptable limits specified in Table 1 of ASTM-D975-74 when checked for viscosity, water and sediment.
- c. At least once per 92 days by verifying that the BOP diesels start automatically and accelerate to at least 900 rpm (ie 60 Hz on the generator) in less than or equal to 60 seconds and operate for 10 minutes.
- d. Verify breaker (bus 2-5 unit 5-4) on the incoming feed from the 13.2/4.16 KV transformer closes on simulated loss of offsite power.

SURVEILLANCE REQUIREMENTSA.C. SOURCES (Continued)

4.8.1.1.3 Each 125-volt BOP diesel generator battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  1. The electrolyte level of each pilot cell is between the minimum and maximum level indication marks.
  2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level, is greater than or equal to 1.20.
  3. The pilot cell voltage is greater than or equal to 2.13 volts.
  4. The overall battery voltage is greater than or equal to 125 volts.
- b. At least once per 92 days by verifying that:
  1. The voltage of each connected cell is greater than or equal to 2.13 volts under float charge and has not decreased more than 0.10 volts from the value observed during the original acceptance test.
  2. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is greater than or equal to 1.20 and has not decreased more than 0.01 from the value observed during the previous test.
  3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.
- c. At least once per 18 months by verifying that:
  1. The cells, cell plates and battery racks show no visual indication of physical damage or deterioration.
  2. The cell-to-cell and terminal connections are clean, tight and coated with anti-corrosion material.

SURVEILLANCE REQUIREMENTS

A.C. SOURCES (Continued)

4.8.1.1.4 The 13.2 kv circuit from the Middletown Junction Substation shall be demonstrated OPERABLE:

- a. At least once per 24 hours by verifying indicated power availability and that the breakers are open (unless closed in accordance with approved procedures).
- b. At least once per 7 days by closing the 13.2 kv line onto bus 2-5, opening breaker 2B-52, and operating at least one circulating water pump for between 5 and 10 minutes.

4.8.2 ONSITE POWER DISTRIBUTION SYSTEMS

A.C. DISTRIBUTION

4.8.2.1 The specified A.C. busses shall be determined OPERABLE with tie breakers open between redundant busses at least once per 7 days by verifying correct breaker alignment and indicated power availability.

D.C. DISTRIBUTION - OPERATING

4.8.2.3.1 Each D.C. bus train shall be determined OPERABLE and energized with tie breakers open at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.3.2 Each 250/125-volt battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  1. The electrolyte level of each pilot cell is between the minimum and maximum level indication marks.
  2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level, is greater than or equal to 1.20.
  3. The pilot cell voltage is greater than or equal to 2.13 volts.
  4. The overall battery voltage is greater than or equal to 250/125 volts.

SURVEILLANCE REQUIREMENTSD.C. DISTRIBUTION (Continued)

- b. At least once per 92 days by verifying that:
1. The voltage of each connected cell is greater than or equal to 2.13 volts under float charge and has not decreased more than 0.10 volts from the value observed during the original acceptance test.
  2. The specific gravity, correct to 77°F and full electrolyte level, of each connected cell is greater than or equal to 1.20 and has not decreased more than 0.01 from the value observed during the previous test.
  3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.
- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or deterioration.
  2. The cell-to-cell and terminal connections are clean, tight and coated with anti-corrosion material.

SURVEILLANCE REQUIREMENTS

4.9 LIQUID RADIOACTIVE WASTE STORAGE

FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

4.9.12 The fuel handling building/auxiliary building air cleanup systems shall be demonstrated OPERABLE:

- a. At least once per 31 days by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 15 minutes.
- b. Initially and at least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:
  - 1. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.
  - 2. Verifying a fuel handling building system flow rate of 56,000 cfm  $\pm$  10% during system operation when tested in accordance with ANSI N510-1975.
  - 3. Verifying an auxiliary building system flow rate of 83,000 cfm  $\pm$  10% during system operation when tested in accordance with ANSI N510-1975.
- c. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.
- d. At least once per 18 months by:
  - 1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks of the fuel handling building system is less than 6 inches Water Gauge while operating the system at a flow rate of 56,000 cfm  $\pm$  10%.



SURVEILLANCE REQUIREMENTS

2. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks of the auxiliary building system is less than 6 inches Water Gauge while operating the system at a flow rate of 83,000 cfm  $\pm$  10%.
  3. Verifying that the fuel handling building system maintains the fuel handling building at a negative pressure of greater than or equal to 1/8 inch Water Gauge relative to the diesel building during system operation.
  4. Verifying that the auxiliary building system maintains the auxiliary building at a negative pressure of greater than or equal to 1/8 inch Water Gauge relative to the service building during system operation.
- e. After each complete or partial replacement of a HEPA filter bank in the fuel handling building system by verifying that the HEPA filter banks in this system remove greater than or equal to 99% of the DOP when they are tested in-place in accordance with ANSI N510-1975 while operating the system at a flow rate of 56,000 cfm  $\pm$  10%.
  - f. After each complete or partial replacement of a charcoal adsorber bank in the fuel handling building system by verifying that the charcoal adsorbers in this system remove greater than or equal to 99% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1975 while operating the system at a flow rate of 56,000 cfm  $\pm$  10%.
  - g. After each complete or partial replacement of a HEPA filter bank in the auxiliary building system by verifying that the HEPA filter banks in this system remove greater than or equal to 99% of the DOP when they are tested in-place in accordance with ANSI N510-1975 while operating the system at a flow rate of 83,000 cfm  $\pm$  10%.
  - h. After each complete or partial replacement of a charcoal adsorber bank in the auxiliary building system by verifying that the charcoal adsorbers in this system remove greater than or equal to 99% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1975 while operating the system at a flow rate of 83,000 cfm  $\pm$  10%.

PROJECT

SURVEILLANCE REQUIREMENTS

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4.7.6 FLOOD PROTECTION

4.7.6.1.1 The water level at the Unit 1 intake structure shall be determined to be within the limit by:

- a. Measurement at least once per 24 hours when the water level is below elevation 301 feet Mean Sea Level USGS datum.
- b. Measurement at least once per 2 hours when the water level is equal to or above elevation 301 feet Mean Sea Level datum.

4.7.6.1.2 The dike surrounding the island and the dike erosion protection shall be:

- a. Inspected at least once per 6 months and within 31 days after every flood which exceeds 485,000 cfs at the Harrisburg gage. This inspection shall include:
  1. Dike slope and notation of any location where slope is steeper than 2:1.
  2. Depressions and notation of their size and location.
  3. Stone degradation and notation of any areas where splitting, spalling or weathering of stone appears excessive.
  4. Vegetation and notation of location, quantity, and type.
  5. Operability of inside and outside gates of the Southeast Drainage Culvert.

4.7.6.1.3 A Special Report shall be prepared and submitted to the Commission within 10 days if evidence of degradation is noted during an inspection. This report shall describe the extent and nature of the degradation and the plans and schedule for restoring the dike and erosion protection to a status equivalent to the original design provisions.

INTRODUCTION

On March 28, 1979, an accident occurred at the facility which resulted in damage to the reactor core and to certain reactor systems and components. The reactor is being maintained in a stable cooling mode removing decay heat by natural convection circulation of primary coolant through the core and the "A" steam generator. The "A" steam generator is producing steam which is condensed in the condenser and recirculated to the "A" steam generator. An alternate means of removing decay heat from the primary coolant is through the "B" steam generator. The steam side of the "B" steam generator has been modified to provide a water solid, closed loop cooling system which is in turn cooled by the secondary services closed cooling water system. Either steam generator cooling mode is adequate to remove decay heat from the primary coolant.

As a result of the core and equipment damage, various requirements set forth in the Technical Specifications governing operations are no longer appropriate. For example, certain equipment required to be operable is no longer operable as defined in the technical specifications. Other systems not generally relied on for safe shutdown of the reactor are being used to maintain the facility in the stable mode of heat removal. High radioactivity in containment and in the reactor coolant and radioactivity in certain auxiliary areas have limited access to certain components or have limited the ability of certain systems or components in their original design mode. A number of systems and components have been modified in order to respond to the initial emergency condition.

These revised Technical Specifications are to provide assurance that the facility in its present status will provide adequate safety while certain additional work activities are carried out at the facility during the forthcoming period of core cooling.

In addition to continuing to maintain core cooling, certain work is planned for the period during which these revised specifications will be applicable.

- (1) System and components presently functioning to maintain core cooling may be modified to enhance their operability and reliability, and new systems may be added to maintain recovery functions.
- (2) Presently non-functioning systems or components useful to enhance safety are being restored to operable status or modified to function appropriately.
- (3) Activities to maintain control over radioactive effluents from the facility will continue with modifications as necessary to enhance such control.

- (4) Decontamination and cleanup of areas of containment to reduce exposure and to control radioactive materials will continue.

These activities and modifications will be performed during the period in which these revised specifications are effective.

The revised Technical Specifications have been prepared to cover the functional capability or performance levels of systems or components required for the safe operation of the facility. The revised Technical Specifications shall however, in no way modify the obligation of the licensee to take all appropriate steps needed to assure public health and safety in connection with the facility at all times; and shall not be deemed to prohibit or restrict the licensee from taking any emergency action necessary to protect the health and safety of the public in the event of any emergency conditions that may arise during the period governed by these revised Technical Specifications.

Detailed surveillance requirements for implementing these specifications are set forth in the Recovery Operations Plan. The specific plant operating procedures which are subject to review and approval by the NRC staff are identified in Section 6.8 of the Technical Specifications.

These revised Technical Specifications will be effective 30 days after issuance and remain in effect until superceded; and may be supplemented, amended or modified from time to time.

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SECTION 1.0

DEFINITIONS

JUN 15 1979



## 1.0 DEFINITIONS

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### DEFINED TERMS

1.1 The DEFINED TERMS of this section appear in capitalized type and are applicable throughout these Technical Specifications.

### RECOVERY OPERATIONS PLAN

1.2 The RECOVERY OPERATIONS PLAN shall describe unit Operations Requirements for the implementation of these Technical Specifications. This plan, and changes thereto, shall be approved by the Commission prior to implementation.

### SAFETY RELATED

1.3 SAFETY RELATED shall refer to those components and equipment which are used directly for core cooling or prevention of uncontrolled radioactive releases to the environment.

### RECOVERY MODE

1.4 The RECOVERY MODE shall correspond to a condition in which the reactor is subcritical with an average reactor coolant temperature of less than 300°F, and include the time up until the reactor core is completely offloaded.

### ACTION

1.5 ACTION shall be those additional requirements specified as corollary statements to each principle specification and shall be part of the specifications.

