

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

400 Chestnut Street Tower II

April 18, 1983

HTRD-50-518/83-06, -520/83-06
YCRD-50-566/83-03, -567/83-03

U.S. Nuclear Regulatory Commission
Region II
ATTN: James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

HARTSVILLE A AND YELLOW CREEK NUCLEAR PLANTS - REPORTABLE DEFICIENCY -
PIPE SUPPORT SPACING BY ALTERNATE CRITERIA - HTRD-50-518/83-06,
-520/83-06 - YCRD-50-566/83-03, -567/83-03

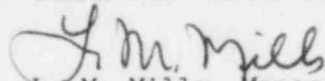
The subject deficiency was initially reported to NRC-OIE, Region II, Inspector
Floyd Cantrell on March 17, 1983, as NRCs HTADNP8302 and YCNDNP8302. In
compliance with paragraph 50.55(e) of 10 CFR Part 50, we are enclosing our
first interim report on the subject deficiency.

TVA has elected to defer construction activities for these projects. Deferral
does not mean that these projects will be cancelled but that TVA is minimizing
its expenditures and construction efforts until such time that TVA has
sufficient information to indicate whether these projects should be completed
or cancelled. Therefore, we will not be submitting a final report for this
nonconformance until a final decision is made regarding these projects.

If you have any questions, please call Jim Domer at FTS 858-2725 for
Hartsville and Ralph Shell at FTS 858-2676 for Yellow Creek.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


L. M. Mills, Manager
Nuclear Licensing

Enclosure
cc: See page 2

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Mr. James P. O'Reilly

April 18, 1983

cc (Enclosure):

Mr. R. C. DeYoung, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

ENCLOSURE
HARTSVILLE A AND YELLOW CREEK NUCLEAR PLANTS
PIPE SUPPORT SPACING BY ALTERNATE CRITERIA
10CFR50.55(e) REPORT NO. 1 (INTERIM)
NCR HTADNP8302 AND YCNDNP8302

Description of Deficiency

Hartsville A

The determination of support spacing for schedule 10S steel piping, sized from above two inches diameter to six inches may be unconservative if the design has been by alternate criteria. The design of pipe support spacing for schedule 10S piping two inches diameter and less is controlled by the C. F. Braun Technical Report (TR-2204-1), "Smaller Piping Design Guide for Size 2 Inches and Smaller." It cannot be readily verified that the alternate criteria design of schedule 10S pipe support spacing is adequate for the thin wall thickness in sizes above two inches to six inches diameter. Erroneous design use of schedule 40 pipe thickness in lieu of schedule 10S may have occurred.

Yellow Creek

The determination of support spacing for schedule 10S stainless steel piping six inches in diameter and less may be unconservative if the design has been by alternate criteria. The scope of the Yellow Creek Nuclear Plant's Alternate Criteria for Piping Analysis and Support (CEB-ME1-78-2) does not provide for the use of schedule 10S stainless steel piping. It cannot be readily verified that the alternate criteria design of schedule 10S pipe support spacing is adequate for the thin wall thickness of 10S pipe. Erroneous design use of 40 pipe thickness in lieu of schedule 10S may have occurred.

Interim Progress

Because of the deferral of the Hartsville A and Yellow Creek Nuclear Plants, investigation into the item will be postponed until the time that design and construction is resumed.