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C321-94-2102
June 14, 1994

U.S. Nuclear Regulatory Commission
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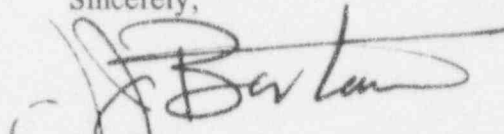
Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report - May, 1994

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Brenda DeMerchant, Oyster Creek Licensing Engineer at (609) 971-4642.

Sincerely,



J. J. Barton
Vice President and Director
Oyster Creek

JJB/BDEM: jc
Attachment
(MOR-RPT.MAY)

cc: Administrator, Region I
Senior NRC Resident Inspector
Oyster Creek NRC Project Manager

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SUMMARY
MAY, 1994

Oyster Creek entered May at full power operation. On May 22 at 1110 hours, the plant commenced a technical specification shutdown due to failure of the 'B' Control Rod Drive pump discharge stop check valve, V-15-10. The generator was taken off line at 1832 hours. Following repairs, the plant went back on line at 1603 on May 26. On May 31 at 1332 hours while operating at full power, an automatic reactor scram occurred on a turbine-generator trip due to JCP&L work in the switchyard inadvertently actuating a generator protective relay. The plant remained in shutdown mode for the remainder of the reporting period.

The plant generated 393,364 net megawatt-hours electric and attained a MDC net capacity factor of 86.7% for the month.

MONTHLY OPERATING REPORT

LICENSEE EVENT REPORTS

The following Licensee Event Reports were submitted during the month of May, 1994:

LER 94-003

On 04/05/94 at 1550 hrs. a turbine trip and reactor scram occurred due to high reactor water level. The cause of the event was the GE/MAC Proportional amplifier, (ID-23J), which provides a density correction signal to both A& B steam flow signals. This resulted in a false high steam flow signal which resulted in the feedwater control system increasing feedwater flow to the reactor which caused reactor water level to increase. When water level reached the high level turbine trip setpoint of 175", the turbine tripped and a reactor scram occurred as designed. The faulty electronics modules were replaced and the feedwater flow control system was returned to operational status.

LER 94-004

Calculations performed as part of the Generic Letter 89-10 Motor-Operated Valve (MOV) Program for the Isolation Condenser System (ICS) DC-powered isolation valves determined that the voltage available at the motor operator combined with the dynamic blowdown conditions existing during ICS high energy line break (HELB) isolation would produce valve closure times greater than the design condition of 60 seconds identified in the Final Safety Analysis Report (FSAR) Section 6.3.2.5 and Table 6.2-12. The root cause of this condition was the reevaluation of the design capabilities of the motor-operated valves as a result of the Generic Letter 89-10 program recommendations.

The safety significance is considered to be minimal since the calculated valve stroke times for the subject valves meet the design criteria for establishing isolation times for high energy line breaks (HELB). These criteria are defined in Oyster Creek Technical Specification Section 4.5 Basis and Oyster Creek FSAR Section 6.3.2.5.

The 60 second isolation design time requirement in the FSAR will be revised for these valves, in conjunction with the results of the ongoing Oyster Creek Generic Letter 89-10 MOV Program.

REFUELING INFORMATION - MAY, 1994

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Currently projected for
September 10, 1994

Scheduled date for restart following refueling: Currently projected for
November 14, 1994

Will refueling or resumption of operation thereafter require a Technical
Specification change or other license amendment?

No

Important licensing considerations associated with refueling, e.g., new or
different fuel design or supplier, unreviewed design or performance analysis
methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance
analysis methods have been approved by the NRC.

The number of fuel assemblies	(a) in the core	=	560
	(b) in the spent fuel storage pool	=	1962
	(c) in dry storage	=	8

The present licensed spent fuel pool storage capacity and the size of any
increase in licensed storage capacity that has been requested or is planned, in
number of fuel assemblies:

Present Licensed Capacity: 2600

Planned Increase in Licensed Storage Capacity: 45

The actual fuel storage capacity of the spent fuel pool is 2645 assemblies.

The projected date of the last refueling that can be discharged to the spent fuel
pool assuming the present licensed capacity:

Based on a projected reload of 172 bundles, full core discharge capacity
to the spent fuel pool will be lost after the 1994 refueling outage.

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET # 50 219
UNIT OYSTER CREEK #1
REPORT DATE 06-06-94
COMPILED BY JIM KRALL
TELEPHONE # 609-971-4115

MONTH: MAY, 1994

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	619	16.	617
2.	622	17.	623
3.	621	18.	622
4.	623	19.	624
5.	625	20.	626
6.	626	21.	625
7.	625	22.	421
8.	625	23.	0
9.	624	24.	0
10.	624	25.	0
11.	622	26.	101
12.	620	27.	609
13.	625	28.	619
14.	625	29.	618
15.	621	30.	615
		31.	343

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: 5/94
3. UTILITY CONTACT: JIM KRALL (609)971-4115
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): 687.5 x 0.8 = 550
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 632
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 610
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS:
NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):
NONE
11. REASON FOR RESTRICTION, IF ANY:
NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HOURS	744.0	623.0	214223.0
13. HOURS RX CRITICAL	658.0	3367.6	142966.6
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	918.2
15. HRS GENERATOR ON-LINE	640.0	3323.0	139524.7
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENERGY (MWH)	1224305	6262276	239375653
18. GROSS ELEC ENERGY (MWH)	409169	2105840	80350154
19. NET ELEC ENERGY (MWH)	393364	2026000	77073714
20. UT SERVICE FACTOR	86.0	91.7	65.1
21. UT AVAIL FACTOR	86.0	91.7	65.1
22. UT CAP FACTOR (MDC NET)	86.7	91.7	58.7
23. UT CAP FACTOR (DER NET)	81.3	86.0	55.4
24. UT FORCED OUTAGE RATE	14.0	8.3	10.4
25. FORCED OUTAGE HRS	104.0	300.0	16257.3

26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):

15-R OUTAGE, SEPTEMBER 10, 1994, 55 - 65 DAYS

27. IF CURRENTLY SHUTDOWN, ESTIMATED STARTUP DATE:

6/2/94

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-219
 UNIT NAME: Oyster Creek
 DATE: June 06, 1994
 COMPLT'D BY: David M. Egan
 TELEPHONE: 971-4818

REPORT MONTH: May 1994

No.	DATE	TYPE		DURATION (hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
		F: Forced	S: Scheduled				
4	940522	F		93.5	a	1	The reactor was manually shutdown due to failure of 'B' Control Rod Drive Pump stop check valve to prevent reverse flow.
5	940531	F		10.5	a	3	Automatic Reactor SCRAM on turbine-generator trip due to JCP&L work in switchyard inadvertently actuating a generator protective relay.

SUMMARY:

(1) REASON

- | | |
|--------------------------------|---------------------------------|
| a. Equipment Failure (Explain) | e. Operator Training & Lic Exam |
| b. Maintenance or Test | f. Administrative |
| c. Refueling | g. Operational Error (Explain) |
| d. Regulatory Restriction | h. Other (Explain) |

(2) METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)