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June 13, 1994

U.S. Nuclear Regulatory Commission
ATTN.: Document Control Desk
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Donald F. Schnell
Senior Vice President
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ULNRC-3031
TAC No. 83445

Gentlemen:

**CALLAWAY PLANT
DOCKET NUMBER 50-483
REACTOR VESSEL STRUCTURAL
INTEGRITY, 10 CFR 50.54(f)**

- Ref.: 1) Generic Letter 92-01,
Revision 1, dated March 6, 1992
2) ULNRC-2649, dated June 16, 1992
3) L. R. Wharton Letter to D. F. Schnell
dated May 12, 1994


NRC Generic Letter 92-01 requested that licensees provide information concerning their reactor pressure vessels. This information will be used by NRC to confirm that licensees satisfy the requirements and commitments necessary to ensure reactor vessel integrity at their facilities.

By letter dated May 12, 1994 (Reference 3) the NRC Staff determined that Union Electric had provided the information requested by GL 92-01. This letter also transmitted tables which contain pressurized thermal shock (PTS) and upper-shelf energy (USE) information for Callaway Plant. The staff requested that Union Electric perform an accuracy review of the information contained in the tables.

Enclosed please find the corrected tables of PTS and USE information for Callaway Plant. This data was taken from our response to GL 92-01 and previously docketed information. Please review this information and update your database accordingly.

If you have any questions concerning this information, please contact us.

Very truly yours,


for Donald F. Schnell

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Enclosures

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Summary File for Pressurized Thermal Shock

| Plant Name | Beltline Ident. | Heat No. Ident. | ID Neut. Fluence at EOL/EFY | IRT _{ndt} | Method of Determin. IRT _{ndt} | Chemistry Factor | Method of Determin. CF | %Cu | %Ni |
|--------------------------------------|---|-----------------|-----------------------------|--------------------|--|------------------|------------------------|------|------|
| Callaway 1 EOL: 10/18/ 2024 | Int. Shell R2707-1 | C4344-1 | 2.39E19 | 40°F | Plant Specific | 26 | Table | 0.04 | 0.57 |
| | Int. Shell R2707-2 | C4383-1 | 2.39E19 | 10°F | Plant Specific | 31 | Table | 0.05 | 0.59 |
| | Int. Shell R2707-3 | C4383-2 | 2.39E19 | -10°F | Plant Specific | 37 | Table | 0.06 | 0.61 |
| | Lower Shell R2708-1 | C4499-2 | 2.39E19 | 50°F | Plant Specific | 26.139 | Calculated | 0.07 | 0.59 |
| | Lower Shell R2708-2 | C4472-1 | 2.39E19 | 10°F | Plant Specific | 31 | Table | 0.05 | 0.57 |
| | Lower Shell R2708-3 | C4499-1 | 2.39E19 | 20°F | Plant Specific | 44 | Table | 0.07 | 0.59 |
| | Int. and Lower Shell Axial Welds G2.03 | 90077 | 2.39E19 | -60°F | Plant Specific | 29.7 | Table | 0.04 | 0.06 |
| | Int. to Lower Shell Circ. Weld E3.14 | 90077 | 2.39E19 | -60°F | Plant Specific | 56.144 | Calculated | 0.04 | 0.07 |

References

Chemical composition and IRT_{ndt} are from Table A-3 of R. G. Lott, et. al, "Analysis of Capsule U from the Union Electric Company Callaway Unit 1 Reactor Vessel Radiation Surveillance Program," WCAP-11374, Revision 1, Westinghouse Electric Corporation, Pittsburgh, PA 15230, June 1987.

Fluence data are from Table 6-14 of WCAP-12946 and December 18, 1991, Letter from D. F. Schnell (UECo) to USNRC Document Control Desk, subject: Revision to Technical Specification 3/4.4.9 Pressure Temperature Limits.

Summary File for Upper Shelf Energy

| Plant Name | Beltline Ident. | Heat No. | Material Type | 1/4T USE at EOL | 1/4T Neutron Fluence at EOL | Unirrad. USE | Method of Determin. Unirrad. USE |
|--|---|----------|-----------------------------------|-----------------|-----------------------------|--------------|----------------------------------|
| Callaway 1 EOL: 10/18/ 2024 | Int. Shell R2707-1 | C4344-1 | A 533B-1 | 62 | 1.31E19 | 78 | Direct |
| | Int. Shell R2707-2 | C4383-1 | A 533B-1 | 79 | 1.31E19 | 100 | Direct |
| | Int. Shell R2707-3 | C4383-2 | A 533B-1 | 79 | 1.31E19 | 99 | Direct |
| | Lower Shell R2708-1 | C4499-2 | A 533B-1 | 65 | 1.31E19 | 82 | Direct |
| | Lower Shell R2708-2 | C4472-1 | A 533B-1 | 83 | 1.31E19 | 105 | Direct |
| | Lower Shell R2708-3 | C4499-1 | A 533B-1 | 80 | 1.31E19 | 101 | Direct |
| | Int. and Lower Shell Axial Welds, G2.03 | 90077 | Linde 0091 Lot No. 0842 SAW | 113 | 1.31E19 | 143 | Direct |
| | Int. to Lower Shell Circ. Weld, E3.14 | 90077 | Linde 124, Lot No. 1061 SAW | 87 | 1.31E19 | 112 | Direct |

References

USE data are from Table A-3 of R.G. Lott et. al. "Analysis of Capsule U from the Union Electric Company Callaway Unit 1 Reactor Vessel Radiation Surveillance Program," WCAP-11374, Revision 1, Westinghouse Electric Corporation, Pittsburgh, PA 15230, June 1987.

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