### OPERABLE - OPERABILITY

1.6 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment, that are required for the system, subsystem, train, component or device to perform its function(s), are also capable of performing their related support function(s).



## DEFINITIONS

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### REPORTABLE OCCURRENCE

1.7 A REPORTABLE OCCURRENCE shall be any of those conditions specified in Specifications 6.9.1.8 and 6.9.1.9.

### CONTAINMENT INTEGRITY

1.8 CONTAINMENT INTEGRITY shall exist when:

- a. All penetrations are closed by automatic valves, manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except those penetrations required open per approved procedures.
- b. The equipment hatch is closed and sealed.
- c. Each airlock is OPERABLE pursuant to Specification 3.6.1.3.
- d. The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

### CHANNEL CALIBRATION

1.9 A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. CHANNEL CALIBRATION may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

### CHANNEL CHECK

1.10 A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.

DEFINITIONSCHANNEL FUNCTIONAL TEST

1.11 A CHANNEL FUNCTIONAL TEST shall be:

- a. Analog channels - the injection of a simulated signal into the channel as close to the primary sensor as practicable to verify OPERABILITY including alarm and/or trip functions.
- b. Bistable channels - the injection of a simulated signal into the channel sensor to verify OPERABILITY including alarm and/or trip functions.

STAGGERED TEST BASIS

1.12 A STAGGERED TEST BASIS shall consist of:

- a. A test schedule for n systems, subsystems, trains or designated components obtained by dividing the specified test interval into n equal subintervals,
- b. The testing of one system, subsystem, train or designated components at the beginning of each subinterval.

FREQUENCY NOTATION

1.13 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.2.

FIRE SUPPRESSION WATER SYSTEM

1.14 A FIRE SUPPRESSION WATER SYSTEM shall consist of: a water source; gravity tank or pumps; and distribution piping and associated sectionalizing control or isolation valves. Such valves shall include yard hydrant curb valves, and the first valve upstream of the water flow alarm device on each sprinkler, hose standpipe or spray system riser.

TABLE 1.2  
FREQUENCY NOTATION

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<u>NOTATION</u>	<u>FREQUENCY</u>
S	At least once per 12 hours.
D	At least once per 24 hours.
W	At least once per 7 days.
M	At least once per 31 days.
Q	At least once per 92 days.
SA	At least once per 184 days.
A	At least once per 12 months.
R	At least once per 18 months.
N.A.	Not applicable.

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SECTION 2.0  
SAFETY LIMITS  
AND  
LIMITING SAFETY SYSTEM SETTINGS

JUN 13 1979

2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS

REACTOR COOLANT SYSTEM PRESSURE

2.1.3 The Reactor Coolant System pressure shall not exceed 2750 psig.

APPLICABILITY: RECOVERY MODE.

ACTION:

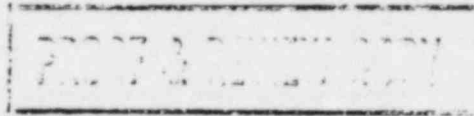
Whenever the Reactor Coolant System pressure has exceeded 2750 psig, reduce the Reactor Coolant System pressure to within its limit.

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BASES  
FOR  
SAFETY LIMITS

JUN 05 1979



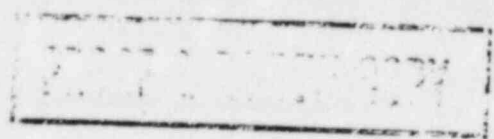


NOTE

The summary statements contained in this section provide the bases for the specifications of Section 2.0 and are not considered a part of these technical specifications as provided in 10 CFR 50.36.

JUN 25 1973

## 2.1 SAFETY LIMITS



### BASES

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#### 2.1.3 REACTOR COOLANT SYSTEM PRESSURE

The restriction of this Safety Limit protects the integrity of the Reactor Coolant System from overpressurization and thereby prevents the release of radionuclides contained in the reactor coolant from reaching the containment atmosphere.

The reactor pressure vessel and pressurizer are designed to Section III of the ASME Boiler and Pressure Vessel Code which permits a maximum transient pressure of 110%, 2750 psig, of design pressure. The Reactor Coolant System piping, valves and fittings, are designed to ANSI B 31.7, 2/68 Edition. Reactor Coolant System valves are designed to ANSI B 16.5-1963, MSSP-61 and MSSP-66. The maximum transient pressure for the Reactor Coolant System valves is permitted by ASME to be 110%, 2750 psig, of design pressure. The Safety Limit of 2750 psig is therefore consistent with the design criteria and associated code requirements.

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SECTIONS 3.0  
LIMITING CONDITIONS FOR OPERATION

JUN 15 1973

### 3 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

#### 3.0 APPLICABILITY

##### LIMITING CONDITIONS FOR OPERATION

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3.0.1 Limiting Conditions for Operation and ACTION requirements shall be applicable during the RECOVERY MODE or other conditions specified for each specification.

3.0.2 Adherence to the requirements of the Limiting Condition for Operation and/or associated ACTION within the specified time interval shall constitute compliance with the specification. In the event the Limiting Condition for Operation is restored prior to expiration of the specified time interval, completion of the ACTION statement is not required.

3.0.3 In the event a Limiting Condition for Operation and/or associated ACTION requirements cannot be satisfied because of circumstances in excess of those addressed in the specification, initiate appropriate actions to rectify the problem to the extent possible under the circumstances, and take all other actions necessary to maintain the unit in a stable condition; promptly notify the NRC Site Staff; and submit a report to the Commission pursuant to Specification 6.9.1.

## LIMITING CONDITIONS FOR OPERATION

### 3.1 WATER INJECTION COOLING AND REACTIVITY CONTROL SYSTEMS

#### 3.1.1 BORATION CONTROL

##### BORON INJECTION

3.1.1.1 At least two systems capable of injecting borated cooling water into the Reactor Coolant System shall be OPERABLE.

a. One system\* shall be comprised of the following:

1. One OPERABLE makeup pump.\*\*
2. One OPERABLE decay heat removal pump.
3. An OPERABLE flow path from the boric acid storage system from the EWST. The boric acid storage system shall contain borated water in accordance with Figure 3.1-1 at a minimum temperature of 105°F. The BWST shall contain at least 45,620 gallons (53' 6") of borated water at a minimum temperature of 50°F and at a boron concentration of between 3000 and 4500 ppm.

b. The second system\* shall be comprised of the following:

1. Pressure Volume Control System

APPLICABILITY: RECOVERY MODE

##### ACTION:

With one of the above required systems inoperable, restore the inoperable system to OPERABLE status within 72 hours.

\* Both systems shall be considered operable with single valve isolation.

\*\* When putting a makeup pump in service, if DH-V-171 or DH-V-1 is open apply administrative controls to maintain RCS pressure below 350 psig.

##### BORON CONCENTRATION

3.1.1.2 The reactor coolant shall be maintained at a boron concentration of between 3000 and 4500 ppm and at a temperature above 50°F.

APPLICABILITY: RECOVERY MODE

##### ACTION:

None except as provided in Specification 3.0.3.

LIMITING CONDITIONS FOR OPERATION

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3.1.3 CONTROL ASSEMBLIES

ROD POSITIONS

3.1.3.1 All control rod drive breakers shall be open.

APPLICABILITY: RECOVERY MODE

ACTION:

None except as provided in Specification 3.0.3.



3.1-1-2 UNIT 2 0037

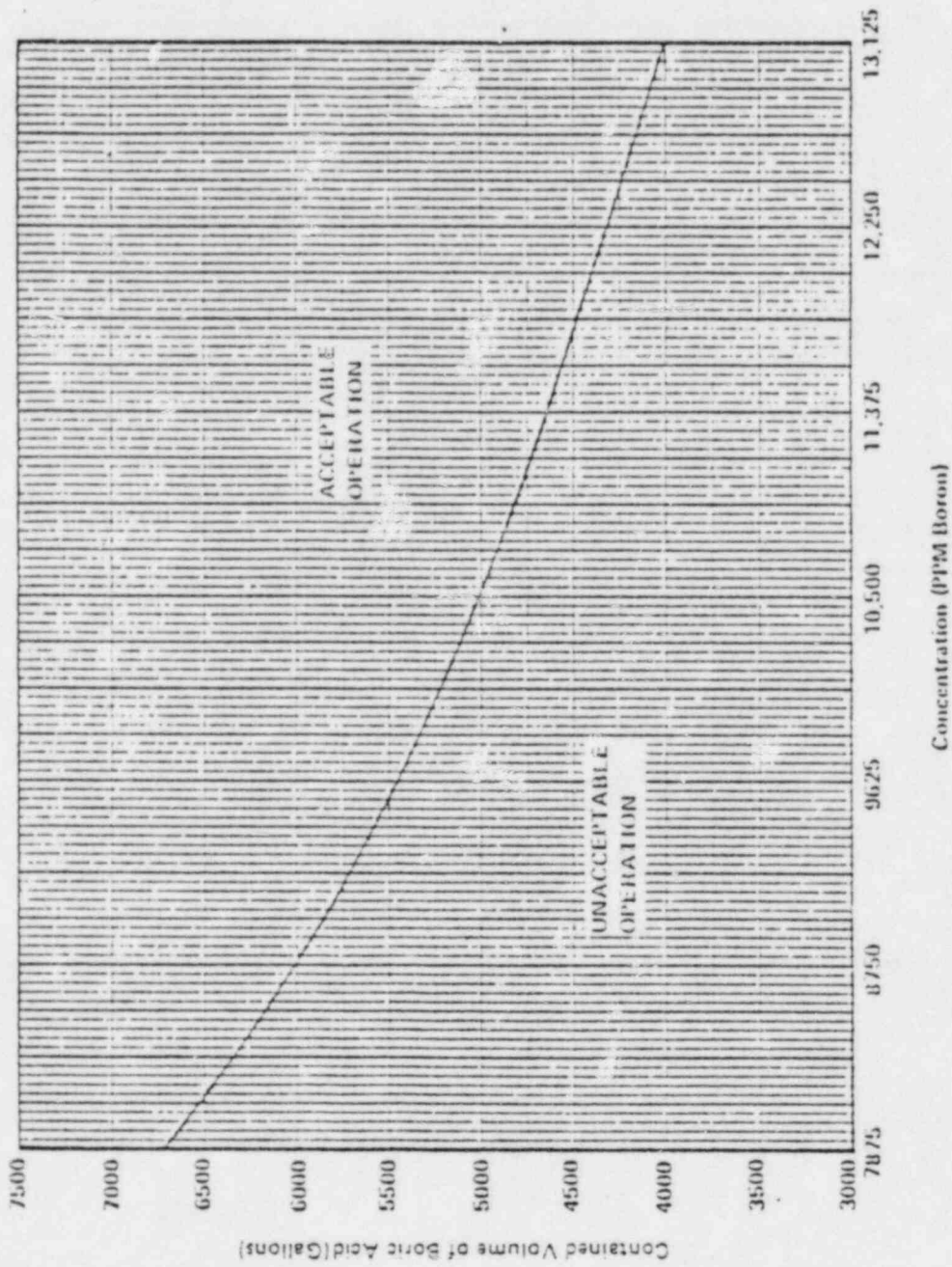
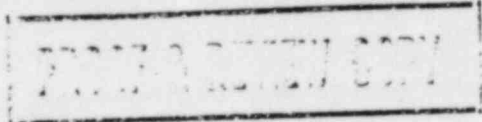


Figure 3.1-1 Minimum Boric Acid Tank Contained Volume as a Function of Stored Boric Acid Concentration



LIMITING CONDITIONS FOR OPERATION

3.3. INSTRUMENTATION

3.3.1 NEUTRON MONITORING INSTRUMENTATION

3.3.1.1 As a minimum, the neutron monitoring instrumentation channels of Table 3.3-1 shall be OPERABLE.

APPLICABILITY: RECOVERY MODE.

