7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
0 1	LICENSEE CODE ②	V A S P S 1
	LICENSE NUMBER ③	0 0 - 0 0 0 0 0 - 0 0
	LICENSE TYPE ④	4 1 1 1 1
	CAT ⑤	
0 1 (CON'T)	REPORT SOURCE ⑥	L
	DOCKET NUMBER ⑦	0 5 0 0 0 2 8 0
	EVENT DATE ⑧	0 3 1 9 8 3
	REPORT DATE ⑨	0 4 1 8 8 3

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 With Unit 1 at cold shutdown, it was discovered that 1-CC-242, a containment isolation
0 3 valve, had been moved approximately 10 inches without performing a seismic analysis.
0 4 This is reportable per T.S.-6.6.2.b.(3). Valve leakage and pressure tests were
0 5 performed. A recent seismic re-analysis indicates that the redesigning of seismic
0 6 supports is not necessary. Therefore, the health and safety of the public were not
0 7 affected.
0 8

Line	Field	Value
0 9	SYSTEM CODE ⑪	S D
	CAUSE CODE ⑫	D
	CAUSE SUBCODE ⑬	Z
	COMPONENT CODE ⑭	Z Z Z Z Z Z
	COMP. SUBCODE ⑮	Z
	VALVE SUBCODE ⑯	Z
⑰ LER/RO REPORT NUMBER	EVENT YEAR	8 3
		—
	SEQUENTIAL REPORT NO.	0 1 7
		/
	OCCURRENCE CODE	0 3
	REPORT TYPE	L
		—
	REVISION NO.	0
	ACTION TAKEN ⑱	G
	FUTURE ACTION ⑲	Z
	EFFECT ON PLANT ⑳	Z
	SHUTDOWN METHOD ㉑	Z
	HOURS ㉒	0 0 0 0
	ATTACHMENT SUBMITTED ㉓	Y
	NPRD-4 FORM SUB. ㉔	N
	PRIME COMP. SUPPLIER ㉕	A
	COMPONENT MANUFACTURER ㉖	Z Z Z Z

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 The cause was due to defective procedures. Specifically, performing a modification
1 1 without a seismic re-analysis. Since this event, formal inservice inspection repair
1 2 program has been implemented.
1 3
1 4

Line	Field	Value
1 5	FACILITY STATUS ㉘	H
	% POWER ㉙	0 0 0
	OTHER STATUS ㉚	N/A
	METHOD OF DISCOVERY ㉛	A
	DISCOVERY DESCRIPTION ㉜	Engineering Evaluation
1 6	ACTIVITY RELEASED ㉝	Z
	CONTENT OF RELEASE ㉞	Z
	AMOUNT OF ACTIVITY ㉟	N/A
	LOCATION OF RELEASE ㊱	N/A
1 7	PERSONNEL EXPOSURES NUMBER ㊲	0 0 0
	TYPE ㊳	Z
	DESCRIPTION ㊴	N/A
1 8	PERSONNEL INJURIES NUMBER ㊵	0 0 0
	DESCRIPTION ㊶	N/A
1 9	LOSS OF OR DAMAGE TO FACILITY TYPE ㊷	Z
	DESCRIPTION ㊸	N/A
2 0	PUBLICITY ISSUED ㊹	N
	DESCRIPTION ㊺	N/A
	NRC USE ONLY	

8304260126 830418
PDR ADOCK 05000280
S PDR

NAME OF PREPARER J. L. Wilson PHONE: (804) 357-3184

ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 1
DOCKET NO: 50-280
REPORT NO: 83-017/03L-0
EVENT DATE: 03-19-83

TITLE OF THE EVENT: 1-CC-242 MOVED WITHOUT DOCUMENTATION OR REVIEW

1. Description of the Event

With the unit at cold shutdown, it was found that containment isolation valve 1-CC-242 had been moved about 10 inches in July 1981 without a seismic evaluation. The valve was moved to facilitate maintenance while correcting a valve leakage problem. This was discovered during a current engineering evaluation for the relocation of the valve. This event is reportable per T.S.-6.6.2.b.(3).

2. Probable Consequences and Status of Redundant Equipment

Check valve 1-CC-242 is a containment isolation valve. It was satisfactorily leak tested after the initial movement of the valve. Seismic analysis of the current location of the valve indicates the redesigning of seismic supports is not necessary therefore, the health and safety of the public were unaffected.

3. Cause

The cause for this event was an inadequate repair procedure. There was no procedural requirement to perform a seismic evaluation.

4. Immediate Corrective Action

With the unit shutdown and defueled, no immediate action was necessary.

5. Subsequent Corrective Action

Since this event, an inservice inspection repair program has been implemented. This program requires engineering evaluation for the repair or replacement of any safety related component.

6. Action Taken to Prevent Recurrence

The current inservice inspection program for repair and replacement is designed to prevent a recurrence of this event.

7. Generic Implications

None.