

U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Reports No. 50-266/82-20(DETP); 50-301/82-20(DETP)

Docket Nos. 50-266; 50-301

Licenses No. DPR-24; DPR-27

Licensee: Wisconsin Electric Power Company
231 West Michigan
Milwaukee, WI 53201

Facility Name: Point Beach Nuclear Power Plant, Units 1 and 2

Inspection At: Point Beach Site, Two Creeks, WI

Inspection Conducted: September 14-16, 1982

Inspectors: R. A. Hasse

J. C. Hawk for

9/29/82

J. C. Hawk for
J. M. Peschel

9/29/82

Approved By: F. C. Hawkins, Chief
Management Programs Section

J. C. Hawk

9/29/82

Inspection Summary

Inspection on September 14-16, 1982 (Reports No. 50-266/82-20(DETP);
50-301/82-20(DETP))

Areas Inspected: Surveillance and calibration procedures and performance history (Operations and I&C) for Units 1 and 2. This inspection involved a total of 28 onsite inspector-hours by two NRC inspectors.

Results: No items of noncompliance or deviations were identified.

DETAILS

1. Persons Contacted

Wisconsin Electric Power Company

*J. Zach, Manager, PBNP
*R. Link, Superintendent, Engineering, Quality, and Regulatory Services
*F. Zeman, Supervisor, Staff Services
G. Maxfield, Superintendent, Operations
G. Rau, Acting Superintendent, I&C
J. Bauer, Technical Specialist, Operations
E. LeClair, Supervisor, I&C

U.S. NRC

*R. Hague, Senior Resident Inspector

*Denotes persons attending exit meeting of September 16, 1982.

2. Surveillance Testing

a. Procedures

The inspectors reviewed current revisions of several surveillance test procedures to determine whether the required reviews and approvals were obtained, prerequisites and acceptance criteria for the tests were specified, provisions for restoring systems or components to operation following testing were included, and if applicable Technical Specification surveillance requirements were met. The following surveillance test procedures were reviewed:

IT - 05 (Unit 1) "Inservice Testing of Spray Pumps and Eductor
IT - 06 (Unit 2) Supply Check Valves"

IT - 01 (Unit 1) "Inservice Testing of High Head Safety Injection
IT - 02 (Unit 2) Pumps"

TS - 1 "Technical Specification Test of Emergency Diesel Generator 3D."

Acceptance criteria are not included in these procedures. However, because the licensee is committed to ANSI 18.7 only (not Regulatory Guide 1.33), the inclusion of acceptance criteria in the procedure is not required. Acceptance criteria are kept in the operations office and the results of the test compared to the acceptance criteria by the responsible individual.

b. Surveillance Tests

The inspectors reviewed surveillance test results to verify that tests were performed at the required periodicity, completed test results received the required reviews and approval, and that appropriate corrective action was initiated and completed in a timely

manner when acceptance criterion were not met. The inspectors reviewed 100% of the following surveillance test results obtained over the time period January through August, 1982:

- IT - 05 "Inservice Testing of Spray Pumps and Eductor Supply
- IT - 06 Check Valves" (Pumps only)

- IT - 01 "Inservice Testing of High Head Safety Injection Pumps"
- IT - 02

- TS - 1 "Technical Specification Testing of Emergency Diesel Generator 3D."

c. Inservice Testing

The licensee submitted its original inservice testing program and requests for technical specification changes to the Office of Nuclear Reactor Regulation (NRR) in May, 1977. Additional Technical Specification changes were submitted to NRR in November, 1978. The licensee is presently following an inservice testing program that was submitted to NRR on July 21, 1980. The program is based upon the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 1974 edition and addenda through the summer of 1975, with relief requests for code requirements, considered by the licensee, to be impractical. In a letter dated September 31, 1980, NRR has given the licensee interim approval to implement its submitted program, except where current technical specifications are more restrictive.

Subsequently, the licensee submitted its program for the second 120 month interval to the NRC on February 10, 1981. In the submittal cover letter the licensee stated that the new program would be implemented 90 days after NRR approved the program and associated technical specification changes requested in May 1977 and November 1978. NRR has not provided approval of the program and/or technical specification changes.

A review was conducted of the program the licensee is implementing to satisfy the requirements of 10 CFR 50.55a with regards to inservice testing of pumps and valves. Test documentation for selected pumps showed that the tests were being performed at the frequencies referenced in the July 21, 1980 program.

The review and approval of the inservice test program is considered an open item. (50-266/82-02-01; 50-301/82-02-01).

3. Instrument Calibration

The inspectors reviewed I&C calibration results taken during the fall 1981 refueling outage for Unit 1 in accordance with Procedure ICP 4.1, "Calibration Procedure - Reactor Protection and Safeguards Analog Racks." Results were reviewed to determine if specified calibration tolerances were met and if all calibrations required by the procedure were performed. No deficiencies were noted. The status of several instruments used for these calibrations was also determined and traced back to NBS Standards.

4. Exit Interview

The inspectors met with licensee representatives denoted in Paragraph 1 at the conclusion of the inspection on September 16, 1982. The inspectors summarized the scope and findings of the inspection.