

ATTACHMENT

Quad Cities Station Unit 1

Proposed Changes to Appendix A

Technical Specifications to Operating License DPR-29

Revised pages: 1.2/2.2-1*
3.6/4.6-4

* This page includes changes proposed in the T. Rausch letter to H. Denton dated August 19, 1982 (50.59 reloads with ODYN) as supplemented in the T. Rausch to H. Denton letter dated October 18, 1982.

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1.2/2.2 REACTOR COOLANT SYSTEM

SAFETY LIMIT

Applicability:

Applies to limits on reactor coolant system pressure.

Objective:

To establish a limit below which the integrity of the reactor coolant system is not threatened due to an overpressure condition.

LIMITING SAFETY SYSTEM SETTING:

Applicability:

Applies to trip settings of the instruments and devices which are provided to prevent the reactor system safety limits from being exceeded.

Objective:

To define the level of the process variables at which automatic protective action is initiated to prevent the safety limits from being exceeded.

SPECIFICATIONS

A. The reactor coolant system pressure as measured by the vessel steam space pressure indicator shall not exceed 1345 psig at any time when irradiated fuel is present in the reactor vessel.

A. Reactor coolant high-pressure scram shall be ≤ 1060 psig.

B. Primary system safety valve nominal settings shall be as follows:

- 1 valve at 1135 psig⁽¹⁾
- 2 valves at 1240 psig
- 2 valves at 1250 psig
- 4 valves at 1260 psig

⁽¹⁾Target Rock combination safety/relief valve

The allowable setpoint error for each valve shall be $\pm 1\%$.

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2. Both the sump and air sampling systems shall be operable during reactor power operation. From and after the date that one of these systems is made or found to be inoperable for any reason, reactor power operation is permissible only during the succeeding 7 days.
3. If the conditions in 1 or 2 above cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.

E. Safety and Relief Valves

1. Prior to reactor startup for power operation, during reactor power operating conditions, and whenever the reactor coolant pressure is greater than 90 psig and temperature greater than 320° F, all nine of the safety valves shall be operable. The solenoid-activated pressure valves shall be operable as required by Specification 3.5.D.
2. If Specification 3.6.E.1 is not met, the reactor shall remain shut down until the condition is corrected or, if in operation, an orderly shutdown shall be initiated and the reactor coolant pressure and temperature shall be below 90 psig and 320° F within 24 hours.

F. Structural Integrity

The structural integrity of the primary system boundary shall be maintained at the level required by the ASME Boiler and Pressure Vessel Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components", 1974 Edition, Summer 1975 Addenda (ASME Code Section XI).

E. Safety and Relief Valves

A minimum of 1/2 of all safety valves shall be bench checked or replaced with a bench checked valve each refueling outage. The popping point of the safety valves shall be set as follows:

<i>Number of Valves</i>	<i>Setpoint (psig)</i>
1	1135 ⁽¹⁾
2	1240
2	1250
4	1260

The allowable setpoint error for each valve is $\pm 1\%$.

All relief valves shall be checked for set pressure each refueling outage. The set pressures shall be:

<i>Number of Valves</i>	<i>Setpoint (psig)</i>
1	$\leq 1135^{(1)}$
2	≤ 1115
2	≤ 1135

⁽¹⁾Target Rock combination safety/relief valve.

F. Structural Integrity

The nondestructive inspections listed in Table 4.6-1 shall be performed as specified in accordance with Section XI of the ASME Boiler and Pressure Vessel Code, 1971 Edition, Summer 1971 Addenda. The results obtained from compliance with this specification will be evaluated after 5 years and the conclusions will be reviewed with the NRC.