INU -LEAR REDUCATURE CURRENTS NRC FORM 366 (7.77) LICENSEE EVENT REPORT ļ0 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: (4) 3 0 0 0 0 2 3 (2) R F 01 0 L 0 LICENSE NUMBER LICENSEE CODE CON'T 3 8 0 4 2 0 8 3 (9) REPORT 6 (7) 0 31 2 8 0 1 (6)0 5 0 0 0 0 21 9 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During routine testing by SI 4.7. D. 1. b-1. primary containment isolation valve 0 2 This valve is required to close on a Group 3-FCV-77-2A failed to close. 03 There were no effects isolation signal by Technical Specification 3.7.D.1. 0 4 3-FCV-77-2B upon public health or safety. The redundant isolation valve, 0 5 was operable, and placed in the closed position, 0 6 0 7 0 8 8 0 COMP CAUSE SUBCODE VALVE SYSTEM CAUSE COMPONENT CODE SUBCODE SUBCODE CODE CODE 16 F (12 (13) 0 9 X 18 OCCURRENCE REVISION SEQUENTIAL REPORT CODE TYPE NO. EVENT YEAR REPORT NO. LER'RO (17 REPORT NUMBER COMPONENT SUBMITTED NPRD 4 PRIME COMP. ACTION ACTION EFFEC METHOD (22) MANUFACTURER HOURS FORMOUS SUPPLIER ON PLANT N (24) 01 N (25) (21 y (23) Z 010 9 0 10 V 18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Investigation revealed cil accumulation on the soleroid valve shaft 1 0 3-FCV-77-2B was closed manually and left prevented air leak-off. 1 1 closed until 3-FCV-77-2A (versa VSG-3521 3-way valve) was repaired and returned 1 2 service. These valves will be replaced with a new type valve and the 1 3 oil supply removed during refueling outages. 1 4 8 METHOD OF FACILITY OTHER STATUS (30) DISCOVERY DESCRIPTION (32) S POWER 0 8 1 (31) Surveillance 5 F (28) 5 (29 B ... 9 10 ACTIVITY CONTENT 13 SOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) ELEASED OF RELEASE Z 33 Z 34 6 NA 8 10 44 11 PERSONNEL EXPOSURES DESCRIPTION (39) NU BER 0 (37) 01 8 13 PERSONNEL INJURIES DESCRIPTION (41) NUMBER NA 8304260452 830420 PDR ADOCK 05000296 LOSS OF OR DAMAGE TO FACILITY (43)TYPE DESCRIPTION PDR Z (42) 9 NA 10 PUBLICITY NAC USE CINLY DESCRIPTION (45 SUED_ N (44) 0 68 69 NAME OF PREPAREN D. A. Lochbaum (205) 729-084 PHONE ..

Tennessee Valley Authority Browns Ferry Nuclear Plant

Form BF 17 BF 15.2 2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50-296/ 83022Technical Specification Involved3.7.D.1Reported Under Technical Specification 6.7.2.b(2)* Date Due NRC4-21-83

Event Narrative:

Unit 1 was at 87-percent, unit 2 was at 14 percent, and unit 3 was at 83-percent power. Only unit 3 was affected by this event.

SI 4.7.D.1.b-1 (Primary Containment Isolation Valve Operability Test) was performed on unit 3 which indicated 3-FCV-77-2A incapable of closing. This normally open valve is required to close upon a Group 2 isolation signal by Technical Specification 3.7.D.1. The redundant isolation valve on this line, 3-FCV-77-2B, was immediately closed as required by technical specifications. MR A129202 was written to have 3-FCV-77-2A repaired. Maintenance activities revealed oil accumulation on the solenoid valve shaft prevented air leak-off from the actuator. 3-FCV-77-2A was repaired and returned to service. These valves will be replaced and the in-line oil supply removed during refueling outages.

* Previous Similar Events:

BFRO-50-259/82040, 73051W, 82044 BFRO-50-296/77017, 83014

Retention: Period - Lifetime; Responsibility - Document Control Supervisor *Revision: