



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Power Authority of the State of New York dated August 12, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

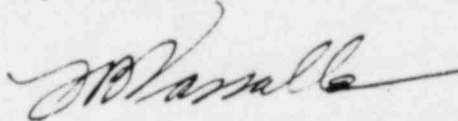
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 70, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment became effective August 13, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 8, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
76c	76c
142a	142a
-	142b
143	143
-	143a
-	143b

NOTES FOR TABLE 3.2-5 (CONTINUED)

2. In the event that all indications of this parameter is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.
3. Three (3) indicators from level instrument channel A, B, & C. Channel A or B are utilized for feedwater control, reactor water high and low level alarms, recirculation pump runback. High level trip of main turbine and feedwater pump turbine utilizes channel A, B, & C.
4. One (1) recorder utilized the same level instrument channel as selected for feedwater control.
5. Three (3) indicators from reactor pressure instrument channel A, B, & C. Channel A or B are utilized for feedwater control and reactor pressure high alarm.
6. One (1) recorder. Utilizes the same reactor pressure instrument channel as selected for feedwater control.
7. The position of each of the 137 control rods is monitored by the Rod Position Information System. For control rods in which the position is unknown, refer to Paragraph 3.3.A.
8. Neutron monitoring operability requirements are specified by Table 3.1-1 and Paragraph 3.3.B.4.
9. A minimum of 3 IRM or 2 APRM channels respectively must be operable (or tripped) in each safety system.
10. Each Safety Relief Valve is equipped with two acoustical detectors of which one is in service and a backup thermocouple detector. In the event that a thermocouple is inoperable SRV performance shall be monitored daily with the associated acoustical detector.
11. From and after the date that none of the acoustical detectors is operable but the thermocouple is operable, continued operation is permissible until the next outage in which a primary containment entry is made. Both acoustical detectors shall be made operable prior to restart.
12. In the event that both primary and secondary indications of this parameter for any one valve are disabled and neither indication can be restored in forty-eight (48) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in twelve (12) hours and in a Cold Shutdown within the next twenty-four (24) hours.
13. From and after the date that the minimum number of operable instrument channels is one less than the minimum number specified for each parameter, continued operation is permissible during the succeeding 7 days unless the minimum number specified is made operable sooner.

E. Safety and Safety/Relief Valves

1. During reactor power operating conditions and prior to startup from a cold condition, or whenever reactor coolant pressure is greater than atmosphere and temperature greater than 212°F,
the safety mode of all safety/relief valves shall be operable, except as specified by Specification 3.6.E.2. The Automatic Depressurization System valves shall be operable as required by Specification 3.5.D.

E. Safety and Safety/Relief Valves

1. At least one half of all safety/relief valves shall be bench checked or replaced with bench checked valves once each operating cycle. The safety/relief valve settings shall be set as required in Specification 2.2.B. All valves shall be tested every two operating cycles.

E. Safety and Safety/Relief Valves

1. During reactor power operating conditions and prior to startup from a cold condition, or whenever reactor coolant pressure is greater than atmosphere and temperature greater than 212^oF, the safety mode of all safety/relief valves shall be operable, except as specified by Specification 3.6.E.2. The Automatic Depressurization System valves shall be operable as required by Specification 3.5.D.
2. Reactor operation may continue with one safety/relief valve inoperable. From and after the date that two safety/relief valves are made or found inoperable, continued reactor operation is permissible only during the succeeding 7 days, unless one valve is made operable.

E. Safety and Safety/Relief Valves

1. At least one half of all safety/relief valves shall be bench checked or replaced with bench checked valves once each operating cycle. The safety/relief valve settings shall be set as required in Specification 2.2.B. All valves shall be tested every two operating cycles.

(cont'd)

2. a. From and after the date that the safety valve function of one safety/relief valve is made or found to be inoperable, continued operation is permissible only during the succeeding 30 days unless such valve is sooner made operable.
- b. From and after the time that the safety valve function on two safety/relief valves is made or found to be inoperable, continued reactor operation is permissible only during the succeeding 7 days unless such valves are sooner made operable.
3. If Specification 3.6B.1 and 3.6B.2 are not met, the reactor shall be placed in a cold condition within 24 hr.
4. Low power physics testing and reactor operator training shall be permitted with inoperable components as specified in Item B.2 above, provided that reactor coolant temperature is $\leq 212^{\circ}\text{F}$ and the reactor vessel is vented or the reactor vessel head is removed.

IPP

4.6 (cont'd)

2. At least one safety/relief valve shall be disassembled and inspected once/operating cycle.
3. The integrity of the safety/relief valve bellows shall be continuously monitored.
 - a. The bellows monitoring pressure switches shall be removed and bench checked once/operating cycle. Modified safety/relief valves with two-stage assemblies do not have a bellows arrangement and are, therefore, not subject to this requirement.
4. The integrity of the nitrogen system and components which provide manual and ADS actuation of the safety/relief valves shall be demonstrated at least once every 3 months.

3. If Specification 3.6.E.1 and 3.6.E.2 are not met the reactor shall be placed in a cold condition within 24 hr.
4. Low power physics testing and reactor operator training shall be permitted with inoperable components as specified in 3.6.E.2, and provided that reactor coolant temperature is $\leq 212^{\circ}\text{F}$ and the reactor vessel is vented or the vessel head is removed.

2. At least one safety/relief valve shall be disassembled and inspected once/operating cycle.
3. Deleted
4. The integrity of the nitrogen system and components which provide manual and ADS actuation of the safety/relief valves shall be demonstrated at least once every 3 months.

3.6 (cont'd)

5. If, for a period of longer than 24 hours, the temperature of any safety/relief discharge pipe is more than 40°F above its steady state value, or the acoustical monitor reading of any safety/relief valve discharge pipe is more than 3 times greater than its steady state value, the following actions shall be taken:
- a. a report shall be issued in accordance with 6.9.A.4.1 which addresses the actions that have been taken or a schedule of actions to be taken.
 - b. an engineering evaluation shall be performed justifying continued operation for the corresponding increase in temperature or acoustical monitor reading.
 - c. the affected safety/relief valve shall be removed at the next cold shutdown of 72 hours or more, tested in the as-found condition, and recalibrated as necessary prior to reinstallation.
 - d. NRC approval of the engineering evaluation specified in 3.6.E.5.b above shall be obtained prior to continuing power operation for more than 90 days after the initial discovery of the 40°F increase in temperature or the factor of 3 increase in acoustical monitor reading.

The steady state values of temperature and acoustical monitor readings shall be as measured after 5 days of steady state power operation.