ACTION:

With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement of Table 3.3-1, restore inoperable channel to OPERABLE status within 7 days. With no channels OPERABLE, verify compliance with the boron concentration requirements of Specification 3.1.1.2 at least once per 24 hours by mass balance and once every 7 days by chemical analysis.

3.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

3.3.2.1 The Engineered Safety Feature Actuation System (ESFAS) instrumentation channels shown in Table 3.3-3 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3-4.

APPLICABILITY: RECOVERY MODE.

ACTION:

- a. With an ESFAS instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3-4, declare the channel inoperable and apply the applicable ACTION requirement of Table 3.3-3 until the channel is restored to OPERABLE status with the trip setpoint adjusted consistent with the Trip Setpoint Value.
- b. With an ESFAS instrumentation channel inoperable, take the action shown in Table 3.3-3.

TABLE 3.3-1

NEUTRON MONITORING INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Intermediate Range, Neutron Flux and Rate	1	0	1
2. Source Range, Neutron Flux and Rate	1	0	1

TABLE 3.3-3

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
1. LOSS OF POWER				
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)				
1. Emergency Bus #2-1E and 2-2E	2/Bus	2/Bus	2/Bus	10
2. Emergency Bus #2-3E and 2-4E	2/Bus	1/Bus	2/Bus	11

3.0.3-1 - UNIT 2  
 3.0.3-2

ACTION 10 - With the number of OPERABLE channels one less than the Total Number of Channels, place the inoperable channel in the tripped condition.

ACTION 11 - None except as provided in Specification 3.0.3.

TABLE 3.3-4

ENGINEERED SAFETY FEATURE ACTUATION SYSTEMS INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. LOSS OF POWER		
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)		
1. Emergency Bus #2-1E and 2-2E	2278 + 69 volts with a 1.5 + .15 second time delay	2278 + 114 volts with a 1.5 + 0.2 second time delay
2. Emergency Bus #2-3E and 2-4E	2278 + 69 volts with a 1.5 + .15 second time delay	2278 + 114 volts with a 1.5 + 0.2 second time delay

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LIMITING CONDITIONS FOR OPERATION

3.3.3 MONITORING INSTRUMENTATION

RADIATION MONITORING INSTRUMENTATION

3.3.3.1 At least one fuel storage pool area gaseous activity and at least one particulate activity radiation monitor shall be OPERABLE.

APPLICABILITY: With liquid radioactive wastes in the pool storage tanks.

ACTION: With the above required instrumentation inoperable, suspend all operations involving movement of liquid and gaseous radwaste.

SEISMIC INSTRUMENTATION

3.3.3.3 The seismic monitoring instrumentation shown in Table 3.3-7 shall be OPERABLE.

APPLICABILITY: RECOVERY MODE

ACTION:

With one or more seismic monitoring instruments inoperable for more than 30 days, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 10 days outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.

METEOROLOGICAL INSTRUMENTATION

3.3.3.4 The meteorological monitoring instrumentation channels shown in Table 3.3-8 shall be OPERABLE.

APPLICABILITY: RECOVERY MODE

ACTION:

With any of the above required meteorological monitoring channels inoperable, restore the inoperable channel(s) to OPERABLE status within 8 hours.

TABLE 3.3-7

SEISMIC MONITORING INSTRUMENTATION

<u>INSTRUMENTS AND SENSOR LOCATIONS</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM INSTRUMENT OPERABLE</u>
1. Triaxial Time-History Accelographs		
a. Reactor Building Ring Girder (454' - 8")	± 1 G	1
b. Reactor Building Mat (281'-6")	± 1 G	1
2. Triaxial Peak Accelographs		
a. Reactor Service Structure	± 1 G	1
b. "B" Core Flood Tank Piping	± 1 G	1
c. 2-1E Switchgear	± 1 G	1
3. Triaxial Seismic Switches		
a. Reactor Building Base - Setpoint -	0.25 G	1*
4. Triaxial Response-Spectrum Recorders		
a. Reactor Building Mat (281'-6")	(Hz) 2 to 25.4 (Acceleration) 1.6 to 64	1

\* With reactor control room indication



TABLE 3.3-8

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METEOROLOGICAL MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>LOCATION</u>	<u>MINIMUM OPERABLE</u>
1. WIND SPEED		
a. Nominal Elev. 100 ft.		1
2. WIND DIRECTION		
a. Nominal Elev. 100 ft.		1
3. AIR TEMPERATURE - DELTA T		
a. Nominal Elev. 33 ft.		1
b. Nominal Elev. 150 ft.		1

REMOTE SHUTDOWN INSTRUMENTATION

3.3.3.5 The remote shutdown monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE with readouts displayed external to the control room.

APPLICABILITY: RECOVERY MODE.

ACTION:

With the number of OPERABLE remote shutdown monitoring channels less than required by Table 3.3-9, restore the inoperable channel to OPERABLE status within 30 days.

POST-ACCIDENT INSTRUMENTATION

3.3.3.6 The post-accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: RECOVERY MODE.

ACTION:

With the number of OPERABLE post-accident monitoring channels less than required by Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 8 hours.

CHLORINE DETECTION SYSTEMS

3.3.3.7 Two independent chlorine detection systems, with their alarm/trip setpoints adjusted to actuate at a chlorine concentration of less than or equal to 5 ppm, shall be OPERABLE:

- a. One at the air intake tunnel, and
- b. One at the control room air supply duct

APPLICABILITY: RECOVERY MODE.

ACTION:

With no chlorine detection systems OPERABLE within 1 hour initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation; restore the inoperable detection system to OPERABLE status within 30 days.

TABLE 3.3-9

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>DISPLAY METERS INSTRUMENT</u>	<u>READOUT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM OPERABLE CHANNELS</u>
1. RC Inlet Temp	Cab. 217	50-650°F	1/Loop
2. Makeup Tank Level	Cab. 217	0-100 inches	1
<u>DISPLAY INDICATIONS</u>			
3. Control Rod Drive Trip Breakers	Cable Room	open/closed	1/Breaker
<u>DISPLAY PATCH POINTS INSTRUMENT</u>	<u>PATCH POINT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM OPERABLE CHANNELS</u>
4. NI Intermediate Range Level Log N	Cab. 217	$10^{-11}$ to $10^{-3}$ Amps	1
5. NI Source Range Level	Cab. 217	$10^{-1}$ to $10^6$ CPS	1
6. RC Flow	Cab. 217	0 to $180 \times 10^6$ LBS/HR	1/Loop
7. Decay Heat Removal Flow	Cab. 217	0-5000 GPM	1/Loop
8. Steam Generator Pressure A/B	Cab. 217	0-1200 psig	1/St. Gen



TABLE 3.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Building Pressure	2
2. Core Flood Tank Level	1/Tank
3. Reactor Coolant Outlet Temperature	2
4. Borated Water Storage Tank Level	1
5. High Pressure Injection Flow	1/loop
6. Low Pressure Injection Flow	1/loop
7. Reactor Building Spray Pump Flow	1
8. Steam Generator Pressure	1/steam generator
9. Incore Thermocouples	All available*
10. Reactor Coolant System Pressure	1
11. Reactor Building Water Level	1

\* Report all failures pursuant to Specification 6.9.1.8.

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LIMITING CONDITIONS FOR OPERATION

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FIRE DETECTION

3.3.3.8 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-11 shall be OPERABLE.

APPLICABILITY: RECOVERY MODE

ACTION:

With the number of OPERABLE fire detection instruments less than required by Table 3.3-11, insure that an alternate instrument with the same coverage is OPERABLE, or;

1. Within 1 hour, establish a fire watch patrol, except in areas inaccessible due to occupational exposure considerations, to inspect the zone with the inoperable instrument(s) at least once per hour, and
2. Restore the inoperable instrument(s) to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.

TABLE 3.3-11

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FIRE DETECTION INSTRUMENTS

Instrument Location	Required Instruments Operable			Alternate Instrument		
	Heat	Flame	Smoke	Heat	Flame	Smoke
1. Auxiliary Building	N/A	N/A	2	N/A	N/A	2
2. Control Building						
351' Elevation	N/A	N/A	1	N/A	N/A	3
331' Elevation (Control Room)	N/A	N/A	1	N/A	N/A	1
305', 293', 280' Elevations	N/A	N/A	1	N/A	N/A	7
3. Control Building Area	N/A	N/A	1	N/A	N/A	3
4. Diesel Generator Building						
A Diesel	N/A	1	1	N/A	N/A	1
B Diesel	N/A	1	1	N/A	N/A	1
5. Fuel Handling Building	N/A	N/A	3	N/A	N/A	1
6. Reactor Building						
RC-P-1A/2A	N/A	N/A	1	N/A	N/A	1
RC-P-1B/2B	N/A	N/A	1	N/A	N/A	2
7. River Water Pump House	N/A	1	1	N/A	N/A	2
8. Service Building	N/A	N/A	3	N/A	N/A	3
9. BOP Diesel Generators	N/A	1	N/A	N/A	1	N/A

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LIMITING CONDITIONS FOR OPERATION

REVISION BODY

3.4 REACTOR COOLANT SYSTEM

REACTOR COOLANT LOOPS

3.4.1 The Reactor Coolant System shall be operated in accordance with approved procedures. The reactor coolant pumps shall be maintained in an OPERABLE standby status.

APPLICABILITY: RECOVERY MODE.

ACTION:

None except as provided in Specification 3.0.3.

SAFETY VALVES

3.4.3 All pressurizer code safety valves shall be OPERABLE with a lift setting of 2435 PSIG  $\pm$  1%\*.

APPLICABILITY: RECOVERY MODE.

