Docket No. 50-155 LS05-82-09-089

> Mr. David J. VandeWalle Nuclear Licensing Administrator Consumers Power Company 1945 West Parnall Road Jackson, Michigan 49201

Dear Mr. VandeWalle:

SUBJECT: SEP TOPIC III-7.B, DESIGN CODES, DESIGN CRITERIA AND LOAD COMBINATIONS - BIG ROCK POINT

Enclosed is a copy of our draft evaluation of SEP Topic III-7.B. The evaluation identifies areas of codes where changes have occurred to decrease safety margins. It also identifies loads applicable to some or all of the structures at Big Rock Point which have increased in magnitude. After reviewing structural drawings of your facility, we concluded that some code changes of concern were not applicable to your facility because the structural elements to which these code changes are referring were not found in the structural drawings of Big Rock Point which we reviewed. These changes are identified in Appendix A of the enclosure. The evaluation also concludes that, based on analyses performed on a similar containment, it may be possible to overstress the shell under current loading combinations considering the new loads developed in other SEP topics. You are to review how these areas of the codes were applied in the design of Big Rock Point and the ability of structures to resist increased loads and assess the current safety margins.

You are requested to examine the facts upon which the staff has based its evaluation and respond by confirming that the facts are correct or by identifying errors and supplying the corrected information. We encourage you to supply any other material that might affect the staff's evaluation of this topic or be significant in the integrated assessment of your facility.

You are requested to respond to the factual correctness of the SER and propose a schedule for resolution of the open items within 30 days of receipt of this letter.

SE04 Add: Ray Schall

DSU USE EX(18)

schfield, Chi-8210250091 820930 Sincerely, PDR ADOCK 050001 Dennis M. Crutchfield, Chief utchfield Operating Reactors Branch #5 SEPB:DL Division of Licensing ORB#5:PM SEPB: Dky/ OFFICE ! WRussell RHermann 9/17/82 ..9/17/82. 9 /2 /82

Enclosure: As stated

cc w/enclosure: See next page

OFFICE)	*****************	 Tereston and the second	*****************	*****************	******	***************************************

cc Mr. Paul A. Perry, Secretary Consumers Power Company 212 West Michigan Avenue Jackson, Michigan 49201

Judd L. Bacon, Esquire Consumers Power Company 212 West Michigan Avenue Jackson, Michigan 49201

Joseph Gallo, Esquire Isham, Lincoln & Beale 1120 Connecticut Avenue Room 325 Washington, D. C. 20036

Peter W. Steketee, Esquire 505 Peoples Building Grand Rapids, Michigan 49503

Alan S. Rosenthal, Esq., Chairman Atomic Safety & Licensing Appeal Board U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Mr. John O'Neill, II Route 2, Box 44 Maple City, Michigan 49664

Mr. Jim E. Mills Route 2, Box 108C Charlevoix, Michigan 49720

> Chairman County Board of Supervisors Charlevoix County Charlevoix, Michigan 49720

Office of the Governor (2) Room 1 - Capitol Building Lansing, Michigan 48913

Herbert Semmel
Counsel for Christa Maria, et al.
Urban Law Institute
Antioch School of Law
2633 16th Street, NW
Washington, D. C. 20460

U. S. Environmental Protection Agency Federal Activities Branch Region V Office ATTN: Regional Radiation Representative 230 South Dearborn Street Chicago, Illinois 60604

Peter B. Bloch, Chairman Atomic Safety and Licensing Board U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Dr. Oscar H. Paris Atomic Safety and Licensing Board U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Mr. Frederick J. Shon Atomic Safety and Licensing Board U. S. Nüclear Regulatory Commission Washington, D. C. 20555

ATTN: Mr. C. J. Hartman
Plant Superintendent
Charlevoix, Michigan 49720

Christa-Maria Route 2, Box 108C Charlevoix, Michigan 49720

William J. Scanlon, Esquire 2034 Pauline Boulevard Ann Arbor, Michigan 48103

Resident Inspector
Big Rock Point Plant
c/o U.S. NRC
RR #3, Box 600
Charlevoix, Michigan 49720

Hurst & Hanson 311 1/2 E. Mitchell Petoskey, Michigan 49770 Mr. David J. VandeWalle

cc Dr. John H. Buck Atomic Safety and Licensing Appeal Board U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Ms. JoAnn Bier 204 Clinton Street Charlevoix, Michigan 49720

Thomas S. Moore Atomic Safety and Licensing Appeal Board U. S. Nuclear Regulatory Commission Washington, D. C. 20555

James G. Keppler, Regional Administrator Nuclear Regulatory Commission, Region III 799 Roosevelt Road Glen Ellyn, Illinois 60137

SYSTEMATIC EVALUATION PROGRAM

TOPIC III-7.B

BIG ROCK POINT

TOPIC: III-7.8, DESIGN CODES, DESIGN CRITERIA AND LOAD COMBINATIONS

INTRODUCTION

SEP plants were generally designed and constructed during the time span from the late 1950's to late 1960's. They were designed according to criteria and codes which differ from those accepted by the NRC for new plants.

