

Proposed Technical Specifications
for the General Electric Test Reactor (GETR)

I. Proposed Technical Specification Changes

A. Change Technical Specification 3.2 to read as follows:

- 3.2 Containment integrity shall be maintained whenever the reactor is not secured. Even when the reactor is secured, the time during which containment integrity is not maintained shall be kept to a minimum.

During the extended outage which began on October 27, 1977, and while there is no irradiated fuel or irradiated fueled experiments in the containment building, the reactor is exempt from the requirement to keep the time during which containment integrity is not maintained to a minimum.

B. Add the following paragraphs to Technical Specification 11.0, "System Tests and Calibrations":

- 11.1 j. Leak rate test of the containment building per Technical Specification 3.4 unless containment integrity is required.
- k. Leak detection test of the ventilation valves and airlock penetrations per Technical Specification 3.5 unless containment integrity is required.
- l. Functional test of the building pressure detector per Technical Specification 3.6 unless containment integrity is required.
- m. Functional test of the radiation monitor per Technical Specification 3.6 unless containment integrity is required.
- n. Operability test of the building vacuum relief system per Technical Specification 3.8 unless containment integrity is required.

II. Safety Analysis

Prior to implementation of these Technical Specification changes, all irradiated reactor fuel and irradiated fueled experiments will have been removed and shipped from the General Electric Test Reactor containment building. (Note: All unirradiated SNM except for check sources has also been removed.) Consequently, there is no need for containment building integrity as defined in Technical Specification 1.5 to License TR-1 or for containment building integrity surveillance.

- A. Without irradiated reactor fuel or fueled experiments, there are no sources of radioactivity inside the containment building which could result in airborne concentrations or exposures exceeding 10CFR20 limits in unrestricted areas. All systems used for the capture and retention of radioactive gases are depressurized, and there is no storage of radioactive gases. The remaining radioactivity in the form of mildly contaminated primary cooling water, activated hardware, check and calibration sources, etc., are judged to be materials not at risk and do not require containment integrity as defined in Technical Specification 1.5 for safe storage and handling. Consequently, proposed Technical Specification 3.2 permits the facility to be in a condition without containment integrity.

In fact, two deactivated reactor facilities at the Vallecitos Nuclear Center (VNC), the EVESR (License DR-10, Docket 50-183) and the VBWR (License DPR-1, Docket 50-18), have similar residual radioactive materials, and containment integrity is not required.

- B. The proposed addition to Technical Specification 3.2 permits the postponement of tests for equipment that are not required as long as containment integrity is not required. The proposed additions to Technical Specification 11.1 postpone these unnecessary tests. Technical Specification 11.2, as modified by our application of February, 1992, of course, requires satisfactory completion of all postponed tests prior to returning fuel or experiments to the reactor or the pool.