ACTION:

None except as provided in Specification 3.0.3.

3.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

3.4.9.1 The Reactor Coolant System shall be maintained at a  $T_{avg}$  of less than 300°F and at a pressure of less than 1000 psig.

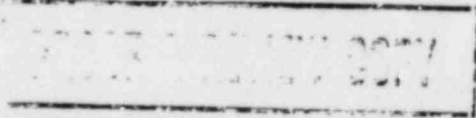
APPLICABILITY: RECOVERY MODE

ACTION:

None except as provided in Specification 3.0.3.

\*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

LIMITING CONDITIONS FOR OPERATION



3.6 CONTAINMENT SYSTEMS

3.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained and all containment penetrations, including at least two containment isolation valves in each penetration, shall be closed when not required open per approved procedures until a plan is approved by the NRC to allow discharge of radioactive materials from the containment to the radioactive waste processing system and containment entry.

APPLICABILITY: RECOVERY MODE.

ACTION:

With one containment isolation valve per containment penetration open or inoperable, maintain the affected penetration(s) closed with:

- a. At least one deactivated automatic valve secured in the isolation position, and
- b. At least one closed manual valve, or a blind flange.

CONTAINMENT AIR LOCKS

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for transit entry and exit through the containment, than at least one air lock door shall be closed.
- b. An overall air lock leakage rate of less than or equal to  $0.05 L_a$  at  $P_a$ , 56.2 psig.

APPLICABILITY: RECOVERY MODE.

ACTION:

With an air lock inoperable, maintain at least one door closed and restore the air lock to OPERABLE status within 24 hours.

## LIMITING CONDITIONS FOR OPERATION

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### INTERNAL PRESSURE

3.6.1.4 Primary containment pressure shall be maintained between -2 and +3 psig.

APPLICABILITY: RECOVERY MODE

ACTION:

With the containment internal pressure more negative than -2 psig or greater than +3 psig, restore the internal pressure to within the limits within 1 hour.

### AIR TEMPERATURE

3.6.1.5 Primary containment average air temperature shall not exceed 130°F.

APPLICABILITY: RECOVERY MODE

ACTION:

With the containment average air temperature greater than 130°F, reduce the average air temperature to within the limit within 24 hours.

### 3.6.4 COMBUSTIBLE GAS CONTROL

#### HYDROGEN ANALYZERS

3.6.4.1 One gas partitioner shall be OPERABLE.

APPLICABILITY: RECOVERY MODE

ACTION:

With the gas partitioner inoperable, restore the inoperable instrument to OPERABLE status within 7 days.

## LIMITING CONDITIONS FOR OPERATION

### 3.7 PLANT SYSTEMS

#### 3.7.1 FEEDWATER SYSTEM

3.7.1 The steam generator condensate and feedwater system shall be operated in accordance with approved procedures. Two motor-driven emergency feedwater pumps (both capable of being powered from OPERABLE emergency busses) and associated flow paths shall be OPERABLE.

APPLICABILITY: RECOVERY MODE.

ACTION:

With one emergency feedwater system inoperable, restore the inoperable system to OPERABLE status within 72 hours.

#### 3.7.2 SECONDARY SERVICES CLOSED COOLING WATER SYSTEM

3.7.2.1 At least two independent secondary services closed cooling water system pumps and heat exchangers and the associated flow path shall be operable with each pump capable of being powered from separate emergency busses.

APPLICABILITY: RECOVERY MODE.

ACTION:

With only one secondary services closed cooling water pump or only one secondary services heat exchanger OPERABLE, restore the inoperable pump or heat exchanger to OPERABLE status within 72 hours.

#### 3.7.3 CLOSED CYCLE COOLING WATER SYSTEM

##### NUCLEAR SERVICES CLOSED CYCLE COOLING SYSTEM

3.7.3.1 At least two independent nuclear services closed cycle cooling water pumps and heat exchangers and the associated flow path shall be OPERABLE with each pump capable of being powered from separate emergency busses.

APPLICABILITY: RECOVERY MODE.

ACTION:

With only one nuclear services closed cycle cooling water pump or only one nuclear services heat exchanger OPERABLE, restore the inoperable pump or heat exchanger to OPERABLE status within 72 hours.

LIMITING CONDITIONS FOR OPERATION

REVISION 0001

DECAY HEAT CLOSED COOLING WATER SYSTEM

3.7.3.2 One independent decay heat closed cooling water loop shall be OPERABLE.

APPLICABILITY: RECOVERY MODE.

ACTION:

With no decay heat closed cooling water loop inoperable, restore the inoperable loop to OPERABLE status within 72 hours.

3.7.4 NUCLEAR SERVICE RIVER WATER SYSTEM

3.7.4.1 Two independent nuclear service river water loops shall be OPERABLE.

APPLICABILITY: RECOVERY MODE.

ACTION:

With one nuclear service river water loop inoperable, restore the inoperable loop to OPERABLE status within 72 hours.

LIMITING CONDITIONS FOR OPERATION

3.7.6 FLOOD PROTECTION

3.7.6.1 Flood protection shall be provided for all safety related systems, components and structures when the water level of the Susquehanna River exceeds 301 feet Mean Sea Level USGS datum, at the river water intake structure of Three Mile Island Nuclear Station, Unit 1.

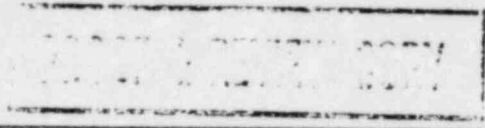
APPLICABILITY: RECOVERY MODE.

ACTION:

- a. With the water level at the Unit 1 Intake Structure approaching 301 ft. Mean Sea Level USGS datum:
  - 1. Initiate patrol and inspection of the dikes surrounding the site for signs of deterioration such as undermining or excessive seepage.
  - 2. Inform the Unit Superintendent and as directed by him:
    - a) Prepare all flood panels and door seals for installation,
    - b) Check all building floor drains and pumps to ensure proper operation,
    - c) Commence daily soundings of the Intake Screen House Floor,
    - d) Check all water tight doors to ensure proper operation,
    - e) Fill all outdoor storage tanks to inhibit flotation, and
    - f) Arrange for alternate supplies of diesel fuel oil and ensure fuel storage tanks are filled.
- b. With the water level at the Unit 1 Intake Structure exceeding 301 ft. and approaching 302 ft. Mean Sea Level USGS datum:
  - 1. Ensure all door seals and flood panels are installed and all water tight doors are closed within 2 hours,
  - 2. Inform the Unit Superintendent.



LIMITING CONDITIONS FOR OPERATION



3.7.3 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM

3.7.7.1 The control room ventilation and emergency air cleanup system shall be OPERABLE with:

- a. Two control room supply fans and associated cooling coils,
- b. Two control room bypass fans,
- c. One charcoal absorber and HEPA filter train, and
- d. Two isolation dampers in the outside air intake duct.

APPLICABILITY: RECOVERY MODE.

ACTION:

- a. With one control room supply fan or its associated cooling coil inoperable, restore the inoperable fan and/or cooling coil to OPERABLE status within 7 days.
- b. With one control room bypass fan inoperable restore the inoperable fan to OPERABLE status within 7 days.
- c. With the filter train inoperable, restore the filter train to OPERABLE status within 24 hours.
- d. With one isolation damper in the outside air intake duct inoperable, restore the inoperable damper to OPERABLE status or close the duct within 4 hours by use of at least one isolation damper secured in the closed position.

LIMITING CONDITIONS FOR OPERATION

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3.7.10 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

3.7.10.1 The FIRE SUPPRESSION WATER SYSTEM shall be OPERABLE with;

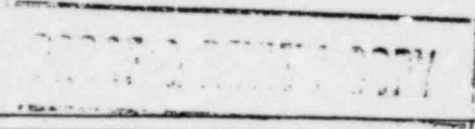
- a. At least 3 of the following 4 high pressure pumps shall be OPERABLE with their discharge aligned to the fire suppression header:
  - 1. Unit 1 Circulating Water Flume Diesel Fire Pump
  - 2. Unit 1 River Water Intake Diesel Fire Pump
  - 3. Unit 2 River Water Intake Diesel Fire Pump
  - 4. Unit 1 River Water Intake Motor Fire Pump
- b. Two (2) separate water supplies of the following four (4) shall be available with at least 90,000 gallons each:
  - 1. Altitude Tank
  - 2. Unit 1 Circulating Water Flume
  - 3. Unit 1 River Water Intake Structure
  - 4. Unit 2 River Water Intake Structure
- c. An OPERABLE flow path capable of taking suction from a water supply and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves and the first valve ahead of the water flow alarm device on each sprinkler, hose standpipe or spray system riser required to be OPERABLE per Specifications 3.7.10.2 and 3.7.10.4.

APPLICABILITY: RECOVERY MODE.

ACTION:

- a. With two pumps or three water supplies inoperable, restore the inoperable equipment to OPERABLE status within 7 days or in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system.

LIMITING CONDITIONS FOR OPERATION



FIRE SUPPRESSION WATER SYSTEM (Continued)

ACTION (Continued)

- b. With the fire suppression water system otherwise inoperable:
  - 1. Establish a backup fire suppression water system within 24 hours, and
  - 2. Submit a Special Report in accordance with Specification 6.9.2;
    - a. By telephone within 24 hours, and
    - b. Confirmed by telegraph, mailgram or facsimile transmission no later than the first working day following the event, and
    - c. In writing within 30 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

LIMITING CONDITIONS FOR OPERATION

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DELUGE/SPRINKLER SYSTEMS

3.7.10.2 The Deluge and/or Sprinkler Systems located in the following areas shall be OPERABLE:

- a. Diesel Generator Fuel Oil Tanks
- b. Diesel Generator Building Air Intake
- c. Air Intake Tunnel (Deluge - 2 of the 3 zones)
- d. Hydrogen Purge Exhaust Filter AH-F-34\*\*
- e. Reactor Building Purge Exhaust Filters AH-F-31A/B\*\*
- f. Control Room Bypass Filter AH-F-5\*\*
- g. Diesel Generators Rooms
- h. Fuel Handling Building Exhaust Filter AH-F-14A/B\*\*
- i. Waste Gas Disposal Filter WDG-F-1\*\*
- j. Auxiliary Building Exhaust Filters AH-F-1QA/B\*\*
- k. Condenser Exhaust Filters\*\*
- l. Auxiliary Building Backup Exhaust Filters\*\*
- m. Balance of Plant Diesel Generators

APPLICABILITY: RECOVERY MODE.

