



Tech. Spec. 6.9.A.2

BOSTON EDISON

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

E. T. Boulette, PhD
Senior Vice President - Nuclear

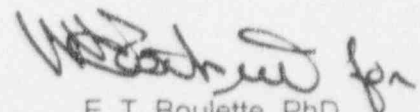
June 14, 1994
BECo Ltr. #94-070

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Docket No. 50-293
License No. DPR-35

MAY 1994 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


E. T. Boulette, PhD

WJM/lam/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
DATE: 6/14/94
COMPLETED BY: W. Munro
TELEPHONE: (508) 830-8474

MONTH May 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	78	17	660
2	445	18	661
3	659	19	661
4	664	20	660
5	664	21	663
6	661	22	662
7	664	23	661
8	663	24	661
9	663	25	661
10	664	26	661
11	663	27	661
12	664	28	662
13	643	29	661
14	595	30	661
15	661	31	661
16	662		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE 6/14/94
 COMPLETED BY: W. Munro
 TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

1. Unit Name Pilgrim I
2. Reporting Period May 1994
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744.0</u>	<u>3623.0</u>	<u>188255.0</u>
12. Hours Reactor Critical	<u>744.0</u>	<u>3330.0</u>	<u>116271.5</u>
13. Hours Reactor Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>733.6</u>	<u>3210.8</u>	<u>111999.8</u>
15. Hours Unit Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1428144.0</u>	<u>6033432.0</u>	<u>197178168.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>489440.0</u>	<u>2069250.0</u>	<u>66715184.0</u>
18. Net Electrical Energy Generated (MWH)	<u>471131.0</u>	<u>1991313.0</u>	<u>64125007.0</u>
19. Unit Service Factor	<u>98.6</u>	<u>88.6</u>	<u>59.5</u>
20. Unit Availability Factor	<u>98.6</u>	<u>88.6</u>	<u>59.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>94.5</u>	<u>82.0</u>	<u>50.8</u>
22. Unit Capacity Factor (Using DER Net)	<u>96.7</u>	<u>83.9</u>	<u>52.0</u>
23. Unit Forced Outage Rate	<u>1.4</u>	<u>5.6</u>	<u>11.7</u>
24. Shutdowns scheduled over next 6 months (type, date, and duration of each) - MCO-10 October (30 Days)			
25. If shutdown at end of report period, estimated date of startup - UNIT OPERATING			

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR MAY 1994

The unit started the reporting period with the reactor at approximately 18 percent core thermal power (CTP), the mode switch in RUN, and the turbine generator off-line to facilitate repair of a steam leak on the Train 'B' second point heater. Following repair of the steam leak, the unit was synchronized to the grid at 1026 hours on May 1, 1994. Reactor power was increased and at 1918 hours on May 2, 1994, the unit was at approximately 100 percent CTP. This power level was essentially maintained until May 13, 1994, when power was reduced to approximately 75 percent CTP to facilitate a rod pattern adjustment, and perform maintenance on control rods 46-43 and 38-43 solenoid valves and filters. Reactor power was returned to approximately 100 percent CTP on May 14, 1994, at 1976 hours, and was essentially maintained throughout the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES
MONTH OF MAY 1994

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: April 1, 1995
3. Scheduled date for restart following next refueling: May 26, 1995
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1629 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
 - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 691 fuel assemblies.

MONTH MAY 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Salt Service Water (SSW) System	SSW Pump P-208A	Pump trending down in discharge head. Pump in alert.	Under Investigation	Pump overhauled, disassembled and inspected. Installed rebuilt suction bowl assembly; seven new spider bearings; two line shafts; one head shaft; one stuffing box bearing; packing and new gaskets. Also replaced upper and lower motor bearings. Tested successfully.	Investigating new pump design (spec. M-8.B)	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

DOCKET NO: 50-293
 NAME: _____
 DATE: 6/14/94
 COMPLETED BY: W. Munro
 TELEPHONE: (508) 830-8474
 REPORT MONTH: May 1994

NO	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
6	5/01/94	F	10.4	B	-	N/A	N/A	N/A	Continuation of off line maintenance to repair steam leak on the Train "B" second point heater drain.
1		2			2	3		4&5	
F-Forced S-Sched		A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination		F-Admin G-Oper Error H-Other		1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other		Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)	