Nebraska Public Power District

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LQA8200197

October 19, 1982

50-298

Office of Nuclear Reactor Regulation
Attention: Mr. Domenic B. Vassallo, Chief
Operating Reactors Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Revision to In-Service Testing Relief Requests

Reference: 1) Letter from J. M. Pilant to D. B. Vassallo dated March 29, 1982, "In-Service Inspection/ Testing Relief Requests"

Dear Mr. Vassallo:

Our letter of reference 1 submitted eleven relief requests relating to our In-Service Inspection Program at Cooper Nuclear Station and four relief requests relating to in-service testing. As a result of discussions with the Staff, October 13, 1982, the District wishes to modify our relief requests no. RP-02 and RP-04 as indicated on the attached replacement pages.

Should the Staff require additional information relating to any of these revisions, please contact me.

Sincerely,

Jay M. Pilant
Division Manager of Licensing
and Quality Assurance

JMP:JDW:cmk

Enclosure

A047

GENERAL ELECTRIC

INSTALLATION & SERVICE ENGINEERING DIVISION

INSERVICE TESTING PROGRAM

COOPER NUCLEAR STATION

RELIEF REQUESTS FOR PUMP & VALVE TESTING

Relief Request No. RP-02

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Revision No. 1

PUMP OR VALVE NO. 00	ASME CODE CLASS 01	ASME CATEGORY 02	FUNCTION 03	SECTION XI TEST REQUIREMENT 04	BASIS FOR RELIEF 05	ALTERNATIVE TEST 06
SLC-1A SLC-1B	2	--		Measure Pump Inlet pressure and Pump differential pressure.	It is impractical to measure standby liquid control pump inlet pressure (thus making pump differential pressure impractical) in accordance with Section XI requirements. During pump testing, the pump suction is from a test tank rather than the main standby liquid control tank. The only means available to measure inlet pressure is to correlate tank level to inlet pressure. These pumps are positive displacement, the measurement of inlet pressure is not critical in judging pump performance. Measuring the discharge pressure and the flow rate is adequate to detect changes in the hydraulic characteristics of the pumps.	Monitor pump discharge at each inservice test.

GENERAL ELECTRIC

INSTALLATION & SERVICE ENGINEERING DIVISION

INSERVICE TESTING PROGRAM

COOPER NUCLEAR STATION

RELIEF REQUESTS FOR PUMP & VALVE TESTING

Relief Request No. RP-04

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Revision No. 1

PUMP OR VALVE NO. 00	ASME CODE CLASS 01	ASME CATEGORY 02	FUNCTION 03	SECTION XI TEST REQUIREMENT 04	BASIS FOR RELIEF 05	ALTERNATIVE TEST 06
All Pumps	--	--	N/A	Requirements of Table IWP-3100-2	Portable vibration monitoring equipment allowing for the measurement of vibration velocity in inches/second is available at the plant site. This measurement gives a more accurate determination of abnormal vibrations at frequencies other than shaft rotational speed.	The vibration testing data will be analyzed in accordance with the percent change of the established reference value. The allowable percent changes are shown on Table 1 on page 2.

TABLE 1

ALLOWABLE RANGES OF VIBRATION QUANTITIES

V Reference	Acceptable	Alert	Required Action
0-.1 in/sec	0-2Vr (100%)*	>2Vr-4Vr (300%)	>4Vr (>300%)
>.1-.2 in/sec	0-1.75Vr (75%)	>1.75Vr-2.5Vr (150%)	>2.5Vr (>150%)
>.2-.3 in/sec	0-1.25Vr (25%)	>1.25Vr-2.0Vr (100%)	>2.0Vr (>100%)
>.3 in/sec	0-1.10Vr (10%)	>1.1Vr-1.75Vr (75%)	1.75Vr (>75%)

*All percentages shown in parentheses are the maximum allowable percents over the original reference value for that particular range.