ACTION:

With one or more of the above required deluge and/or sprinkler systems inoperable, establish a roving (once per hour) fire watch\* with backup fire suppression equipment for the unprotected area(s) within 1 hour; restore the system to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

\* Except in the air intake tunnel and areas inaccessible per occupational exposure considerations where no roving fire watch is required.

\*\* These filters are OPERABLE with single valve isolation. In the case of an alarm, the fire shall be verified before the single valve is actuated.

LIMITING CONDITIONS FOR OPERATION

HALON SYSTEM

3.7.10.3 The following Halon systems shall be OPERABLE with the storage tanks having at least 95% of full charge weight and 90% of full charge pressure (corrected to 70°F).

- a. Cable Room and Transformer Room - Control Building 305' elevation.
- b. Air Intake Tunnel (4 Zones)

APPLICABILITY: RECOVERY MODE.

ACTION:

With one or more of the above required Halon systems inoperable, establish a roving (once per hour) fire watch\* with backup fire suppression equipment for the unprotected area(s) within 1 hour; restore the system to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

FIRE HOSE STATIONS

3.7.10.4 The fire hose stations listed in Table 3.7-4 shall be OPERABLE:

APPLICABILITY: RECOVERY MODE.

ACTION:

With one or more of the fire hose stations shown in Table 3.7-4 inoperable, route an additional equivalent capacity fire hose to the unprotected area(s) except areas inaccessible due to occupational exposure considerations, from an OPERABLE hose station within 1 hour.

FIRE HOSE STATIONSAuxiliary Building

1. Fire hose near stairway at South end of building near Decay Heat Closed Cooling Surge Tank, 328' elevation.
2. Fire hose near entrance to Chemical Addition Area, 328' elevation.
3. Fire hose near Miscellaneous Waste Holdup Tank room, 305' elevation.
4. Fire hose near Intermediate Closed Coolers, 305' elevation.
5. Fire hose near Evaporator Condensate Test Tank Room, 280' elevation.
6. Fire hose near Reactor Building Emergency Cooling Booster Pumps, 280' elevation.

Control Building Area

1. Fire hose near 480 Volt Bus 2-34 282'6" elevation.
2. Fire hose near entrance to Service Building 282'6" elevation.

Control Building

1. Fire hose near doorway on North wall 305' elevation.

Fuel Handling Building

1. Fire Hose on east wall 347'6" elevation.



LIMITING CONDITIONS FOR OPERATION

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PENETRATION FIRE BARRIERS

3.7.11 All penetration fire barriers protecting safety related areas shall be functional.

APPLICABILITY: RECOVERY MODE.

ACTION:

With one or more of the above required penetration fire barriers non-functional, establish a roving (once per hour) fire watch on at least one side of the affected penetration within 1 hour, except areas inaccessible due to occupational exposure considerations.



LIMITING CONDITIONS FOR OPERATION

3.8.2 CNSITE POWER DISTRIBUTION SYSTEMS

A.C. DISTRIBUTION

3.8.2.1 The following A.C. electrical busses shall be OPERABLE and energized with tie breakers open (unless closed in accordance with approved procedures) between redundant busses:

- 4160            volt Emergency Bus # 2-1E and 2-3E
- 4160            Volt Emergency Bus # 2-2E and 2-4E
- 4160            Volt Busses # 2-3, 2-4, and 2-5
- 480             volt Emergency Bus # 2-11E, 2-12E and 2-31E
- 480             volt Emergency Bus # 2-21E, 2-22E and 2-41E
- 480             volt Busses # 2-31, 2-41, 2-32, 2-42, 2-35, 2-45, 2-36, 2-46
- 120             volt A.C. Vital Bus # 2-1V
- 120             volt A.C. Vital Bus # 2-2V
- 120             volt A.C. Vital Bus # 2-3V
- 120             volt A.C. Vital Bus # 2-4V

APPLICABILITY: RECOVERY MODE.

ACTION.

With less than the above complement of A.C. busses OPERABLE, restore the inoperable bus to OPERABLE status within 8 hours.



TABLE 3.8-1

TESTING FREQUENCY MATRIX

		Component Testing Frequencies						
		a <sub>1</sub>	a <sub>2</sub>	b <sub>1</sub>	b <sub>2</sub>	c <sub>1</sub>	c <sub>2</sub>	d
Inoperable Component	a <sub>1</sub>	X	*	*	*	*	*	*
	a <sub>2</sub>	*	X	*	*	*	*	*
	b <sub>1</sub>	*	*	X	*	#	#	#
	b <sub>2</sub>	*	*	*	X	#	#	#
	c <sub>1</sub>	*	*	#	#	X	*	#
	c <sub>2</sub>	*	*	#	#	*	X	#
	d	*	*	#	#	#	#	X

- Key: \*Within 8 hour and every 12 hours thereafter  
 #Within 8 hour and every 36 hours thereafter  
 a<sub>1</sub> Offsite power circuit No. 1  
 a<sub>2</sub> Offsite power circuit No. 2  
 b<sub>1</sub> Class 1E diesel generator (Red)  
 b<sub>2</sub> Class 1E diesel generator (Green)  
 c<sub>1</sub> Balance of Plant diesel generator (Gray)  
 c<sub>2</sub> Balance of Plant diesel generator (White)  
 d 13.2 kv line from Middletown Junction Substation

3.8.1.1 UNIT 2  
 3.8.1.1 UNIT 2

TABLE 3.8-2

RESTORATION TIME MATRIX

		Restore One Component (Hours)	Restore Other Component (Hours)
Combination of Two Inoperable Components	aa	24	72
	ab	12	72
	ac	12	72
	ad	12	72
	bb	12	72
	bc	12	72
	bd	12	72
	cc	12	72
	cd	12	72

Note: a, b, c and d above correspond to components described in Specification 3.8.1.1 items a, b, c and d respectively.

LIMITING CONDITIONS FOR OPERATION

RECOVER MODE

D.C. DISTRIBUTION

3.8.2.3 The following D. C. bus trains shall be energized and OPERABLE with tie breakers between bus trains open (unless closed in accordance with approved procedures):

TRAIN "A" consisting of 250/125-volt D.C. bus 2-1DC, 250/125-volt D.C. battery bank A and a full capacity charger.

TRAIN "B" consisting of 250/125-volt D.C. bus 2-2DC, 250/125-volt D.C. battery bank B, and a full capacity charger.

APPLICABILITY: RECOVERY MODE.

ACTION:

- a. With one 250/125-volt D.C. bus inoperable, restore the inoperable bus to OPERABLE status within 2 hours.
- b. With one 250/125-volt D.C. battery and/or its charger inoperable, restore the inoperable battery and/or charger to OPERABLE status within 2 hours.

LIMITING CONDITIONS FOR OPERATION

REVISION 1071

3.9 LIQUID RADIOACTIVE WASTE STORAGE

FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

3.9.12 The fuel handling building/auxiliary building air cleanup systems shall be OPERABLE with exhaust ventilation flow through the HEPA filters and charcoal adsorbers during system operation.

APPLICABILITY: At all times.

ACTION:

- a. With one fuel handling building/auxiliary building air cleanup system inoperable, restore the inoperable system to OPERABLE status within 7 days.
- b. With no fuel handling building/auxiliary building air cleanup system OPERABLE, suspend all operations involving movement of liquid and gaseous radioactive wastes until at least one fuel handling building/auxiliary building air cleanup system is restored to OPERABLE status.



SECTION 4.0

SURVEILLANCE REQUIREMENTS

#### 4 SURVEILLANCE REQUIREMENTS

#### 4.0 APPLICABILITY

#### SURVEILLANCE REQUIREMENTS

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4.0.1 Surveillance Requirements shall be applicable during the RECOVERY MODE or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement. The Surveillance Requirements shall be performed to demonstrate compliance with the OPERABILITY requirements of the Limiting Conditions for Operations and are contained in the RECOVERY OPERATIONS PLAN.

4.0.2 Each Surveillance Requirement shall be performed within the specified time interval with:

- a. A maximum allowable extension not to exceed 25% of the surveillance interval, and
- b. A total maximum combined interval time for any 3 consecutive tests not to exceed 3.25 times the specified surveillance interval.

4.0.3 Performance of a Surveillance Requirement within the specified time interval shall constitute compliance with OPERABILITY requirements for a Limiting Condition for Operation and associated ACTION statements unless otherwise required by the specification.

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BASES  
FOR  
LIMITING CONDITIONS FOR OPERATION  
AND  
SURVEILLANCE REQUIREMENTS

JUN 5 1979

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NOTE

The summary statements contained in this section provide the bases for the specifications of Sections 3.0 and 4.0 and are not considered a part of these technical specifications as provided in 10 CFR 50.36.

JUN 15 1979

### 3/4.0 APPLICABILITY

#### BASES

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The specifications of this section provide the general requirements applicable to each of the Limiting Conditions for Operation and Surveillance Requirements within Section 3/4.

3.0.1 This specification defines the applicability of each specification in terms of the RECOVERY MODE or other specified conditions and is provided to delineate specifically when each specification is applicable.

3.0.2 This specification defines those conditions necessary to constitute compliance with the terms of an individual Limiting Condition for Operation and associated ACTION requirement.

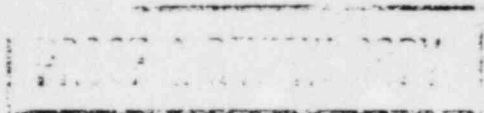
3.0.3 This specification delineates the ACTION to be taken for circumstances not directly provided for in the ACTION statements or when the provisions of the stated ACTION statement are not or cannot be complied with. The intent of this specification is to require that inoperable equipment be restored to an OPERABLE status in a prompt manner, that the unit be maintained in stable conditions, and that the Commission be promptly notified of such conditions.

4.0.1 This specification provides that surveillance activities necessary to insure the Limiting Conditions for Operation are met and will be performed during the RECOVERY MODE or other conditions for which the Limiting Conditions for Operation are applicable.

4.0.2 The provisions of this specification provide allowable tolerances for performing surveillance activities beyond those specified in the nominal surveillance interval. These tolerances are necessary to provide operational flexibility because of scheduling and performance considerations. The phrase "at least" associated with a surveillance frequency does not negate this allowable tolerance value and permits the performance of more frequent surveillance activities.

The tolerance values, taken either individually or consecutively over 3 test intervals, are sufficiently restrictive to ensure that the reliability associated with the surveillance activity is not degraded beyond that obtained from the nominal specified interval.

