

(7590-01)

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)
FLORIDA POWER CORPORATION, ET AL) Docket No. 50-302
(Crystal River Unit No. 3 Nuclear)
Generating Plant))
)

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

I.

The Florida Power Corporation (the licensee) and eleven other co-owners are the holders of Facility Operating License No. DPR-72 which authorizes the operation of the Crystal River Unit No. 3 Nuclear Generating Station (the facility) at steady-state power levels not in excess of 2544 megawatts thermal. The facility is a pressurized water reactor (PWR) located at the licensee's site in Citrus County, Florida.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial

additional protection in the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or after July 1, 1981. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982. Subsequently, on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff had proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

III.

Florida Power Corporation responded to the Generic Letter 82-05 by letters dated April 14, June 18 (2 letters), June 30, September 8 and 14, 1982; Florida Power Corporation responded to the Generic Letter 82-10 by letters dated June 4, and September 17, 1982. In these submittals, Florida Power Corporation confirmed that some of the items identified in the Generic Letters had been completed and made firm commitments to complete the remainder. The attached Tables summarizing the licensee's schedular commitments or status were developed by the staff from the Generic Letters and the licensee-provided information.

There are six items from Generic Letter 82-10 that, as noted in the Table (Attachment 2), have licensee schedules to be determined and are therefore not included in this Order. Some of the items addressed in this Order are considered by the licensee to be completed or to require no modifications. The staff's evaluation of the licensee's delays for the remaining items is provided herein:

11.3.2 Plant Shielding

This item will be delayed by the licensee and will be completed during the next refueling outage, R.O.IV. R.O.IV is scheduled to start in March 1983 and to continue for about 16 weeks. The delay was caused

by late delivery of valve operator starters and manual valve extenders. The manual valve extenders have been received and are scheduled to be installed prior to November 1982. In addition, a six day outage will be required to complete final hookup of the valve operators. The licensee has stated that temporary shielding methods and procedures for manual action will be used as a compensatory measure until final installation has been completed.

II.B.3 Post Accident Sampling

This item will be delayed by the licensee and will be completed by December 1983. Delay in installation was caused by late delivery of a number of items of equipment and subsequent interface design problems in the plant. Also contributing to the delay was seismic justification deficiencies for the Target Rock valves. As compensatory measures, plant procedures for handling and analyzing post-accident samples were developed and implemented. These will be utilized until the post-accident sampling equipment becomes operational.

II.E.1.2 Auxiliary Feedwater (AFW) Initiation and Flow Indication

Safety grade AFW initiation and flow indication is part of an expanded AFW system upgrade called Emergency Feedwater Indication and Control (EFIC). The upgrade will include modifications (Safety Grade Control) that are beyond those suggested in NUREG-0737 and accordingly the expanded upgrade will not be completed until R.O.V, scheduled to begin in November 1984. Contributing to the delay has been the scheduled delivery dates for equipment such as

the qualified Target Rock valves, qualified steam generator level transmitters and the EFIC cabinets. As an interim measure pending completion of the EFIC system, the licensee has installed and has operational a control grade system that is single-failure proof.

II.F.1 (1-6) Post-Accident Monitoring (6 items)

The licensee will delay three items, II.F.1(1), (2) and (6) for completion by December 1983; and three items, II.F.1(3), (4) and (5) for completion during R.O.IV, scheduled for March 1983. For Items II.F.1(1), Noble Gas Effluent Radiological Monitor, and II.F.1(2), Continuous Sampling of Plant Effluent, development problems in background shielding requirements and related instrument sensitivities resulted in the design of additional shielding and changes in placement of the modules. Also late delivery schedules for vendor and other auxiliary equipment caused the schedule delay. For Item II.F.1(6), Containment Hydrogen Monitor, the design required modification when the original specified hydrogen monitor was replaced with a monitor upgraded for reliability and meeting the regulatory requirements. The redesign requirements along with late delivery schedules of vendor and other auxiliary equipment caused the delay in completion of this item. For II.F.1(3), Containment High Range Monitor, all equipment is onsite but a plant shutdown is required to complete the installation. For Items II.F.1(4), Containment Pressure Monitor, and II.F.1(5), Containment Water Level

Monitor, design modifications and late delivery of equipment contributed to the delay in installation. In addition, a plant outage is required to complete the installation. For Items II.F.1(1) through II.F.1(5), as a compensatory measure, the licensee has currently in operation monitors with shorter ranges than those being installed. However, there is only a small likelihood that the extended range will be necessary in the interim period. For II.F.1(6), as a compensatory measure, the licensee has made provisions for taking grab samples of the reactor containment atmosphere for analysis at on-site facilities.

