50-170



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DEFENSE NUCLEAR AGENCY

ARMED FORCES RADIOBIOLOGY RESEARCH INSTITUTE BETHESDA, MARYLAND 20814

000297

APR 4 1983

Chief, Standardization & Special Projects Branch Division of Licensing Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, DC 20555

Dear Sir:

As part of an Institute-wide ventilation system upgrade, AFRRI is planning a modification to the reactor building ventilation system. The original ventilation system was described in the 1962 Final Safety Analysis Report and upgraded in 1967.

10 CFR 50.59 requires that a modification made to portions of a licensed facility that are described in the facility Safety Analysis Report be documented with a written analysis to provide the basis for a determination that the change does not involve an unreviewed safety question. An analysis was performed with the results submitted to the AFRRI Reactor and Radiation Facility Safety Committee. The findings of both the analysis and the committee was that there were no unreviewed safety questions and that, in fact, the margin of safety was increased.

The proposed modification will require minor administrative changes to the current reactor license, Technical Specification (R-84), and the Safety Analysis Report and Technical Specifications now pending before the USNRC in a relicensing effort (Docket-50-170). It is requested that an amendment be granted to the current license #R-84 to include the page changes given in Attachment A #1. Also attachment A #2 is to be incorporated into the proposed Technical Specifications and attachment A #3 into the proposed Safety Analysis Report now before the NRC as part of our relicensing.

As this system is part of an Institute-wide upgrade project, a precise effective date cannot be specified. We therefore request that the change be effective upon completion of installation and testing of the new system. A complete copy of the modification analysis is available at AFRRI as Technical Note #83-1, a copy of which, along with the testing results, will be submitted with the AFRRI annual report when the upgrade is completed.

В304060274 В30404 PDR ADOCK 05000170 P PDR The point of contact for information concerning this request is Mark Moore, Reactor Operations Supervisor, AFRRI Reactor Facility, telephone (202) 295-1290.

Sincerely,

BOBBY R. ADCOCK Colonel, MSC, USA Director

Cy Furn: Division of Engineering & Technical Programs U.S. Nuclear Regulatory Commission, Region I 631 Park Avenue King of Prussia, PA 19406

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ATTACHMENT A

The following administrative changes are requested as amendment number 18 to the AFRRI reactor license R-84. These changes have been reviewed and do not represent an unreviewed safety question. The technical specifications changes requested reflect minor administrative and wording changes and are not changes in reactor safety systems.

It is requested that these proposed changes become effective upon installation and testing of the modified reactor ventilation system.

CURRENT TECHNICAL SPECIFICATIONS

Change 1:

Section 1:A.1, first sentence, replace the sentence with the following, "Areas of the reactor building including; the reactor room, the control room suite, ER rooms 1, 2, and 3, and the warm storage-decontamination area shall have a separate branch of the AFRRI ventilating system."

PROPOSED TECHNICAL SPECIFICATION (on file with USNRC as part of relicensing request Docket #50-170)

Change 1:

IV Section 5, specification a, first sentence, replace the sentence with the following, "Areas of the reactor building including; the reactor room, the control room suite, ER rooms 1,2, and 3, and the warm storage-decontamination area shall have a separate branch of the AFRRI ventilation system."

PROPOSED SAFETY ANALYSIS REPORT (on file with the USNRC as part of relicensing request Docket 50-170)

Change 1:

Replace section 3.2.2 with attachment C.

Change 2:

Remove section 3.2.4, paragraph 2. (This paragraph is included in attachment C included in change 1.)

Change 3:

Section 3.2.4 paragraph three sentence two, replace with: "The high radiation alarm set point of the reactor room primary CAM will initiate automatic closure of the reactor room dampers."

Change 4:

Replace figure 3-5 with inclosed figure 3-5A.