APPLICABILITY

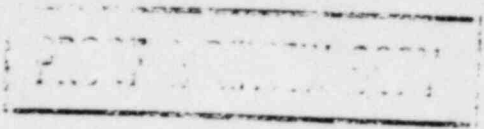


BASES

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4.0.3 The provisions of this specification set forth the criteria for determination of compliance with the OPERABILITY requirements of the Limiting Conditions for Operations. Under this criteria, equipment, systems or components are assumed to be OPERABLE if the associated surveillance activities have been satisfactorily performed within the specified time interval. Nothing in this provision is to be construed as defining equipment, systems or components OPERABLE, when such items are found or known to be inoperable although still meeting the Surveillance Requirements.



### 3/4.1 WATER INJECTION COOLING AND REACTIVITY CONTROL SYSTEMS

#### BASES

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#### 3/4.1.1 BORATION CONTROL

The limitation on minimum boron concentration ensures that the core will remain subcritical under all possible conditions which may exist during the long term cooling mode. The maximum boron concentration is provided to ensure that precipitation of boron will not occur in the reactor coolant system and thereby cause possible flow restrictions. The specification requiring the OPERABILITY of two systems capable of injecting borated cooling water into the Reactor Coolant system are provided to ensure that makeup water can be injected into the reactor coolant system and that the reactor coolant will be maintained within the required boron concentration limits.

#### 3/4.1.3 CONTROL ASSEMBLIES

All full length control rods were fully inserted as a result of the reactor trip on March 28, 1979. This specification requires that all control rods remain stationary thereby ensuring that the core's reactivity status will not be changed due to control rod movement. Maintaining the control rod drive breakers open provides assurance that the rods will not be inadvertently withdrawn.



### 3/4.3 INSTRUMENTATION

#### BASES

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#### 3/4.3.1 NEUTRON MONITORING INSTRUMENTATION

The neutron monitoring instrumentation, which was included in the normal Reactor Protection System Instrumentation, provides information regarding the shutdown status of the core and it will be used to monitor the core and to ensure that the core remains subcritical.

#### 3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

Except for automatic starting of the diesel generators on loss of offsite power, all automatic features of the ESFAS instrumentation have been defeated. This action prevents inadvertent actuation of the ESF systems.

The diesel generators will start automatically on loss of offsite power.

#### 3/4.3.3 MONITORING INSTRUMENTATION

##### 3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that 1) the radiation levels are continually measured in the areas served by the individual channels and 2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded.

##### 3/4.3.3.3 SEISMIC INSTRUMENTATION

The OPERABILITY of the seismic instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event so that the response of those features important to safety may be evaluated. This capability is required to permit comparison of the measured response to that used in the design basis for the facility. This instrumentation is consistent with the recommendations of Regulatory Guide 1.12, "Instrumentation for Earthquakes," April 1974. Due to the high radiation levels in certain areas of the facility, the surveillance requirements for these and various monitoring instruments include provisions excluding inaccessible instruments from the required surveillance activities.

## INSTRUMENTATION

### BASES

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#### 3/4.3.3.4 METEOROLOGICAL INSTRUMENTATION

The OPERABILITY of the meteorological instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public. This instrumentation is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs," February 1972.

#### 3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit monitoring of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criterion 19 of Appendix A, 10 CFR 50.

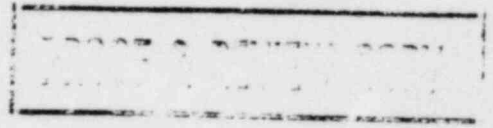
#### 3/4.3.3.6 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following the incident of March 28, 1979.

#### 3/4.3.3.7 CHLORINE DETECTION SYSTEMS

The OPERABILITY of the chlorine detection systems ensures that an accidental chlorine release will be detected promptly and the control room emergency ventilation system will automatically isolate the control room and initiate its operation in the recirculation mode to provide the required protection. The chlorine detection systems required by this specification are consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operations Against an Accidental Chlorine Release," February 1975.

## INSTRUMENTATION



### BASES

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#### 3/4.3.3.8 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is returned to service.

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3/4.4 REACTOR COOLANT SYSTEM

BASES

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3/4.4.1 REACTOR COOLANT LOOPS

Operation of a reactor coolant loop in natural circulation with heat rejection through at least one steam generator provides adequate cooling of the core in its present condition. To the extent possible, the reactor coolant pumps are being maintained OPERABLE in a standby basis. In the event this present mode of cooling the core is lost, procedures have been prepared and approved which will provide alternate means for cooling the core.

3/4.4.3 SAFETY VALVES

The pressurizer code safety valves operate to prevent the RCS from being pressurized above its Safety Limit of 2750 psig. Each safety valve is designed to relieve 348,072 lbs per hour of saturated steam at the valve's setpoint.

3/4.4.9 PRESSURE/TEMPERATURE LIMIT

The RCS pressure and temperature will be controlled in accordance with approved procedures to prevent a nonductile failure of the RCS while at the same time permitting the RCS pressure to be maintained at a sufficiently high value to permit operation of the reactor coolant pumps. (See Figure B3/4 4-1)

THREE MILE ISLAND - UNIT 2  
 B3/44-1a

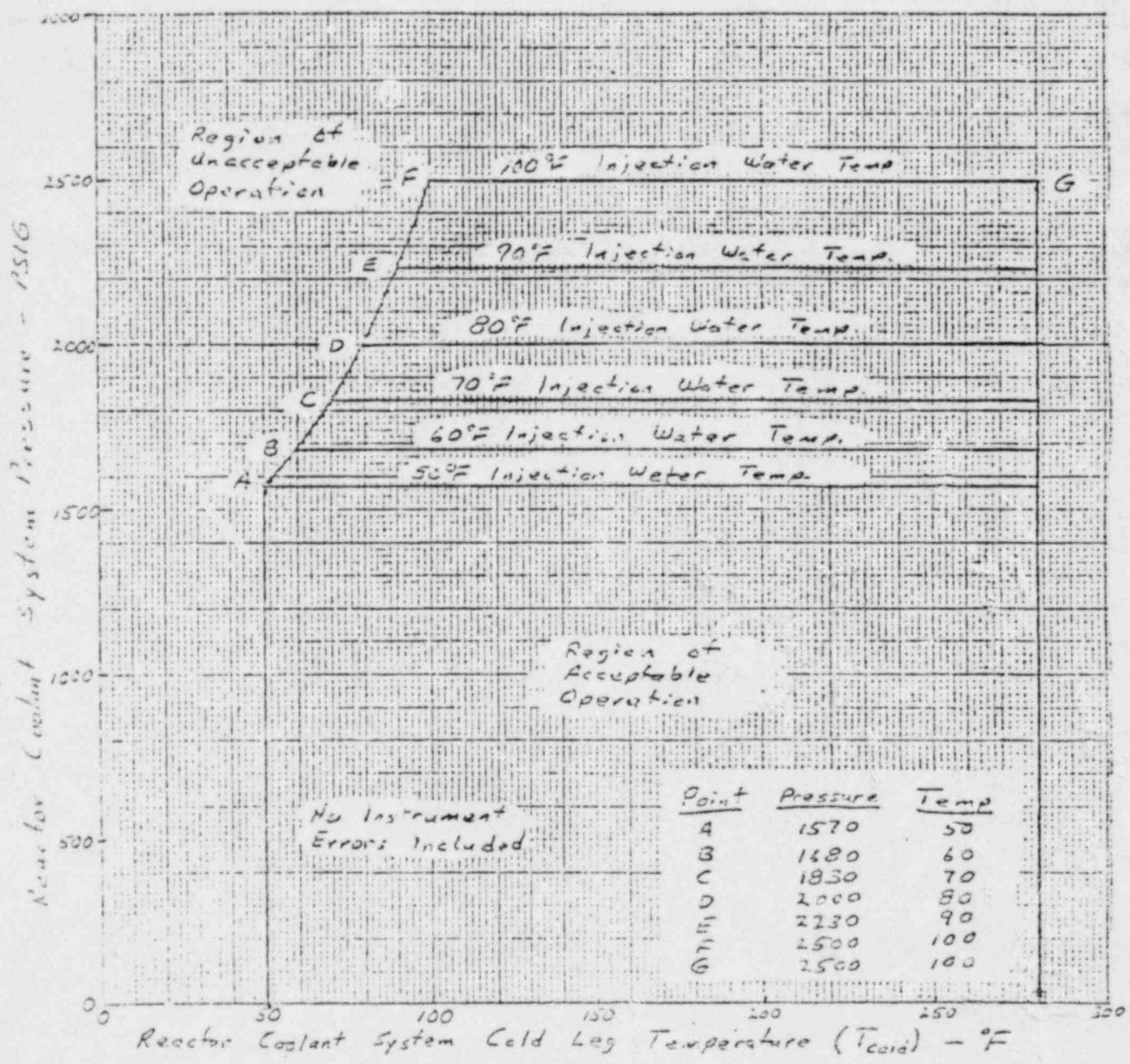


Figure B3/44-1

RCS Pressure vs RCS Cold Leg Temperature  
 Allowable Operating Envelope for  
 Reactor Vessel NDT Limits



## 3/4.6 CONTAINMENT SYSTEMS

### BASES

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#### 3/4.6.1 PRIMARY CONTAINMENT

##### 3/4.6.1.1 CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY must be maintained to ensure that the radioactive materials which were released to the containment atmosphere during the March 28, 1979 incident will not be released to the atmosphere.

##### 3/4.6.1.3 CONTAINMENT AIR LOCKS

The containment air locks must be maintained OPERABLE to provide CONTAINMENT INTEGRITY. These air locks will be used during future entries into the containment to ensure that the containment atmosphere will not be released to the environs.

##### 3/4.6.1.2 INTERNAL PRESSURE

The negative pressure limit provides assurance that the containment will not exceed its design negative pressure differential. The positive pressure limit provides assurance that leakage from the containment will be limited for dose considerations.

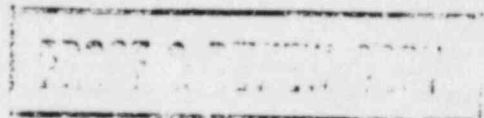
##### 3/4.6.1.5 AIR TEMPERATURE

The average air temperature of the containment atmosphere is currently being maintained well below the 130°F limit. The action will maximize the service life of the instrumentation and equipment installed in the containment. Continued OPERABILITY of these items is required to continue monitoring and mitigating the March 28, 1979 incident.

##### 3/4.6.4 COMBUSTIBLE GAS CONTROL

The gas partitioner is provided to analyze the hydrogen concentration in the containment atmosphere so that actions can be initiated to reduce the hydrogen concentration if it approaches its flammable limit. If excessive hydrogen concentrations are detected, operation of the hydrogen recombiner will be initiated to reduce the hydrogen concentration to a safe level.