We find, based on the above evaluation, that: 1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; 2) there is good cause for the several delays (unexpected design complexity, interface problems, and equipment delays); and 3) as noted above, interim compensatory measures have been provided.

In view of the foregoing, I have determined that these modifications and actions are required in the interest of public health and safety and, therefore, the licensee's commitments should be confirmed by Order.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensee shall:

Implement and maintain the specific items described in the Attachments to this Order in the manner described in the licensee's submittals noted in Section III herein no later than the dates in the Attachments.

V.

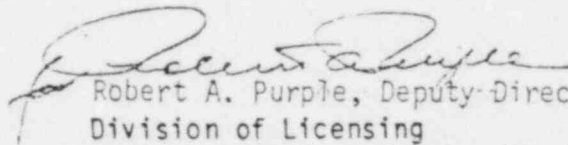
The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensee, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Purple, Deputy Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,
this 14th day of March
1963.

Attachments:

1. Licensee's Commitments on Applicable
NUREG-0737 Requirements from Generic
Letter 82-05
2. Licensee's Commitments on Applicable
NUREG-0737 Requirements from Generic
Letter 82-10

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
I.A.3.1	Simulator Exams	10/1/81	Include simulator exams in licensing examinations	Complete
II.B.2	Plant Shielding	1/1/82	Modify facility to provide access to vital areas under accident conditions	Refueling outage IV (3/83)
II.B.3	Post-Accident Sampling	1/1/82	Install upgrade post-accident sampling capability	12/83
II.B.4	Training for Mitigating Core Damage	10/1/81	Complete training program	Complete
II.E.1.2	Aux. Feedwater Initiation & Flow Indication	7/1/81	Modify instrumentation to level of safety grade	Refueling outage V (11/84)
II.E.4.2	Containment Isolation Dependability	7/1/81	Part 5-lower containment pressure setpoint to level compatible w/normal operation	Complete
II.F.1	Accident Monitoring	7/1/81	Part 7-isolate purge & vent valves on radiation signal	Complete
			(1) Install noble gas effluent monitors	12/83
II.F.1	Accident Monitoring	1/1/82	(2) Provide capability for effluent monitoring of iodine	Refueling outage IV (3/83)
			(3) Install in-containment radiation-level monitors	Refueling outage IV (3/83)
II.F.1	Accident Monitoring	1/1/82	(4) Provide continuous indication of containment pressure	Refueling outage IV (3/83)
			(5) Provide continuous indication of containment water level	Refueling outage IV (3/83)
II.F.1	Accident Monitoring	1/1/82	(6) Provide continuous indication of hydrogen concentration in containment	12/83
II.K.2.10	Safety Grade Trips	7/1/81	Install anticipatory reactor trips	Complete

*where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility.

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-10

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*
I.A.1.3.1	Limit Overtime	10/1/82 per Gen. Ltr. 82-12 dtd. 6/15/82	Revise administrative procedures to limit overtime in accordance w/NRC Policy Statement issued by Generic Ltr. No. 82-12, dtd. June 15, 1982	Complete
I.A.1.3.2	** Minimum Shift Crew	To be superseded by Proposed Rule	To be addressed in the Final Rule on Licensed Operator Staffing at Nuclear Power Units.	To be addressed when Final Rule is issued.
I.C.1	** Revise Emergency Procedures	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be determined
II.D.1.2	RV and SV Test Programs	7/1/82	Submit plant specific rpts. on relief & safety valve program.	Complete
II.D.1.3	Block Valve Test Program	7/1/82	Submit rept. of results of test program.	Complete
II.K.3.30 & 31	** SBLOCA Analysis	1 yr. after staff approval of model	Submit plant specific analyses	To be determined following staff approval of model
III.A.1.2	** Staffing Levels for Emergency Situations	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be Determined
III.A.1.2	** Upgrade Emergency Support Facilities	" " " "	" " " "	" " " "
III.A.2.2	** Meteorological Data	" " " "	" " " "	" " " "
III.D.3.4	Control Room Habitability	To be determined by licensee	Modify facility as identified by licensee study.	Complete

*Where completion date refers to a refueling outage (the estimated date when the outage begins), the item will be completed prior to the restart of the facility.

**Not Part of Confirmatory Order