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Docket No. 50-336

Mr. W. G. Council, Vice President
Nuclear Engineering & Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06101

Dear Mr. Council:

SUBJECT: SAFETY EVALUATION FOR ENVIRONMENTAL QUALIFICATION OF SAFETY-RELATED ELECTRICAL EQUIPMENT, MILLSTONE NUCLEAR POWER STATION UNIT 2

This letter transmits the Safety Evaluation with attached Technical Evaluation Report (TER) for the Environmental Qualification of Safety-Related Electrical Equipment at your facility. This evaluation is based on your response to our previous Safety Evaluation, dated May 27, 1981, and subsequent submittals dated August 26, 1981 and February 9, 1982. The Safety Evaluation presents the results of the Environmental Qualification Review for safety-related electrical equipment exposed to a harsh environment, in accordance with NRC requirements. You should provide to us by May 20, 1983, your plans for qualification or replacement of the equipment in NRC Categories I.B, II.A, II.B and IV as applicable to your plant (presented in the Technical Evaluation Report) in addition to the justification for continued operation (JCO) required in the near term, and the schedule for accomplishing your proposed corrective actions in accordance with the Equipment Qualification rule (10 CFR 50.49(g) issued on January 21, 1983, Federal Register Volume 48, No. 15, PP 2729-2734).

As indicated in the conclusion section of the Safety Evaluation, we request that you reaffirm the justification for continued operation and within thirty (30) days of receipt of this letter submit information for items in NRC Categories I.B, II.A and IV (presented in the enclosed Technical Evaluation Report) for which justification for continued operation was not previously submitted to the NRC. In addition, for items in NRC Category II.B, justification for continued operation, if not previously provided, is to be provided within ten (10) days of receipt of this letter. Should issues or conflicts exist which prohibit a response in a timely manner, a 10 CFR Part 50.54(f) letter will be issued. Guidelines set forth in paragraph (i) of 10 CFR 50.49 should be considered in your justification for continued operation for your facility.

Upon completion of the plant specific review for all plants, a cross-reference of non-qualified equipment existing in any plant will be conducted by the NRC staff to determine if the same type of equipment exists on other plants and has been declared qualified. Should the cross-reference indicate that such items do exist in your plant, the staff will contact you to reconfirm the qualification of these items for your plant.

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PDR

OFFICE
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DATE

Appendix D of the TER provides a technical review of your statements regarding the justification for continued operation (JCO) that were submitted in the 90-day response to an earlier staff safety evaluation (published in mid-1981). Appendix D is not necessarily applicable to the deficiencies identified in the TER. You should review all JCOs provided to date to ensure that a JCO exists for all equipment which may not be qualified.

As a separate action to your response to this letter, the Commission has promulgated an Equipment Qualification rule which in Paragraph (g) of the rule requires that by May 20, 1983, licensees identify electrical equipment important to safety within the scope of the rule that is already qualified and submit a schedule for the qualification or replacement of the remaining electrical equipment within the scope of the rule in accordance with the qualification deadline specified in paragraph (g). The submittal required by the rule should specifically indicate whether your previous submittals comply with paragraphs (a) and (b) of 10 CFR 50.49. In addition, you are requested to discuss the methods used to identify the equipment covered by paragraph 10 CFR 50.49(b)(2) and to establish any qualification programs not previously described for such equipment. This rule is now the focus the staff intends to rely upon in resolving the EQ program.

The Technical Evaluation Report contains certain identified information which you have previously claimed to be proprietary. We request that you inform us as indicated in the proprietary review section of the Safety Evaluation whether any portions of the identified pages still require proprietary protection. It should be noted that the NRC's policy on proprietary information, as specified in SECY 81-119, is that summary data on equipment qualification testing will not be treated as proprietary by the NRC. This information shall be submitted within thirty (30) days of receipt of this letter. A general guideline is enclosed.

Questions regarding the enclosed SE and attached TER, especially the findings and actions required, should be directed through the NRC Project Manager for this facility.

Sincerely,

Original signed by
Robert A. Clark

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosures:

- 1. Safety Evaluation with attached FRC TER
- 2. Proprietary Review Guidelines

cc w/o Proprietary FRC
TER: See next page

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