## 3/4.7 PLANT SYSTEMS



### BASES

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#### 3/4.7.1 FEEDWATER SYSTEM

The emergency feedwater system is required to be maintained in an OPERABLE status since it is a backup means of supplying water to the steam generators to remove the decay heat from the reactor coolant system. Decay heat can be removed from the reactor coolant system by either steam generator. The "A" steam generator can perform this cooling function using feedwater supplied by any one of the three condensate pumps. The "B" steam generator can be used for this cooling function by using its newly installed closed loop cooling system.

#### 3/4.7.2 SECONDARY SERVICES CLOSED COOLING WATER SYSTEM

The secondary services closed cooling water system is required to be maintained in an OPERABLE condition since it is used to cool the "B" steam generator closed loop cooling system.

#### 3/4.7.3.1 NUCLEAR SERVICES CLOSED CYCLE COOLING SYSTEM

OPERABILITY of the nuclear services closed cycle cooling system is required during operation of the reactor coolant and makeup pumps since this system provides cooling water to the motor coolers and bearings of these pumps.

#### 3/4.7.3.2 DECAY HEAT CLOSED COOLING WATER SYSTEM

The decay heat closed cooling water system is required to be maintained in an OPERABLE status since it is provided to remove heat from the DHR system which is being maintained OPERABLE in a backup status for possible core cooling.

#### 3/4.7.4 NUCLEAR SERVICE RIVER WATER SYSTEM

The nuclear service river water system uses river water to cool the nuclear services closed cycle cooling system, the secondary services closed



## CONTAINMENT SYSTEMS

### BASES

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cooling water system, and decay heat closed cooling water system; therefore, it must be OPERABLE too. This system rejects its heat to the river as the ultimate heat sink.

#### 3/4.7.6 FLOOD PROTECTION

The limitation on flood protection ensures that facility protective actions will be taken in the event of flood conditions. The limit of elevation 302 Mean Sea Level is based on the maximum elevation at which facility flood control measures provide protection to safety related equipment.

#### 3/4.7.7 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM

The OPERABILITY of the control room emergency air cleanup system ensures that 1) the ambient air temperature does not exceed the allowable temperature for continuous duty rating for the equipment and instrumentation cooled by this system and 2) the control room will remain habitable for operations personnel during and following all credible accident conditions. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criterion 19 of Appendix "A", 10 CFR 50.

#### 3/4.7.10 FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, Halon and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program. Any two of the four main fire pumps provide combined capacity greater than 3575 gpm.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the affected equipment can be restored to service.

In the event that the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a Special Report to the Commission provides for timely evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued operation of the nuclear plant.

## CONTAINMENT SYSTEMS

### BASES

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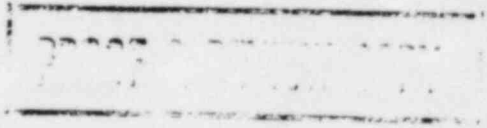
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#### 3/4.7.11 PENETRATION FIRE BARRIER

The functional integrity of the fire barrier penetration seals ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The fire barrier penetration seals are a positive element in the facility fire protection program and are subject to periodic inspections.

During periods of time when the seals are not functional, a roving fire watch is required to be maintained in the vicinity of the affected seal until the seal is restored to functional status.

3/4.8 ELECTRICAL POWER SYSTEMS



BASES

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The OPERABILITY of the A.C. and D.C. power sources and associated distribution systems during operation ensures that sufficient power will be available to supply the safety related equipment required to maintain the unit in stable conditions during recovery from the March 28, 1979 incident.

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3/4.9 LIQUID RADIOACTIVE WASTE STORAGE

BASES

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3/4.9.12 FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

The requirements for the fuel handling/auxiliary building air cleanup systems to be operating or OPERABLE ensure that all radioactive material released from the liquid radioactive wastes being stored in the new radwaste storage tanks which have been installed in the spent fuel storage pool or elsewhere in the auxiliary building will be filtered through the HEPA filters and charcoal adsorbers prior to release to the atmosphere.

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SECTION 5.0  
DESIGN FEATURES

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## 5.0 DESIGN FEATURES

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### 5.1 SITE

#### EXCLUSION AREA

5.1.1 The exclusion area is shown on Figure 5.1-1.

#### LOW POPULATION ZONE

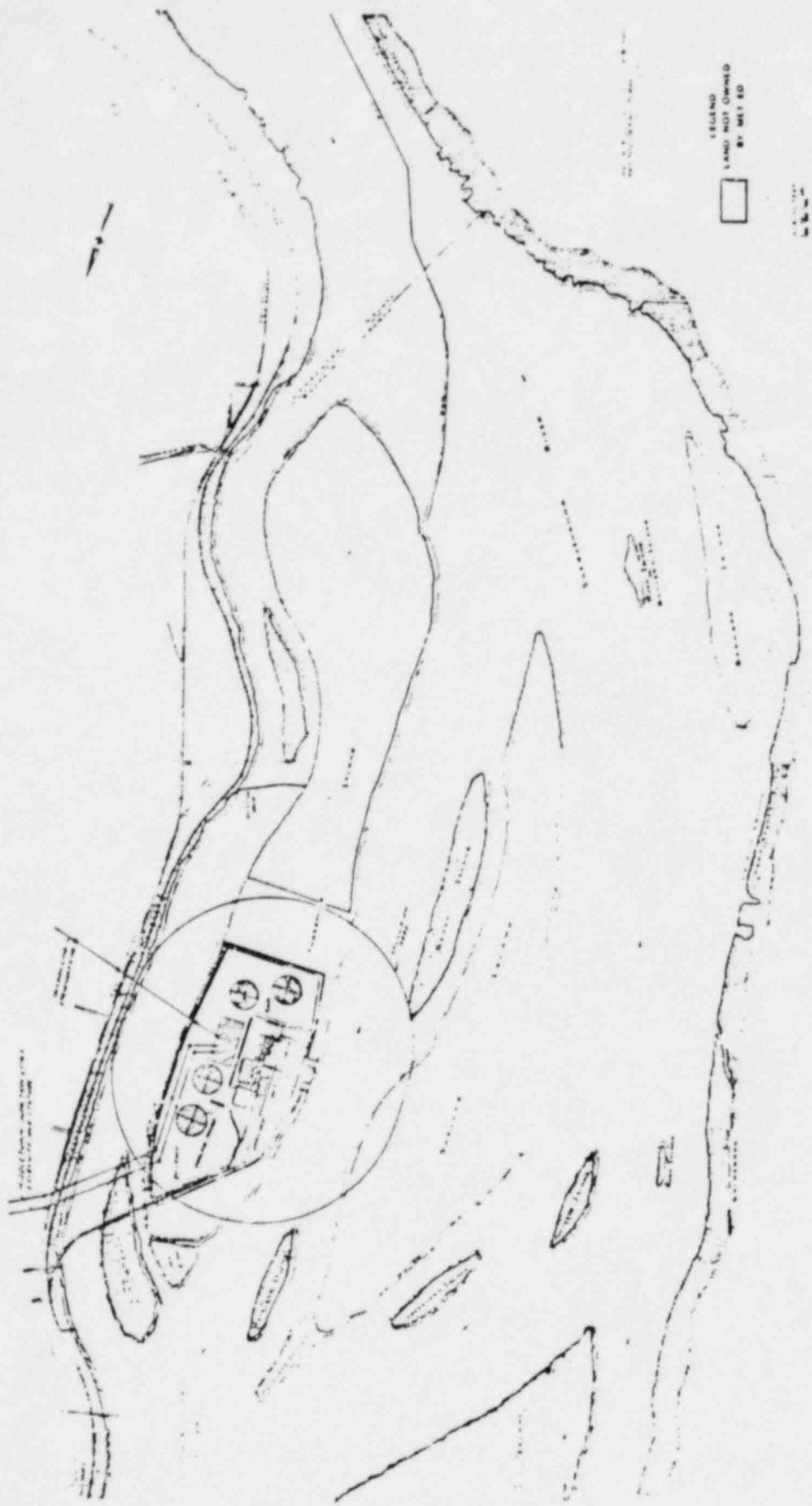
5.1.2 The low population zone is shown on Figure 5.1-2.

### 5.2 CONTAINMENT

#### CONFIGURATION

5.2.1 The reactor containment building is a steel lined, reinforced concrete building of cylindrical shape, with a dome roof and having the following design features:

- a. Nominal inside diameter = 130 feet.
- b. Nominal inside height = 157 feet.
- c. Minimum thickness of concrete walls = 4 feet.
- d. Minimum thickness of concrete roof = 3.5 feet.
- e. Minimum thickness of concrete floor pad = 13.5 feet.
- f. Nominal thickness of steel liner = 1/2 inches.
- g. Net free volume =  $2.1 \times 10^6$  cubic feet.