The purpose of this topic is to assess the safety margins existing in Category I structures as a result of changes in design codes and criteria.

II. REVIEW GUIDELINES

The current licensing criteria which governs the safety issue in this topic is 10 CFR 50, Appendix A, GDC 1, 2, and 4 as interpreted by Standard Review Plan 3.8.

III. RELATED SAFETY TOPICS

The following SEP topics are related to III-7.8:

- 1. III-2, Wind and Tornado Loadings
- 2. III-3.A. Effects of High Water Level on Structures
- 3. III-4.A, Tornado Missiles
- 4. III-5.A, Effects of High Energy Pipe Breaks Inside Containment
- 5. III-5.B, Effects of High Energy Pipe Breaks Outside Containment
- 6. III-6. Seismic Design Considerations
- VI-2.D, Mass and Energy Release for Postulated Pipe Break Inside Containment
- 8. VI-3, Containment Pressure and Heat Removal Capability

IV. EVALUATION

The evaluation is based on a Technical Evaluation Report (TER) prepared by the Franklin Research Center (FRC) in conjunction with the NRC staff through contract. The report is entitled, "Design Codes, Design Criteria and Loading Combinations" and is attached to this Safety Evaluation Report as Enclosure (1).

We have compared structural design codes employed in the design of Category I structures at Big Rock Point to present codes. This was done through generic code versus code comparison without investigating specifically how the original code was applied to the Big Rock Point design; however, after reviewing drawings of structures at Big Rock

Point we concluded that certain portions of the codes were not applicable to Big Rock Point because the types of structures to which the codes are referring were non-existent at Big Rock Point. We have compared the loads and loading combinations employed in the design of Big Rock Point.

A result of these comparisons is that a number of code changes could potentially impact significantly margins of safety (denoted by scale A and Ax in Enclosure 1). This can be attributed to several factors such as :

- 1. New codes have imposed stricter limitations than old,
- New codes have included sections governing design of certain types of structures which were not included in the older codes,
- Design loads required today were not included in the plant design, and
- Certain load combinations judged to be significant were not included in plant design.

In Enclosure (1), some items have been judged to potentially impact margins of safety regarding the containment as a result of comparing ASME B&PV Section VIII, 1962 to ASME BPV, Section III, Subsection NE, 1980.

The code changes of concern from Enclosure (1) are:

	Examined	New Code	Old Code
Com	posite Construction	AISC 1980	AISC 1953
1.	Shear connectors in composite beams	1.11.4	13
2.	Composite beams or girders with formed steel deck	1.11.5	-
3.	Width of concrete flange - limitations	1.11.1	13 (a)
Com	pression Elements	AISC 1980	AISC 1953
1.	With width-to-thickness ratio higher than speci- fied in 1.9.1.2	1.9.1.2 and Appendix C	18 (b)
2.	Members where sideway is not prevented	1.8.3	16
Ten	usion Members	AISC 1980	AISC 1953
1.	When load is transmitted by bolts or rivets	1.14.2.2	-
2.	Built up members	1.18.3	28 (b)
Con	nections	AISC 1980	AISC 1953
1.	Beam ends with top flange coped, if subject to shear	1.5.1.2.2	_
2.	Connections carrying moment or restrained member connection	1.15.5.2 1.15.5.3 1.15.5.4	-

^{*}Double dash (--) indicates that older code had no provisions.

Examined	Hew Code	Old Code	
Members Designed to Operate in an Inelastic Regime	AISC 1980	AISC 1953	
Spacing of lateral bracing	2.9	_	
Rolled Sections and	AISC 1980	AISC 1953	
Built up Members	1.5.1.4.1	15 (a) (3)	
Partial length cover plates	1.10.4	26 (d)	
Members Subject to Axial	AISC 1980	AISC 1953	
and Bending Stresses	1.6	12 (a)	
Web Plate Girders	AISC 1980	AISC 1953	
 Subject to shear and tension stresses 	1.10.7	-	
2. Stiffeners	1.10.10.2	26	
Partial Penetration Weld			
Effective throat thickness	1.14.6.1	15 (f)	
Short Brackets and Corbels	ACI 349-76	ACI 318-56	
having a shear span-to- depth ratio of unity or less	11.13	-	
Shear Walls used as a	ACI 349-76	ACI 318-56	
primary load-carrying member	11.16	-	
Precast Concrete Structural	ACI 349-76	ACI 318-56	
Elements, where shear is not a measure of diagonal tension			
Concrete Regions Subject to High Temperatures	ACI 349-76	ACI 318-56	
Time-dependent and position-dependent	Appendix A	-	
temperature variations			

