



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated October 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

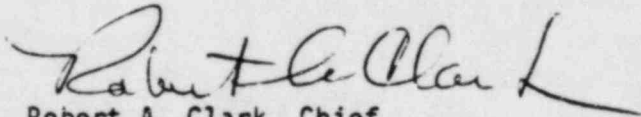
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 70, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1983



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated October 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I:
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

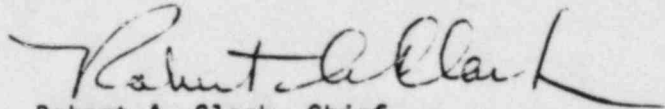
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages

15.4.4-4
15.4.4-5
15.4.4-6

Insert Pages

15.4.4-4
15.4.4-5
15.4.4-6

acceptance criteria in Section 1.B above, a Type "A" test shall be performed at each plant shutdown for refueling, or approximately every 18 months, whichever occurs first. The accelerated test schedule shall continue until two consecutive Type "A" tests pass, after which time the retest schedule in I.C.1 may be resumed.

D. Report of Test Results

1. Each Type "A" leakage rate test will be the subject of a summary technical report, which will include summaries of Type "B" and "C" tests (Items II and III below) that were performed since the last Type "A" test.

II. Type "B" Tests

A Type "B" test measures leakage across individual and/or portions of pressure containing or leakage-limiting boundaries of primary reactor containment penetrations as defined in II.A.5.

A. Test

1. Type "B" tests shall be performed at intervals specified in II.C. below.
2. With the exception of the airlock door seal test, for the purposes of the 3 day test requirement, Type "B" tests shall be performed at a pressure of not less than P_a .
3. Testing of the airlock door seals, in lieu of the full pressure airlock test, may be used to fulfill the 3 day airlock testing requirement specified in II.C.1.d below. This airlock door seal test shall be performed with a pressure differential across the door seals of at least 10" of mercury. This pressure differential may be established via the use of a positive pressure or a vacuum. Airlock door seal testing shall not be substituted for the 6 month test of the entire air lock.
4. Acceptable methods of testing are halogen leak detection, pressure decay and fluid flow using air or nitrogen. Another method may be used if it can be shown to have equivalent sensitivity.
5. The local leakage shall be measured for each of the following components:
 - a. Containment penetrations that employ resilient seals, gaskets or sealant compounds, piping penetrations fitted with expansion bellows and electrical penetrations fitted with flexible metal seal assemblies.

- b. Airlock and equipment door seals, including operating mechanism and penetrations with resilient seals which are part of the containment boundary in the airlock structure.
- c. Fuel transfer tube flange seal.
- d. Other containment components which require leak repair in order to meet the acceptance criterion for any integrated leakage rate test.

B. Acceptance Criterion

- 1. The total leakage from items II.A.5 and III.A.3 shall not exceed $0.6 L_a$.
 - a. If at any time it is determined that $0.6 L_a$ is exceeded, repairs shall be initiated immediately. After repair, a retest to confirm conformance to the acceptance criterion of II.B. is required.
 - b. If repairs are not completed and conformance to the acceptance criterion of II.B. is not demonstrated within 48 hours, the reactor shall be taken to cold shutdown conditions until repairs are effected and the local leakage meets this acceptance criterion.
- 2. The leakage from the airlock doors seal test, resulting from the 3 day testing requirement in II.C.1.d, shall be considered acceptable if the leakage sum from the worst door in each airlock, extrapolated to P_a , and added to the total of items II.A.5 and III.A.3, is less than $0.6 L_a$.
 - a. If the total identified in II.B.2, above, exceeds $0.6 L_a$, then the airlock containing the worst door shall be full pressure tested to determine the actual leakage performance.

C. Test Frequency

- 1. Individual penetrations shall be tested during each shutdown for major fuel reloading except as specified in a and b below. In no case shall the interval be greater than two years.
 - a. The containment equipment hatch flange seals and the fuel transfer tube flange seals shall be tested at each shutdown for major fuel reloading or after each time used, if that be sooner.

- b. The air locks shall be tested at 6-month intervals at test pressure not less than P_a .
- c. Personnel airlocks shall be tested at a pressure of no less than P_a following periods when containment integrity is defeated through the use of the airlock.
- d. Personnel airlocks opened during periods when containment integrity is established shall be tested within 3 days after being opened. Personnel airlocks opened more frequently than once every 3 days shall be tested at least once every 3 days during the period of frequent openings.

III. Type "C" Tests

A Type "C" test measures the leakage across an individual valve or across a group of valves used to isolate an individual penetration through the primary reactor containment as defined in III.A.3.

A. Test

1. Type "C" tests shall be performed at intervals specified in III.D below and at a pressure of not less than P_a .
2. Acceptable methods of testing are by local pressurization and the methods described in II.A.4 above. The pressure shall be applied in the same direction as that when the valve would be required to perform its safety function, unless it can be determined that the results from the tests for a pressure applied in a different direction will provide equivalent or more conservative results. Each valve to be tested shall be closed by normal operation and without any preliminary exercising or adjustments.