



A Centenor Energy Company

EDISON PLAZA
300 MADISON AVENUE
TOLEDO, OHIO 43652-0001

June 14, 1994

KB-94-1279

Docket No. 50-346

License No. NPF-3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Monthly Operating Report, May 1994
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of May 1994.

If you have any questions, please contact S. D. Koch at (419) 321-7791.

Very truly yours,

A handwritten signature in cursive script that reads 'John K. Wood'.

John K. Wood
Plant Manager
Davis-Besse Nuclear Power Station

SDK/dmc

Enclosures

cc: Mr. J. B. Martin
Region III Administrator

Mr. S. Stasek
NRC Senior Resident Inspector

Mr. G. West, Jr.
NRC Senior Project Manager

9406170375 940531
PDR ADDCK 05000346
R PDR

Handwritten initials 'JE24' with a vertical line drawn through the bottom of the '4'.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-0346

UNIT Davis-Besse Unit 1

DATE 6-1-94

COMPLETED BY STEVE KOCH

TELEPHONE 419-321-7791

MONTH MAY 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	878	17	873
2	878	18	875
3	877	19	874
4	876	20	870
5	874	21	849
6	876	22	866
7	876	23	870
8	874	24	872
9	875	25	870
10	876	26	873
11	873	27	876
12	875	28	875
13	868	29	870
14	661	30	867
15	860	31	864
16	869		

OPERATING DATA REPORT

DOCKET NO 50-0346
 DATE 6-1-94
 COMPLETED BY Steve Koch
 TELEPHONE 419-321-7791

OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period.....MAY 1994
3. Licensed Thermal Power (MWt).....2772
4. Nameplate Rating (Gross MWe).....925
5. Design Electrical Rating (Net MWe).....906
6. Maximum Dependable Capacity (Gross MWe).....913
7. Maximum Dependable Capacity (Net MWe).....868
8. If Changes Occur in Capacity Ratings
 (Items number 3 through 7) since last report, give reasons: _____

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any (Net MWe): _____

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period.....	744.00	3,623.00	138,816.00
12. Number Of Hours Reactor Was Critical.....	744.00	3,623.00	85,863.45
13. Reactor Reserve Shutdown Hours.....	0.00	0.00	5,532.00
14. Hours Generator On-Line.....	744.00	3,623.00	83,646.73
15. Unit Reserve Shutdown Hours.....	0.00	0.00	1,732.50
16. Gross Thermal Energy Generated (MWH).....	2,044,405	10,018,451	215,790,220
17. Gross Electrical Energy Generated (MWH).....	676,739	3,326,296	69,666,317
18. Net Electrical Energy Generated (MWH).....	643,447	3,163,423	65,689,793
19. Unit Service Factor.....	100.00	100.00	60.26
20. Unit Availability Factor.....	100.00	100.00	61.51
21. Unit Capacity Factor (Using MDC Net).....	99.64	100.59	54.52
22. Unit Capacity Factor (Using DER Net).....	95.46	96.37	52.23
23. Unit Forced Outage Rate.....	0.00	0.00	20.60
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
Scheduled maintenance and refueling outage - October 1, 1994			
Planned duration - 54 days			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.: 50-346
 UNIT NAME: Davis-Besse #1
 DATE: June 6, 1994
 Completed by: S. D. Koch
 Telephone: (419)321-7791

Report Month May, 1994

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
01	94-05-13	S	0	B	5	N/A	N/A	N/A	Planned power reduction to approx. 55 percent to clean low pressure condenser loop 2 waterbox.
02	94-05-20	S	0	B	5	N/A	N/A	N/A	Moderator temperature coefficient testing.

¹F: Forced
 S: Scheduled

²Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Continuation from Previous Month
 5-Load Reduction
 9-Other (Explain)

⁴Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I - Same Source
 *Report challenges to Power Operated Relief Valves (PORVs) and Pressurizer Code Safety Valves (PCSVs)

OPERATIONAL SUMMARY
May 1994

Reactor power was maintained at approximately 100 percent full power until 2200 hours on May 13, 1994, when a manual power reduction was initiated to perform main turbine control valve testing, combined intercept valve testing, and clean the loop 2 low pressure condenser waterbox. Power was manually reduced to approximately 92 percent, at which point main turbine control valve testing was performed. Power was then manually reduced to approximately 55 percent by 0400 hours on May 14, 1994, at which point the loop 2 low pressure condenser cleaning and combined intercept valve testing were initiated. Reactor power was slowly increased to approximately 76 percent at 1130 hours on May 14, 1994, and manually reduced to approximately 70 percent at 1315 hours on May 14, 1994 limited by condenser vacuum. Following CIV testing and loop 2 low pressure condenser waterbox cleaning, reactor power was slowly increased to 100 percent full power, which was achieved at 1815 hours on May 14, 1994. Reactor power was maintained at this level until May 19, 1994.

At approximately 1755 on May 19, 1994, following maintenance on the RPS Channel 1 Reactor Trip Module, the 15 Vdc power supply failed shorting to ground. This caused NNI to produce a high Reactor Coolant System (RCS) pressure signal momentarily opening the Pilot Operated Relief Valve (PORV). The PORV was open for approximately 5 seconds and then closed. RCS pressure decreased approximately 60 psig. Following the PORV closure, Operations placed the pressurizer heaters in manual, entered the pressurizer abnormal operating procedure DB-OP-02513, and verified the PORV closed. RCS pressure and flow inputs to NNI were transferred to RPS Channel 2 and at 1806 the pressurizer heaters were placed back in auto. At 1845 hours, the pressurizer abnormal operating procedure was exited. At 1755 hours on May 19, 1994, reactor power was approximately 99 percent. At approximately 2200 hours on May 19, 1994, reactor power was slowly increased to 100 percent which was achieved at 2230 hours and was maintained at this level until May 20, 1994.

At approximately 2318 hours on May 20, 1994, a manual power reduction to approximately 97 percent power was initiated and achieved by 2345 hours, to perform Moderator Temperature Coefficient (MTC) testing and control rod drive breaker testing. After testing completion, at 1825 hours, reactor power was slowly increased to approximately 100 percent which was achieved at 1918 hours on May 21, 1994. Reactor power was maintained at this level for the rest of the month.