	-5-		
Structural Elements to b	e Code	Change Affect	ing These Elements
Examined		New Code	Old Code
All Structural Elements		ACI 349-76	ACI 318-56
1. Ultimate bond streng	th	Chapter 12	-
2. Allowable bond stress		-	Table 305(a)
Columns with Spliced Reinforcement		ACI 349-76	ACI 318-56
subject to stress reverse fy in compression to 1/2 fy in tension	als;	7.10.3	-
Steel Embedments used to transmit load to concrete		ACI 349-76 Appendix B	ACI 318-56
Element Subject to Impulsive and Impactive I whose failure must be pre	oads .	ACI 349-76 Appendix C	ACI 318-56
Composite Construction		ACI 349-76 Chapter 17	ACI 318-56
Containment Vessels			
1. Plates, if understren		ASME Sec. III, 1980 ME-3112.4	ASME Sec. VIII, 1956 DG-5(b)
 Containment vessels of materials no longer listed as code acceptable 		ASME Sec. III, 1980 WE-3112.4	ASME Sec. VIII, 1956 DG-23
 Containment vessels designed by formula a subject to substantia thermal or mechanical 	nd 1	ASME Sec. III, 1980 WE-3131	ASME Sec. VIII, 1956 Various paragraphs
 Stiffening rings for cylindrical shells subject to buckling 1 	1	LSME Sec. III, 1980 (E-3133.5(a)	ASME Sec. VIII, 1956 UG-29

_	Examined	New Code	ing These Blements Old Code
5.	Stiffening rings of material different than shell material	ASME Sec. III, 1980 NE-3133.5(b)	ASME Sec. VIII, 1956
6.	Vessels with Quick Actuating Closures	ASME Sec. III, 1980 NE-3327.1	ASME Sec. VIII, 1956 Pootnote to DG-35
Sh	ell Openings and Attachments		
1.	Onstayed flat heads and covers	ASMR Sec. III, 1980 NE-3325 Pigs. (c) and (m)	1956 DG-34(d)
2.	Openings and reinforcements; subject to cyclic loads	ASME Sec. III, 1980 NE-3331(b)	ASME Sec. VIII, 1956
3.	Reinforcement for openings	ASME Sec. III, 1980 NE-3334.1, NE-3334.2	ASME Sec. VIII, 1956 DG-40
	Bellows and bellows expansion joints	ASME Sec. III, 1980 NE-3365	ASME Sec. VIII, 1956
	Roofs		

Extreme environmental snow loads are provided by SEP Topic II-2.A. NRC Regulatory Guide 1.102 (Position 3) provides guidance to preclude adverse consequences from ponding or parapet roofs. Failure of roofs not designed for such circumstances could generate impulsive loadings and water damage, possibly extending to Seismic Category I components of all floor levels.

^{1.} Not shown in tabular summary of code change impacts.

Section 10 of Enclosure (1) addresses load and load combination changes which occurred as a result of code changes and identifies specific plant structures for which various load combinations may be significant. Based upon a lack of detailed information on the stress results for loads and load combinations used during design of structures at Big Rock Point, these loads and load combinations may be potentially significant.

Based on an analysis of a similar containment (San Onofre 1) it may be possible to overstress the shell at grade elevation when considering the new loads developed in other SEP Topics (VI-2.D, VI-3 and III-6) in current loading combinations.

V. CONCLUSIONS

We concluded that after comparing design codes, criteria, loads and load combinations, a number of changes have occurred which could potentially impact margins of safety. These changes are identified above. These differences between plant design and current licensing criteria should be resolved as follows:

- Review Seismic Category I Structures at Big Rock Point to determine if any of the structural elements for which a concern exists are a part of the facility design of Big Rock Point. For those that are, assess the impact of the code changes on margins of safety on a plant specific basis.
- 2. Examine on a sampling basis the margins of safety of Seismic Category I Structures for loads and load combinations not covered by another SEP topic and denoted by Ax in Enclosure (1). (The load tables should be reviewed to assure their technical accuracy concerning applicability of the loads for each of the structures and their significance. The Category I structures considered should be reviewed to insure completeness.)

The licensee should determine the ability of the containment to withstand newly developed loads in current loading combinations.