



Commonwealth Edison

1400 Opus Place  
Downers Grove, Illinois 60515

June 10, 1994

Mr. William Russell  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Braidwood Station Unit 1  
Additional Information Regarding  
Technical Specification Amendment for  
Section 3/4.4.5  
NRC Docket No. 50-456

- References:
- 1) D. Saccomando letter to W. Russell transmitting request for Emergency Technical Specification dated April 25, 1994
  - 2) D. Saccomando letter to W. Russell transmitting Additional Information dated May 4, 1994
  - 3) R. Assa letter to D. Farrar transmitting Safety Evaluation for Unit 1 Steam Generator IPC dated May 7, 1994
  - 4) Meeting between Commonwealth Edison Company (ComEd) and the Nuclear Regulatory Commission (NRC) regarding Steam Generator IPC dated June 2, 1994
  - 5) Teleconference between ComEd and NRC regarding Steam Generator IPC Amendment dated June 10, 1994

Dear Mr. Russell,

Reference 1 letter transmitted ComEd's request for an Emergency Technical Specification amendment to Section 3/4.4.5. This amendment request was subsequently granted and transmitted with the Safety Evaluation in Reference 3. On June 2, 1994, ComEd met with the NRC to discuss what actions would be necessary for Braidwood Unit 1 to continue operations beyond the 100 days specified in the Safety Evaluation. During that meeting, the NRC summarized necessary documentation that ComEd would need to submit and requested that ComEd submit a schedule when these items would be docketed. A followup teleconference was held on June 10, 1994, between the Staff and ComEd clarifying the information. The following is our understanding the necessary documentation and the schedule.

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June 10, 1994

- June 13, 1994 Submit finalized WCAP-14046, "Braidwood 1: Technical Support for Cycle 5 S/G Interim Plugging Criteria." This document contains information that will disposition outstanding technical issues, specifically the leakage portion of IPC.
- June 22, 1994 Request to amend Technical Specification Sections 4.4.5.4 and 3.4.8. This amendment request will include a qualitative assessment of the Tube Support Plate Limited Displacement Analysis.

We understand that from the reference teleconference, the following information that was discussed during the June 2nd meeting would not be needed in the pending amendment request:

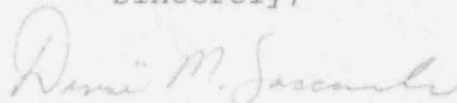
Additional information supportive of the probabilistic risk assessment for multiple steam generator tube rupture event.

A discussion of the differences between core damage frequency due to steam generator tube rupture at Braidwood to Zion or other large tube diameter plants.

A discussion of additional compensatory measures, specifically the use of N-16 monitors at Braidwood. (This issue was previously discussed and transmitted in Reference 2.)

ComEd requests that the Staff review this schedule and the items which will be included in the upcoming amendment request, and notify us immediately if there is any discrepancy as to what is needed in our submittal. Please address any questions to this office.

Sincerely,



Denise M. Saccomando  
Nuclear Licensing Administrator

cc: R. R. Assa, Braidwood Project Manager-NRR  
S. G. Dupont, Senior Resident Inspector-Braidwood  
J. B. Martin, Regional Administrator-Region III  
Office of Nuclear Facility Safety