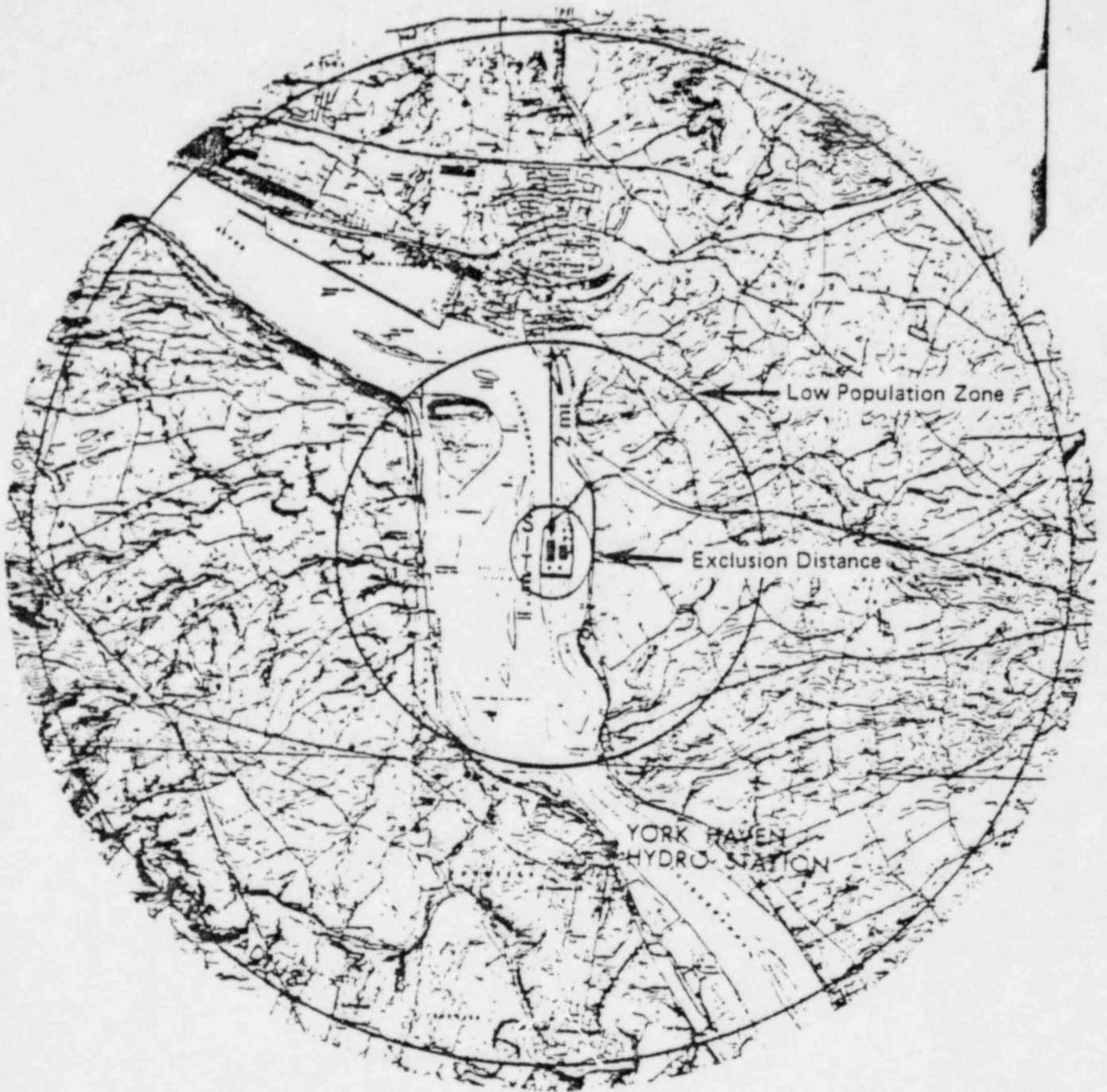


EXCLUSION AREA

FIGURE 5.1-1



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LOW POPULATION ZONE

FIGURE 5.1-2

## DESIGN FEATURES

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### DESIGN PRESSURE AND TEMPERATURE

5.2.2 The reactor containment building is designed and shall be maintained for a maximum internal pressure of 60 psig and a temperature of 286°F.

### 5.3 REACTOR CORE

#### FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 177 fuel assemblies with each fuel assembly containing 208 fuel rods clad with Zircaloy-4. Each fuel rod shall have a nominal active fuel length of 143.5 inches and contain a nominal total weight of 2252 grams uranium. The initial core loading shall have a maximum nominal enrichment of 2.96 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading and shall have a maximum enrichment of 3.50 weight percent U-235.

The first cycle fuel loading shall contain 68 burnable poison rod assemblies with each assembly containing up to 16 burnable poison rods of sintered  $Al_2O_3-B_4C$  clad with Zircaloy-4.

#### CONTROL RODS

5.3.2 The reactor core shall contain 61 safety and regulating and 8 axial power shaping (APSR) control rods. The safety and regulating control rods shall contain a nominal 134 inches of absorber material. The APSR's shall contain a nominal 36 inches of absorber material at their lower ends. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

## DESIGN FEATURES

### 5.4 REACTOR COOLANT SYSTEM

#### DESIGN PRESSURE AND TEMPERATURE

5.4.1 The reactor coolant system is designed and shall be maintained:

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to applicable Surveillance Requirements.
- b. For a pressure of 2500 psig, and
- c. For a temperature of 650°F, except for the pressurizer and pressurizer surge line which is 670°F.

#### VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is 12,180 ± 200 cubic feet at a nominal  $T_{avg}$  of 525°F.

### 5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.5-1.

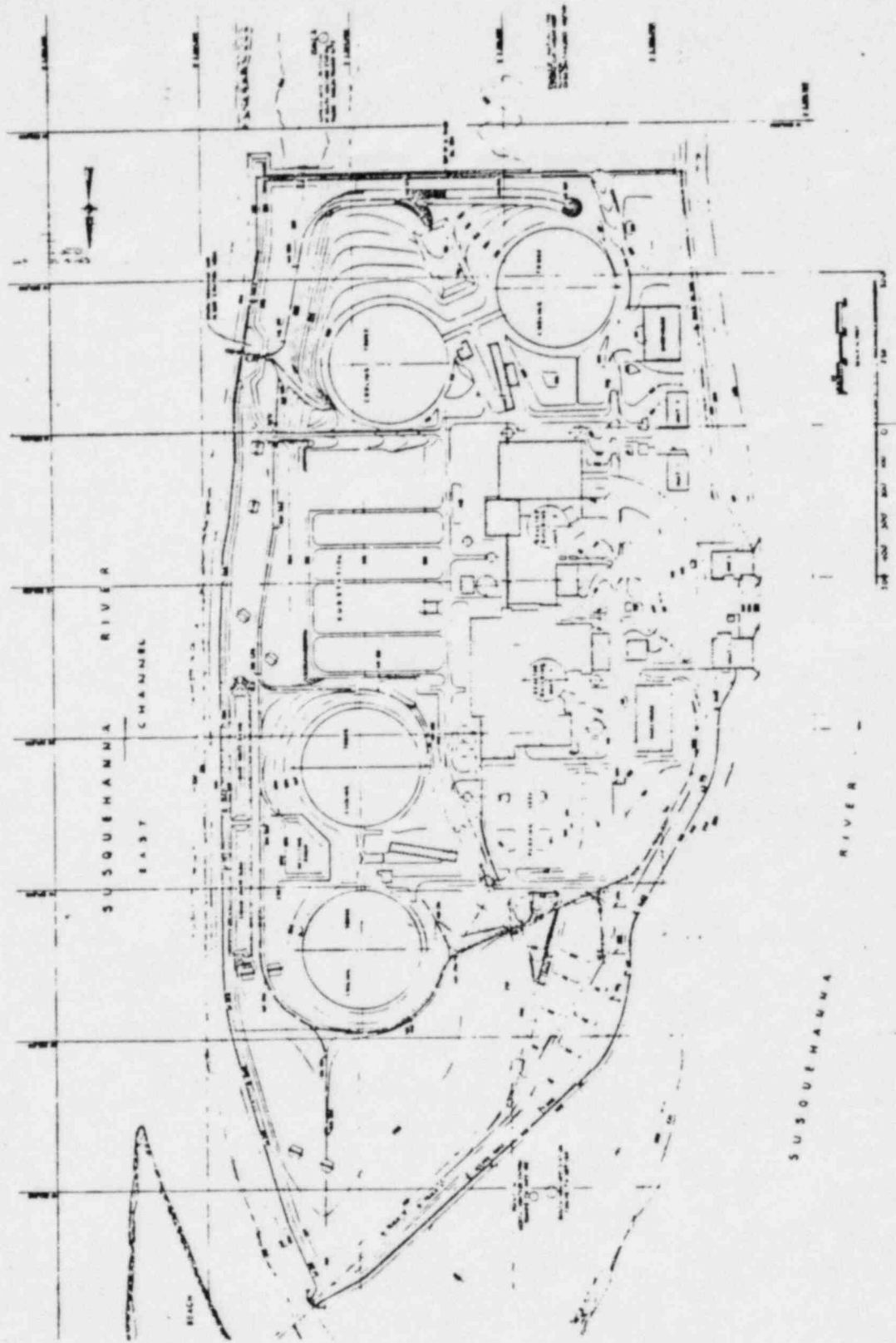
### 5.6 FUEL STORAGE

#### CRITICALITY

5.6.1 The new and spent fuel storage racks are designed and shall be maintained with a nominal 21 1/8 inch center-to-center distance between fuel assemblies placed in the storage racks to ensure a  $k_{eff}$  equivalent to  $\leq 0.95$  with the storage pool filled with unborated water. The  $k_{eff}$  of  $\leq 0.95$  includes a conservative allowance of  $> 1.0\% \Delta k/k$  for uncertainties as described in Section 4.3 of the FSAR.

#### DRAINAGE

5.6.2 The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 345'6".



THREE MILE ISLAND - UNIT 2

5-6

METEOROLOGICAL TOWER LOCATIONS

Figure 5.5-1

DESIGN FEATURES

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CAPACITY

5.6.3 The spent fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 443 fuel assemblies.

5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7-1 are designed and shall be maintained within the cyclic or transient limit of Table 5.7-1.

TABLE 5.7-1

COMPONENT CYCLIC OR TRANSIENT LIMITS

<u>Component or System</u>	<u>Cycle or Transient Limit</u>	<u>Design Cycle or Transient</u>
1. Reactor Coolant System	240 Heatup and Cooldown Cycles each	70°F to 557°F to 70°F
2. Reactor Coolant System	160 Step Load Reduction Cycles (Resulting from turbine trip)	100% to 8% RTP*
3. Reactor Coolant System	150 Step Load Reduction Cycles (Resulting from electrical load rejection)	100% to 8% RTP*
4. Reactor Coolant System	40 Reactor Trip Cycles (Resulting from loss of electric power to all RC pumps)	Reactor Trip
5. Reactor Coolant System	160 Reactor Trip Cycles (Resulting from turbine trip without automatic control action)	Reactor Trip
6. Reactor Coolant System	40 Reactor Trip Cycles (Resulting from rod withdrawal accident)	Reactor Trip
7. Once Through Steam Generator	88 Reactor Trip Cycles (Resulting from complete loss of all main feed-water)	Reactor Trip
8. Once Through Steam Generator	40 Reactor Trip Cycles (Resulting from loss of station power)	Reactor Trip
9. Once Through Steam Generator	20 Reactor Trip Cycles (Resulting from loss of feedwater to one steam generator)	Reactor Trip

\*RATED THERMAL POWER



TABLE 5.7-1 (Continued)

<u>Component or System</u>	<u>Cycle or Transient Limit</u>	<u>Design Cycle or Transient</u>
10. Once Through Steam Generator	10 Reactor Trip Cycles (Resulting from stuck open turbine bypass valve)	Reactor Trip
11. Reactor Coolant System	40 Rapid Depressurization	2200 psig to 300 psig in one hour
12. Reactor Coolant System	20 Change of Flow Cycles	Loss of one or more RC pumps
13. Reactor Coolant System	20 Hydrostatic Test	Pressurized to $\geq$ 3125 psig
14. Once Through Steam Generator	35 Hydrostatic Tests Secondary Side	Pressurized to $\geq$ 1312.5 psig
15. Reactor Coolant System	40 Test Transients	High Pressure Injection Test
16. Reactor Coolant System	240 Test Transients	Core Flooding Check Valve Test



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SECTION 6.0A  
ADMINISTRATIVE CONTROLS

JUN 25 1979